

June 18, 1991

Docket Nos. 50-277  
and 50-278

Mr. George J. Beck  
Manager-Licensing, MC 5-2A-5  
Philadelphia Electric Company  
Nuclear Group Headquarters  
Correspondence Control Desk  
P.O. Box No. 195  
Wayne, Pennsylvania 19087-0195

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GHill(8), P1-37  
Wanda Jones, 7103

Dear Mr. Beck:

SUBJECT: REVISE ISI TECHNICAL SPECIFICATIONS PER GENERIC LETTER 88-01, PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3 (TSCR No. 86-08) (TAC NOS. 79749 AND 79750)

The Commission has issued the enclosed Amendments Nos. 161 and 163 to Facility Operating License Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Unit Nos. 2 and 3. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated December 31, 1990.

These amendments revise the TSs to add the requirement requested by Generic Letter (GL) 88-01 that the Augmented Inservice Inspection Program for piping be performed in accordance with the staff positions on schedule, methods, personnel and sample expansion. The amendments also make some other minor changes to the same Surveillance Requirement (4.6.G) as described in the Safety Evaluation.

The issuance of these TSs resolves all issues relating to GL 88-01 for Peach Bottom, Units 2 and 3 except for the one issue involving inspection of the Reactor Water Cleanup System welds outside of containment which was discussed in my letter to you of April 24, 1991.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

Original signed by  
Richard J. Clark

Richard J. Clark, Project Manager  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

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PDR ADDCK 05000277  
P PDR

Enclosures:

1. Amendment No. 161 to DPR-44
  2. Amendment No. 163 to DPR-56
  3. Safety Evaluation
- cc w/enclosures:  
See next page

OFC	: PDI-2/LA	: PDI-2/PM	: PDI-2/PM	: OGC	: PDI-2/D
NAME	: MO'Brien	: RClark	: PMilano	: WButler	
DATE	: 5/30/91	: 05/30/91	: 1/91	: 6/15/91	: 6/12/91

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

June 18, 1991

Docket Nos. 50-277  
and 50-278

Mr. George J. Beck  
Manager-Licensing, MC 5-2A-5  
Philadelphia Electric Company  
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Correspondence Control Desk  
P.O. Box No. 195  
Wayne, Pennsylvania 19087-0195

Dear Mr. Beck:

SUBJECT: REVISE ISI TECHNICAL SPECIFICATIONS PER GENERIC LETTER 88-01, PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3 (TSCR No. 86-08) (TAC NOS. 79749 AND 79750)

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These amendments revise the TSs to add the requirement requested by Generic Letter (GL) 88-01 that the Augmented Inservice Inspection Program for piping be performed in accordance with the staff positions on schedule, methods, personnel and sample expansion. The amendments also make some other minor changes to the same Surveillance Requirement (4.6.G) as described in the Safety Evaluation.

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A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

A handwritten signature in cursive script that reads "Richard J. Clark".

Richard J. Clark, Project Manager  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 161 to DPR-44
2. Amendment No. 163 to DPR-56
3. Safety Evaluation

cc w/enclosures:  
See next page

Mr. George J. Beck  
Philadelphia Electric Company

Peach Bottom Atomic Power Station,  
Units 2 and 3

cc:

J. W. Durham, Sr., Esquire  
Sr. V.P. & General Counsel  
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Philadelphia, Pennsylvania 19101

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Harrisburg, Pennsylvania 17108-1880

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Peach Bottom Atomic Power Station  
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Bureau of Radiation Protection  
Pennsylvania Department of  
Environmental Resources  
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Harrisburg, Pennsylvania 17120

Philadelphia Electric Company  
ATTN: Regulatory Engineer, A1-2S  
Peach Bottom Atomic Power Station  
Route 1, Box 208  
Delta, Pennsylvania 17314

Board of Supervisors  
Peach Bottom Township  
R. D. #1  
Delta, Pennsylvania 17314

Resident Inspector  
U.S. Nuclear Regulatory Commission  
Peach Bottom Atomic Power Station  
P.O. Box 399  
Delta, Pennsylvania 17314

Public Service Commission of Maryland  
Engineering Division  
ATTN: Chief Engineer  
231 E. Baltimore Street  
Baltimore, MD 21202-3486

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, Pennsylvania 19406

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Power Plant and Environmental  
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Annapolis, Maryland 21401

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Baltimore, Maryland 21201



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 161  
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company, et. al. (the licensee) dated December 31, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

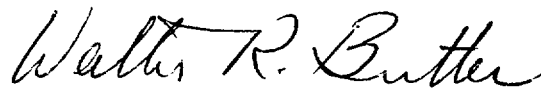
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 161, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director  
Project Directorate I-2  
Division of Reactor Projects - I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: June 18, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 161

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

<u>Remove</u>	<u>Insert</u>
vi	vi
149b	149b
150	150
161	161
162	162
163	163

PBAPS  
LIST OF TABLES

<u>Table</u>	<u>Title</u>	<u>Page</u>
4.2.B	Minimum Test and Calibration Frequency for CSCS	81
4.2.C	Minimum Test and Calibration Frequency for Control Rod Blocks Actuation	83
4.2.D	Minimum Test and Calibration Frequency for Radiation Monitoring Systems	84
4.2.E	Minimum Test and Calibration Frequency for Drywell Leak Detection	85
4.2.F	Minimum Test and Calibration Frequency for Surveillance Instrumentation	86
4.2.G	Minimum Test and Calibration Frequency for Recirculation Pump Trip	88
3.5.K.2	DELETED	133d
3.5.K.3	DELETED	133d
4.6.1	DELETED	150
3.7.1	Primary Containment Isolation Valves	179
3.7.2	Testable Penetrations with Double O-Ring Seals	184
3.7.3	Testable Penetrations with Testable Bellows	184
3.7.4	Primary Containment Testable Isolation Valves	185
4.8.1	Radioactive Liquid Waste Sampling and Analysis	216b-1
4.8.2	Radioactive Gaseous Waste Sampling and Analysis	216c-1
4.8.3.a	Radiological Environmental Monitoring Program	216d-1
4.8.3.b	Reporting Levels for Radioactivity by Concentrations in Environmental Sample	216d-5

LIMITING CONDITION FOR OPERATION

## 3.6.G Structural Integrity

The structural integrity of the primary system boundary shall be maintained at the level required by the original acceptance standards throughout the life of the station. The reactor shall be maintained in a Cold Shutdown condition until each indication of a defect has been investigated and evaluated.

SURVEILLANCE REQUIREMENTS

## 4.6.G Structural Integrity

1. Inservice inspection of ASME Code Class 1, 2, and 3 components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR Part 50, Section 50.55a(g), except where relief has been granted by the Commission pursuant to 10 CFR Part 50, Section 50.55a(g)(6)(i) and (a)(3).
2. The Augmented Inservice Inspection Program for piping shall be performed in accordance with the staff positions on schedule, methods, personnel and sample expansion as provided by NRC Generic Letter 88-01 or in accordance with alternate measures approved by the NRC staff.
3. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.



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3.6.G & 4.6.G BasesStructural Integrity

The inspection programs for ASME Code Class 1, 2 and 3 components ensure that integrity of these components will be maintained at an acceptable level throughout the life of the plant.

The Inservice Inspection Program was developed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code 1970 Edition, except where accessibility for inspection was not provided and where it was impractical to modify the original design.

The Inservice Inspection Program for ASME Code Class 1, 2 and 3 components will be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable addenda as required by 10 CFR Part 50.55a(g) except where specific written relief has been granted by the NRC pursuant to 10 CFR Part 50.55a(g)(6)(i) and (a)(3).

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WASHINGTON, D.C. 20555

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 163  
License No. DPR-56

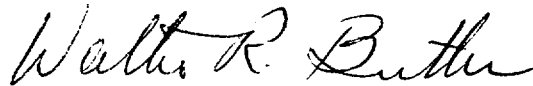
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company, et. al. (the licensee) dated December 31, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 163, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director  
Project Directorate I-2  
Division of Reactor Projects - I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: June 18, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 163

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

<u>Remove</u>	<u>Insert</u>
vi	vi
149b	149b
150	150
161	161
162	162
163	163

PBAPS  
LIST OF TABLES

<u>Table</u>	<u>Title</u>	<u>Page</u>
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4.8.3.a	Radiological Environmental Monitoring Program	216d-1
4.8.3.b	Reporting Levels for Radioactivity Concentrations in Environmental Samples	216d-5



LIMITING CONDITION FOR OPERATION

## 3.6.G Structural Integrity

The structural integrity of the primary system boundary shall be maintained at the level required by the original acceptance standards throughout the life of the station. The reactor shall be maintained in a Cold Shutdown condition until each indication of a defect has been investigated and evaluated.

SURVEILLANCE REQUIREMENTS

## 4.6.G Structural Integrity

1. Inservice inspection of ASME Code Class 1, 2, and 3 components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR Part 50, Section 50.55a(g), except where relief has been granted by the Commission pursuant to 10 CFR Part 50, Section 50.55a(g)(6)(i) and (a)(3).
2. The Augmented Inservice Inspection Program for piping shall be performed in accordance with the staff positions on schedule, methods, personnel and sample expansion as provided by NRC Generic Letter 88-01 or in accordance with alternate measures approved by the NRC staff.
3. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.

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3.6.G & 4.6.G BasesStructural Integrity

The inspection programs for ASME Code Class 1, 2 and 3 components ensure that integrity of these components will be maintained at an acceptable level throughout the life of the plant.

The Inservice Inspection Program was developed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code 1970 Edition, except where accessibility for inspection was not provided and where it was impractical to modify the original design.

The Inservice Inspection Program for ASME Code Class 1, 2 and 3 components will be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable addenda as required by 10 CFR Part 50.55a(g) except where specific written relief has been granted by the NRC pursuant to 10 CFR Part 50.55a(g)(6)(i) and (a)(3).

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 161 AND 163 TO FACILITY OPERATING

LICENSE NOS. DPR-44 and DPR-56

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3

DOCKET NOS. 50-277 AND 50-278

1.0 INTRODUCTION

By letter dated December 31, 1990, the Philadelphia Electric Company, Public Service Electric & Gas Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) submitted a request for changes to the Peach Bottom Atomic Power Station, Unit Nos. 2 and 3, Technical Specifications (TS). The requested changes would modify Surveillance Requirement 4.6.G and the associated Bases to require that the Augmented Inservice Inspection Program for piping be performed in accordance with the staff positions on schedule, methods, personnel and sample expansion in NRC Generic Letter (GL) 88-01, "NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping". The proposed amendments would also make some other minor changes (described below) in the same Surveillance Requirement (4.6.G) to delete obsolete provisions or to enhance clarity and consistency.

Discussion

NRC GL 88-01, issued January 25, 1988, provided guidance in the form of NRC positions regarding Intergranular Stress Corrosion Cracking (IGSCC) problems in Boiling Water Reactor (BWR) piping made of austenitic stainless steel that is four (4) inches or larger in nominal diameter and contains reactor coolant at a temperature above 200 degrees F during reactor power operation regardless of ASME Code classification. NRC GL 88-01 requested licensees of operating BWRs and holders of construction permits for BWRs to provide information regarding conformance with the NRC positions. Two of the items which the GL requested licensees to address were; 1) a TS change to include a statement in the TS section on inservice inspection (ISI) that the ISI program for piping covered by the scope of NRC GL 88-01 will be in conformance with the NRC positions on schedule, methods and personnel, and sample expansion included in the GL, and 2) confirmation of the licensee's plans to ensure that the TSs related to leakage detection will be in conformance with the NRC positions on leak detection included in the GL. The NRC position on leakage detection specifically stated that unidentified leakage be limited to an increase of 2 gpm over a 24 hour period, and that leakage be monitored every eight (8) hours.

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#### 4.0 EVALUATION

We previously completed an evaluation of PECO's programs to meet the 13 staff positions and other guidance in GL 88-01. By letters dated March 27, 1990 and April 24, 1991, we informed PECO that their programs were fully acceptable and satisfied all other requirements in GL 88-01 except for the TSs on ISI and the program for inspection of welds in the Reactor Water Cleaning System outside of containment. These amendments address the TSs on ISI.

The licensee has proposed the following changes to the TSs in conformance with the wording and guidance in GL 88-01:

- (1) Delete existing Surveillance Requirement 4.6.G on page 149b.
- (2) Delete existing Table 4.6.1 entitled "In-service Inspection Program for Peach Bottom, Units 2 and 3" on page 150
- (3) Insert proposed Surveillance Requirement 4.6.G.1 on page 149b which states:

"Inservice inspection of ASME Code Class 1, 2 and 3 components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR Part 50, Section 50.55a(g), except where relief has been granted by the Commission pursuant to 10 CFR Part 50, Section 50.55a(g)(6)(i) and (a)(3)."

- (4) Insert proposed Surveillance Requirement 4.6.G.2 on page 149b which states:

"The Augmented Inservice Inspection (ISI) Program for piping shall be performed in accordance with the staff positions on schedule, methods, personnel and sample expansion as provided by NRC Generic Letter 88-01 or in accordance with alternate measures approved by the NRC staff."

- (5) Insert proposed Surveillance Requirement 4.6.G.3 on page 149b which states:

"Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supercede the requirements of any Technical Specification."

- (6) Delete the existing Bases for 3.6.G and 4.6.G on pages 161, 162 and 163 and replace them with a discussion which reflects the proposed specifications.
- (7) Revise Table of Contents page vi to reflect deletion of Table 4.6.1.

The above changes are in accordance with the staff position in GL 88-01. The proposed changes are acceptable and satisfy all of the TS positions in GL 88-01.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change a surveillance requirement. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Richard Clark

Date: June 18, 1991