

November 29 1994

Mr. George A. Hunger,
Director-Licensing, M-32A-5
PECO Energy Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box No. 195
Wayne, Pennsylvania 19087-0195

SUBJECT: GRANTING OF TECHNICAL SPECIFICATIONS CHANGE REQUEST 93-28, PEACH
BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3 (TAC NOS. M89375 AND
M89376)

Dear Mr. Hunger:

The Commission has issued the enclosed Amendments Nos. 199 and 201 to
Facility Operating License Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic
Power Station, Unit Nos. 2 and 3. These amendments consist of changes to the
Technical Specifications (TS) in response to your application dated April 15,
1994.

These amendments 1) correct a typographical error in the Unit 3 TS, 2) reflect
the name change of Philadelphia Electric Company to PECO Energy Company, and
3) implement line-item TS improvements recommended by Generic Letter 93-05,
"Line-Item Technical Specifications Improvements to Reduce Surveillance
Requirements for Testing During Power Operation." Please inform the staff, in
writing, when these amendments have been implemented.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be
included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

/s/

Joseph W. Shea, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-277/278

Enclosures:

1. Amendment No. 199 to DPR-44
2. Amendment No. 201 to DPR-56
3. Safety Evaluation

cc w/encls:
See next page

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JStolz	McCracken, 0-8D1	RJones, 08-E23

*Previously Concurred

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OFC	: PDI-2/LA	: PDI-2/PM	: SRXB/BC*	: OGC*	: PDI-2/D	:
NAME	: MO'Brien	: JShea	: RJones	:	: JStolz	:
DATE	: 11/21/94	: 11/19/94	: 08/09/94	: 08/24/94	: 11/23/94	:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

November 29, 1994

Mr. George A. Hunger, Jr.
Director-Licensing, MC 52A-5
PECO Energy Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box No. 195
Wayne, Pennsylvania 19087-0195

SUBJECT: GRANTING OF TECHNICAL SPECIFICATIONS CHANGE REQUEST 93-28, PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3 (TAC NOS. M89375 AND M89376)

Dear Mr. Hunger:

The Commission has issued the enclosed Amendments Nos. 199 and 201 to Facility Operating License Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Unit Nos. 2 and 3. These amendments consist of changes to the Technical Specifications (TS) in response to your application dated April 15, 1994.

These amendments 1) correct a typographical error in the Unit 3 TS, 2) reflect the name change of Philadelphia Electric Company to PECO Energy Company, and 3) implement line-item TS improvements recommended by Generic Letter 93-05, "Line-Item Technical Specifications Improvements to Reduce Surveillance Requirements for Testing During Power Operation." Please inform the staff, in writing, when these amendments have been implemented.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Joseph W. Shea".

Joseph W. Shea, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-277/278

Enclosures:

1. Amendment No. 199 to DPR-44
2. Amendment No. 201 to DPR-56
3. Safety Evaluation

cc w/encls:
See next page

Mr. George A. Hunger, Jr.
PECO Energy Company

Peach Bottom Atomic Power Station,
Units 2 and 3

cc:

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Sr. V.P. & General Counsel
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Peach Bottom Township
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PHILADELPHIA ELECTRIC COMPANY

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 199
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated April 15, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, pages 1, 2, 3, 4, 4a, and 5 of the Facility Operating License No. DPR-44 are hereby amended by changing Philadelphia Electric Company to PECO Energy Company.
3. Further, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 199, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

4. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



for

John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachments:

1. Pages 1, 2, 3, 4, and 5
of License No. DPR-44
2. Changes to the Technical
Specifications

Date of Issuance: November 29, 1994

*Pages 1, 2, 3, 4, and 5 are attached, for convenience, for the composite license to reflect this change.

ATTACHMENT TO LICENSE AMENDMENT NO. 199

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Facility Operating License (FOL), the Appendix A Technical Specifications, and Appendix B Environmental Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

	<u>Remove</u>	<u>Insert</u>
FOL	1	1
	2	2
	3	3
	4	4
	4a	--
	5	5
Appendix A	99	99
	116	116
	117	117
	216e	216e
	249	249
	256	256
Appendix B	14	14
	41-47	--



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PECO ENERGY COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY
DOCKET NO. 50-277
PEACH BOTTOM ATOMIC POWER STATION, UNIT 2
FACILITY OPERATING LICENSE

License No. DPR-44
Amendment No. 1

1. The Atomic Energy Commission (the Commission) having found that:
 - A. The application for license filed by PECO Energy Company, formerly Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company (the licensees) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I and all required notifications to other agencies or bodies have been duly made;
 - B. Construction of the Peach Bottom Atomic Power Station, Unit 2 (the facility) has been substantially completed in conformity with Construction Permit No. CPPR-37 and the application, as amended, the provisions of the Act and the rules and regulations of the Commission;
 - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - D. There is reasonable assurance: (1) that the activities authorized by this amended operating license can be conducted without endangering the health and safety of the public, and (2) that such activities will be conducted in compliance with the rules and regulations of the Commission;
 - E. PECO Energy Company is technically qualified and the licensees are financially qualified to engage in the activities authorized by this amended operating license in accordance with the rules and regulations of the Commission;
 - F. The licensees have satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission's regulations;
 - G. The issuance of this amended operating license will not be inimical to the common defense and security or to the health and safety of the public;

- H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental costs and considering available alternatives, the issuance of Amendment No. 1 to Facility Operating License No. DPR-44 is in accordance with 10 CFR Part 50, Appendix D, of the Commission's regulations and all applicable requirements of said Appendix D have been satisfied; and
- I. The receipt, possession, and use of by-product and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30 and 70, including 10 CFR Section 30.33, and 70.23 and 70.31.
2. Amendment No. 1 to Facility Operating License No. DPR-44 issued to the PECO Energy Company (PECO), formerly the Philadelphia Electric Company, Public Service Electric and Gas Company (PSE&G), Delmarva Power and Light Company (DP&LC), and Atlantic City Electric Company (ACEC), is hereby amended in its entirety to read as follows:
- A. This amended license applies to the Peach Bottom Atomic Power Station, Unit 2, a single cycle, forced circulation, boiling water nuclear reactor and associated equipment (the facility), owned by the licensees and operated by PECO Energy Company. The facility is located in Peach Bottom, York County, Pennsylvania and is described in the "Final Safety Analysis Report" as supplemented and amended (Amendments 1 through 29) and the Environmental Report as supplemented and amended (Supplements 1 through 6).
- B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses:
- (1) PECO Energy Company, pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess, use, and operate the facility and PSE&G, DP&LC, and ACEC to possess the facility at the designated location in Peach Bottom, York County, Pennsylvania in accordance with the procedures and limitations set forth in this license;
 - (2) PECO Energy Company, pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended, as of December 15, 1975;
 - (3) PECO Energy Company, pursuant to the Act and 10 CFR Parts 30, 40, and 70 to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) PECO Energy Company, pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form for sample analysis or instrument calibration or when associated with radioactive apparatus or components;

(5) PECO Energy Company, pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not to separate, such byproduct and special nuclear material as may be produced by operation of the facility.

C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

(1) Maximum Power Level

PECO Energy Company is authorized to operate the Peach Bottom Atomic Power Station, Unit 2, at steady state reactor core power levels not to exceed 3458 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. are hereby incorporated in the license. PECO Energy Company shall operate the facility in accordance with the Technical Specifications.

(3) The licensees may perform modifications to the Low Pressure Coolant Injection System as described in the licensees' application for license amendment dated July 9, 1975. The licensees shall not operate the facility prior to receipt of the Commission's authorization.

(4) Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Peach Bottom Atomic Power Station, Units 2 and 3, Physical Security Plan," with revisions submitted through December 16, 1987; "Peach Bottom Atomic Power Station, Units 2 and 3 Plant Security Personnel Training and Qualification Plan," with revisions submitted through July 9, 1986; and "Peach Bottom Atomic Power Station, Units 2 and 3 Safeguards Contingency Plan," with revisions submitted through March 10, 1981. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

(5) The licensee shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility, and as approved in the NRC SER dated May 23, 1979 and Supplements dated August 14, September 15,

October 10 and November 24, 1980, and in the NRC SERs dated September 16, 1993 and August 24, 1994, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. This amended license is subject to the following conditions for the protection of the environment:
 - A. Upon completion of the licensees' "smoke tests," the licensees shall release all procedures, data, and analysis bearing on the calculation of X/Q to the Regulatory Staff as soon as possible but no later than initial fuel loading for Peach Bottom Unit 3. Using the "smoke test" results to re-calculate X/Q and using the other assumptions previously used by the Regulatory Staff (or realistic refinements thereof), the Regulatory Staff shall re-calculate the potential radioiodine thyroid dose to a child via the milk pathway at the points of maximum concentration at or beyond the site boundary where dairy cows are present or could be pastured. If this calculated dose based on the combined operation of Peach Bottom Units 2 and 3 exceeds 15 mrem/year, then the licensees shall install for operation no later than the next refueling cycle for each unit the necessary equipment to reduce the projected dose to such levels.
 - B. If the actual milk sample measurements taken at the nearby farms in accordance with the Technical Specifications, predict a dose to a child's thyroid, based on actual combined operation of Unit 2 and 3, that exceeds 15 mrem/year, licensees shall install for operation no later than the next refueling cycle for each unit the necessary equipment to reduce the projected dose to such levels.
 - C. To the extent matters related to thermal discharges are treated therein, operation of Peach Bottom Atomic Power Station Unit No. 2 will be governed by NPDES Permit No. PA 0009733, as now in effect and as hereafter amended. Questions pertaining to conformance thereto shall be referred to and shall be determined by the NPDES Permit issuing or enforcement authority, as appropriate.
 - D. In the event of any modification of the NPDES Permit related to thermal discharges or the establishment (or amendment) of alternative effluent limitations established pursuant to Section 316 of the Federal Water Pollution Control Act, the licensees shall inform the NRC and analyze any associated changes in or to the Station, its components, its operation or in the discharge of effluents therefrom. If such change would entail any modification to this license, or any Technical Specifications which are part of this license, or present an unreviewed safety question or involve an environmental impact different than analyzed in the Final Environmental Statement, the licensees shall file with the NRC, as applicable, an appropriate analysis of any such change on facility safety, and/or an analysis of any such change on the environmental impacts and on the overall cost-benefit balance for facility operation set forth in the Final Environmental Statement and a request for an amendment to the operating license, if required

by the Commission's regulations. As used in this Condition 3.(d), Final Environmental Statement means the NRC Staff Final Environmental Statement related to Operation of Peach Bottom Atomic Power Station Units Nos. 2 and 3 dated April 1973, as modified by (1) the Initial Decision of the Atomic Safety and Licensing Board dated September 14, 1973, (2) the Supplemental Initial Decision of the Atomic Safety and Licensing Board dated June 14, 1974, (3) the Decision of the Atomic Safety and Licensing Appeal Board dated July 5, 1974, (4) the Memorandum and Order of the Commission dated August 8, 1974, (5) any further modification resulting from further review by the Appeal Board and by the Commission, if any, and (6) any Environmental Impact Appraisal which has been or may be issued by the NRC since the FES was published in April 1973.

4. This license is effective as of the date of issuance and shall expire at midnight on August 8, 2013.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed by R. C. DeYoung, for

A. Giambusso, Deputy Director
for Reactor Projects
Directorate of Licensing

Attachments:

Appendices A and B -
Technical Specifications

Date of Issuance: October 25, 1973

PBAPS

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.3 REACTIVITY CONTROLApplicability:

Applies to the operational status of the control rod system.

Objective:

To assure the ability of the control rod system to control reactivity.

Specification:A. Reactivity Limitations1. Reactivity margin - core loading

A sufficient number of control rods shall be operable so that the core could be made subcritical in the most reactive condition during the operating cycle with the strongest control rod fully withdrawn and all other operable control rods fully inserted.

2. Reactivity margin - inoperable control rods

- a. Control rods which cannot be moved with control rod drive pressure shall be considered inoperable.

If a partially or fully withdrawn control rod drive cannot be moved with drive or scram pressure the reactor shall be brought to a shutdown condition within 48 hours unless

4.3 REACTIVITY CONTROLApplicability:

Applies to the surveillance requirements of the control rod system.

Objective:

To verify the ability of the control rod system to control reactivity.

Specification:A. Reactivity Limitations1. Reactivity margin - core loading

Sufficient control rods shall be withdrawn following a refueling outage when core alterations were performed to demonstrate with a margin of 0.38%Δk/k that the core can be made subcritical at any time in the subsequent fuel cycle with the analytically determined strongest operable control rod fully withdrawn and all other operable rods fully inserted.

2. Reactivity margin - inoperable control rods

- a. Each partially or fully withdrawn operable control rod shall be exercised one notch at least once each week when operating above the RWM low power setpoint. Each partially or fully withdrawn operable control rod shall be exercised at least one notch within 24 hours when operating above the RWM low power setpoint if there are three

Unit 2

PBAPS

LIMITING CONDITIONS FOR OPERATION

3.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd)

B. Normal System Requirements

At all times when the Standby Liquid Control System is required to be operable, the following conditions shall be met:

1. At least 162.7 lbm Boron-10 must be stored in the Standby Liquid Control Solution Tank and be available for injection.
2. The sodium pentaborate solution concentration must be equal to or less than 9.82% weight.

SURVEILLANCE REQUIREMENTS

4.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd)

2. Manually initiate one of the Standby Liquid Control System Pumps and pump demineralized water into the reactor vessel from the test tank.

This test checks explosion of the charge associated with the tested loop, proper operation of the explosive valves, and pump operability. The replacement charge for the explosive valve shall be from the same manufactured batch as the one fired or from another batch which has been certified by having one of the batch successfully fired.

3. Both systems, including both explosive valves, shall be tested in the course of two operating cycles.

B. Normal System Requirements

1. Boron-10 Quantity: At least once per month calculate and record the quantity of Boron-10 stored in the Standby Liquid Control Solution Tank.
2. Concentration: At least once per month check and record. Also, check concentration anytime water or boron is added to the solution.

Unit 2

PBAPS

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd.)

4.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd.)

3. The Standby Liquid Control System conditions must satisfy the following equation:

$$\left(\frac{C}{13\% \text{ wt.}}\right) \left(\frac{Q}{86 \text{ gpm}}\right) \left(\frac{E}{19.8\% \text{ atom}}\right) \geq 1$$

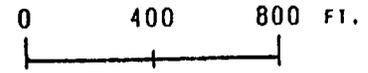
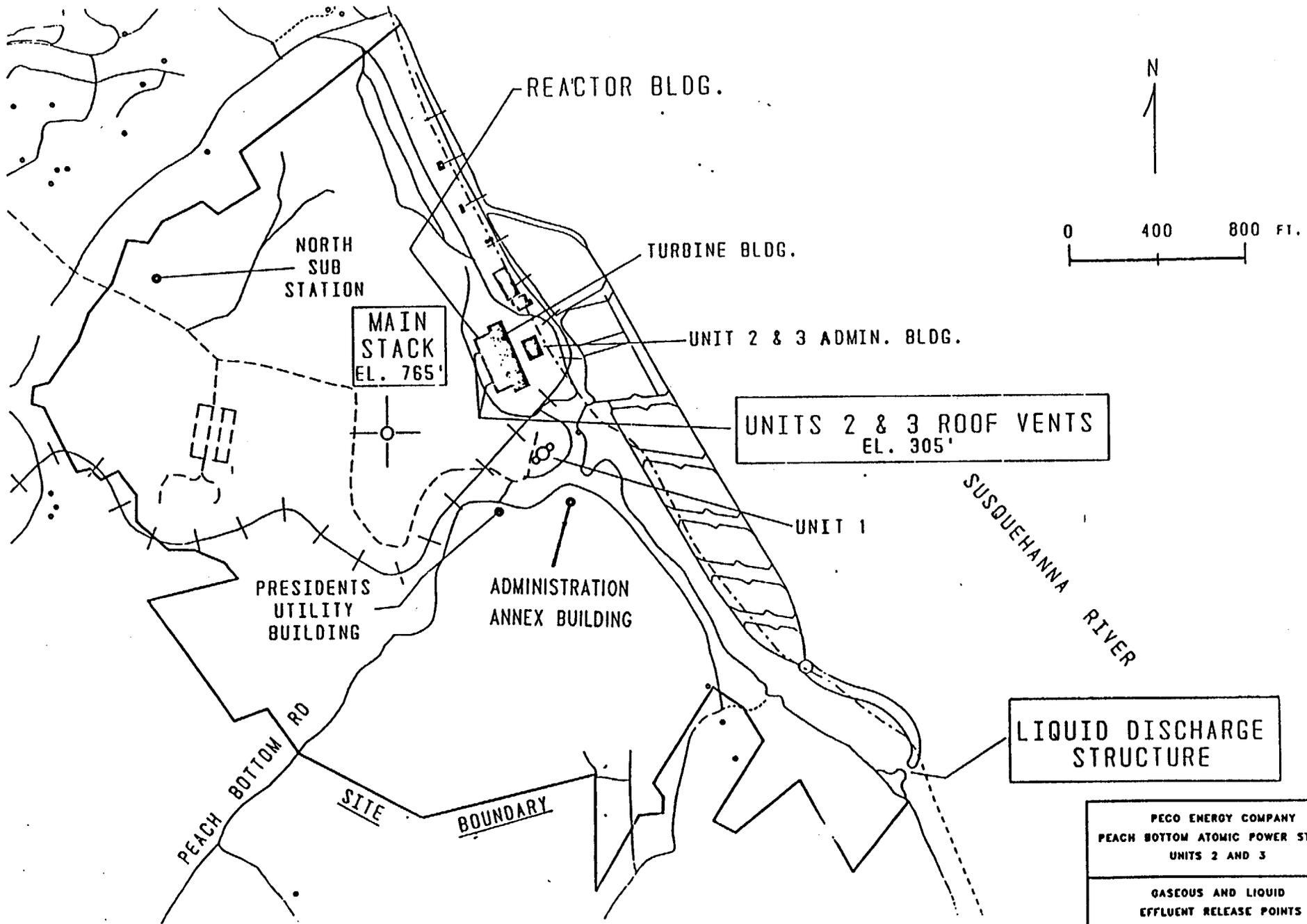
where,

C = Sodium Pentaborate Solution Concentration (% weight)

Q = Pump Flow Rate (gpm) against a system head of 1255 psig.

E = Boron-10 Enrichment (% atom Boron-10)

3. Pump Flow Rate: At least once per 92 days each pump loop shall be functionally tested by pumping boron solution to the test tank. At least once per quarter check and record pump flow rate against a system head of 1255 psig.
4. Enrichment: Following each addition of boron to the solution tank, calculate enrichment within 8 hours. Verify results by analysis within 30 days.
5. Solution Volume: At least once per day check and record.



Unit 2

<p>PECO ENERGY COMPANY PEACH BOTTOM ATOMIC POWER STATION UNITS 2 AND 3</p>
<p>GASEOUS AND LIQUID EFFLUENT RELEASE POINTS FIGURE 3.B.1</p>

PBAPS

6.5.2 Nuclear Review Board Function

6.5.2.1 The Nuclear Review Board (NRB) shall function to provide independent review and audit of designated activities in the area of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

The members of the NRB will be competent in the area of quality assurance practice and cognizant of the Quality Assurance requirements of 10 CFR 50, Appendix B. Additionally, they will be cognizant of the corporate Quality Assurance Program and will have the corporate Quality Assurance organization available to them.

Organization

6.5.2.2 The Chairman, members and alternate members of the NRB shall be appointed in writing by the Executive Vice President - Nuclear, and shall have an academic degree in an engineering or physical science field and in addition, shall have a minimum of five years technical experience, of which a minimum of three years shall be in one or more areas given in 6.5.2.1.

The NRB shall be composed of at least five members who are PECO Energy Company (PECO) employees and at least two outside members who are not PECO employees.

6.9.1 Routine Reports (cont'd)

c. Annual Safety/Relief Valve Report

Describe all challenges to the primary coolant system safety and relief valves. Challenges are defined as the automatic opening of the primary coolant safety or relief valves in response to high reactor pressure.

d. Monthly Operating Report

Routine reports of operating statistics and shutdown experience and a narrative summary of the operating experience shall be submitted on a monthly basis. Each report shall be submitted no later than the 15th of the month following the calendar month covered by the report.

e. Core Operating Limits Report

- (1) Core operating limits shall be established and shall be documented in the CORE OPERATING LIMITS REPORT prior to each Operating Cycle, or prior to any remaining portion of an Operating Cycle, for the following:
 - a. The APLHGR for Specification 3.5.I,
 - b. The MCPR for Specification 3.5.K,
 - c. The core flow and power adjustment factors for Specification 3.5.K and 3.5.I,
 - d. The LHGR for Specification 3.5.J,
 - e. The upscale power biased Rod Block Monitor setpoints and corresponding power levels.
- (2) The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents as amended and approved:
 - a. NEDE-24011-P-A, "General Electric Standard Application for Reactor Fuel" (latest approved version)
 - b. "Maximum Extended Load Line Limit and ARTS Improvement Program Analyses for Peach Bottom Atomic Power Station Units 2 and 3," NEDC-32162P, Revision 1, February, 1993
 - c. PECO Energy Company Methodologies as described in:
 - (1) PECO-FMS-0001-A, "Steady-State Thermal Hydraulic Analysis of Peach Bottom Units 2 and 3 using the FIBWR Computer Code"

PBAPS

6.0 Environmental Surveillance and Special Study Programs

Studies described in this section will be terminated upon agreement by PECO Energy Company and the Nuclear Regulatory Commission that the intended purpose of the study has been satisfied.

6.1 (Deleted)

6.2 (Deleted)

6.3 (Deleted)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PHILADELPHIA ELECTRIC COMPANY

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 201
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated April 15, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, pages 1, 2, 3, 4, 4a, and 5 of the Facility Operating License No. DPR-56 are hereby amended by changing Philadelphia Electric Company to PECO Energy Company.
3. Further, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 20¹, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

4. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



J. F. Stolz
FOR

John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachments:

1. Pages 1, 2, 3, 4, and 5
of License No. DPR-56
2. Changes to the Technical
Specifications

Date of Issuance: November 29, 1994

*Pages 1, 2, 3, 4, and 5 are attached, for convenience, for the composite license to reflect this change.

ATTACHMENT TO LICENSE AMENDMENT NO. 201

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Facility Operating License (FOL), the Appendix A Technical Specifications, and Appendix B Environmental Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

	<u>Remove</u>	<u>Insert</u>
FOL	1	1
	2	2
	3	3
	4	4
	4a	--
	5	5
Appendix A	99	99
	101	101
	116	116
	117	117
	216e	216e
	249	249
	256	256
Appendix B	14	14
	41-47	--



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PECO ENERGY COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY
DOCKET NO. 50-278
PEACH BOTTOM ATOMIC POWER STATION, UNIT 3
FACILITY OPERATING LICENSE

License No. DPR-56

1. The Atomic Energy Commission (the Commission) having found that:
 - A. The application for license filed by PECO Energy Company, formerly Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company (the licensees) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I and all required notifications to other agencies or bodies have been duly made;
 - B. Construction of the Peach Bottom Atomic Power Station, Unit 3 (the facility) has been substantially completed in conformity with Construction Permit No. CPPR-38 and the application, as amended, the provisions of the Act and the rules and regulations of the Commission;
 - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - D. There is reasonable assurance: (1) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (2) that such activities will be conducted in compliance with the rules and regulations of the Commission;
 - E. PECO Energy Company is technically qualified and the licensees are financially qualified to engage in the activities authorized by this amended operating license in accordance with the rules and regulations of the Commission;
 - F. The licensees have satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission's regulations;
 - G. The issuance of this operating license will not be inimical to the common defense and security or to the health and safety of the public;

- H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental costs and considering available alternatives, the issuance of Facility Operating License No. DPR-56 is in accordance with 10 CFR Part 50, Appendix D, of the Commission's regulations and all applicable requirements of said Appendix D have been satisfied; and
- I. The receipt, possession, and use of by-product and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40, and 70, including 10 CFR Section 30.33, 40.32, and 70.23 and 70.31.
2. Facility Operating License No. DPR-56 is hereby issued to the PECO Energy Company (PECO), formerly the Philadelphia Electric Company, Public Service Electric and Gas Company (PSE&G), Delmarva Power and Light Company (DP&LC), and Atlantic City Electric Company (ACEC), is hereby amended in its entirety to read as follows:
- A. This license applies to the Peach Bottom Atomic Power Station, Unit 3, a direct cycle, forced circulation, boiling water nuclear reactor and associated equipment (the facility), owned by the licensees and operated by PECO Energy Company. The facility is located in Peach Bottom, York County, Pennsylvania and is described in the "Final Safety Analysis Report" as supplemented and amended (Amendments 1 through 31) and the Environmental Report as supplemented and amended (Supplements 1 through 6).
- B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses:
- (1) PECO Energy Company, pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess, use, and operate the facility and PSE&G, DP&LC, and ACEC to possess the facility at the designated location in Peach Bottom, York County, Pennsylvania in accordance with the procedures and limitations set forth in this license;
 - (2) PECO Energy Company, pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended, as of December 15, 1975;
 - (3) PECO Energy Company, pursuant to the Act and 10 CFR Parts 30, 40, and 70 to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) PECO Energy Company, pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form for sample analysis or instrument calibration or when associated with radioactive apparatus or components;

(5) PECO Energy Company, pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not to separate, such byproduct and special nuclear material as may be produced by operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

(1) Maximum Power Level

PECO Energy Company is authorized to operate the Peach Bottom Atomic Power Station, Unit 3, at steady state reactor core power levels not in excess of 3293 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. are hereby incorporated in the license. PECO Energy Company shall operate the facility in accordance with the Technical Specifications.

(3) Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Peach Bottom Atomic Power Station, Units 2 and 3, Physical Security Plan," with revisions submitted through December 16, 1987; "Peach Bottom Atomic Power Station, Units 2 and 3 Plant Security Personnel Training and Qualification Plan," with revisions submitted through July 9, 1986; and "Peach Bottom Atomic Power Station, Units 2 and 3 Safeguards Contingency Plan," with revisions submitted through March 10, 1981. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

(4) The licensee shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility, and as approved in the NRC SER dated May 23, 1979 and Supplements dated August 14, September 15, October 10 and November 24, 1980, and in the NRC SERs dated September 16, 1993 and August 24, 1994, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

- (5) Operation beyond the end-of-cycle (all rods out condition) thermal power is limited to seventy (70) percent minimum. Increasing core power level via reduced feedwater heating, once operation in the coastdown mode has begun, is not permitted unless the licensee has performed an analysis of this operating condition that confirms that this condition is bounded by the analysis for the particular cycle of operation.

3. This license is subject to the following conditions for the protection of the environment:

- A. If the actual milk sample measurements taken at the nearby farms in accordance with the Technical Specifications, predict a dose to a child's thyroid, based on actual combined operation of Unit 2 and 3, that exceeds 15 mrem/year, licensees shall install for operation no later than the next refueling cycle for each unit the necessary equipment to reduce the projected dose to such levels.
- B. To the extent matters related to thermal discharges are treated therein, operation of Peach Bottom Atomic Power Station Unit 2 will be governed by NPDES Permit No. PA 0009733, as now in effect and as hereafter amended. Questions pertaining to conformance thereto shall be referred to and shall be determined by the NPDES Permit issuing or enforcement authority, as appropriate.
- C. In the event of any modification of the NPDES Permit related to thermal discharges or the establishment (or amendment) of alternative effluent limitations established pursuant to Section 316 of the Federal Water Pollution Control Act, the licensees shall inform the NRC and analyze any associated changes in or to the Station, its components, its operation or in the discharge of effluents therefrom. If such change would entail any modification to this license, or any technical specifications which are part of this license, or present an unreviewed safety question or involve an environmental impact different than analyzed in the Final Environmental Statement, the licensees shall file with the NRC, as applicable, an appropriate analysis of any such change on facility safety, and/or an analysis of any such change on the environmental impacts and on the overall cost-benefit balance for facility operation set forth in the Final Environmental Statement and a request for an amendment to the operating license, if required by the Commission's regulations. As used in this Condition 3.(d), Final Environmental Statement means the NRC Staff Final Environmental Statement related to Operation of Peach Bottom Atomic Power Station Units Nos. 2 and 3 dated April 1973, as modified by (1) the Initial Decision of the Atomic Safety and Licensing Board dated September 14, 1973, (2) the Supplemental Initial Decision of the Atomic Safety and Licensing Board dated June 14, 1974, (3) the Decision of the Atomic Safety and Licensing Appeal Board dated July 5, 1974, (4) the Memorandum and Order of the Commission dated August 8, 1974, (5) any further modification resulting from further review by the Appeal Board and by

the Commission, if any, and (6) any Environmental Impact Appraisal which has been or may be issued by the NRC since the FES was published in April 1973.

4. This license is effective as the date of issuance and shall expire at midnight on July 2, 2014.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed by Roger Boyd, for

A. Giambusso, Deputy Director
for Reactor Projects
Directorate of Licensing

Attachments:
Amended pages to Appendices A and B
DPR-44 & DPR-56 Technical
Specifications

Date of Issuance: July 2, 1974

PBAPS

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.3 REACTIVITY CONTROLApplicability:

Applies to the operational status of the control rod system.

Objective:

To assure the ability of the control rod system to control reactivity.

Specification:A. Reactivity Limitations1. Reactivity margin - core loading

A sufficient number of control rods shall be operable so that the core could be made subcritical in the most reactive condition during the operating cycle with the strongest control rod fully withdrawn and all other operable control rods fully inserted.

2. Reactivity margin - inoperable control rods

- a. Control rods which cannot be moved with control rod drive pressure shall be considered inoperable.

If a partially or fully withdrawn control rod drive cannot be moved with drive or scram pressure the reactor shall be brought to a shutdown condition within 48 hours unless

4.3 REACTIVITY CONTROLApplicability:

Applies to the surveillance requirements of the control rod system.

Objective:

To verify the ability of the control rod system to control reactivity.

Specification:A. Reactivity Limitations1. Reactivity margin - core loading

Sufficient control rods shall be withdrawn following a refueling outage when core alterations were performed to demonstrate with a margin of $0.38\% \Delta k/k$ that the core can be made subcritical at any time in the subsequent fuel cycle with the analytically determined strongest operable control rod fully withdrawn and all other operable rods fully inserted.

2. Reactivity margin - inoperable control rods

- a. Each partially or fully withdrawn operable control rod shall be exercised one notch at least once each week when operating above the RWM low power setpoint. Each partially or fully withdrawn operable control rod shall be exercised at least one notch within 24 hours when operating above the RWM low power setpoint if there are three

PBAPS

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.3.A Reactivity Limitations
(Cont'd)4.3.A Reactivity Limitations
(Cont'd)

- f. Inoperable control rods shall be positioned such that specification 3.3.A.1 is met. In addition, during reactor power operation, no more than one control rod in any 5 x 5 array may be inoperable (at least 4 operable control rods must separate any 2 inoperable ones). If this Specification cannot be met the reactor shall not be started, or if at power, the reactor shall be brought to a cold shutdown condition within 24 hours.

B. Control Rods

1. Each control rod shall be coupled to its drive or completely inserted and the control rod directional control valves disarmed electrically except as in 3.3.B.1.a. This requirement does not apply in the refuel condition when the reactor is vented. Two control rod drives may be removed as long as Specification 3.3.A.1 is met.

- a. For control rod 54-35, for the remainder of cycle 10 (to be completed before 10/30/95).

If coupling cannot be accomplished, the uncoupled control rod may be withdrawn when $\geq 10\%$ of rated thermal power only if all the following conditions are satisfied:

- 1) no other uncoupled control rod is withdrawn;
- 2) the uncoupled control rod may not be withdrawn past notch position 46.

B. Control Rods

1. The coupling integrity shall be verified for each withdrawn control rod as follows:
- a. When a rod is withdrawn the first time after each refueling outage or after maintenance, observe discernible response of the nuclear instrumentation and rod position indication for the "full-in" and "full-out" position. However, for initial rods when response is not discernible, subsequent exercising of these rods after the reactor is above the Rod Worth Minimizer low power setpoint shall be performed to verify instrumentation response.
 - b. When the rod is fully withdrawn the first time after each refueling outage or after maintenance observe that the drive does not go to the overtravel position.

LIMITING CONDITIONS FOR OPERATION3.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd)B. Normal System Requirements

At all times when the Standby Liquid Control System is required to be operable, the following conditions shall be met:

1. At least 162.7 lbm Boron-10 must be stored in the Standby Liquid Control Solution Tank and be available for injection.
2. The sodium pentaborate solution concentration must be equal to or less than 9.82% weight.

SURVEILLANCE REQUIREMENTS4.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd)

2. Manually initiate one of the Standby Liquid Control System Pumps and pump demineralized water into the reactor vessel from the test tank.

This test checks explosion of the charge associated with the tested loop, proper operation of the explosive valves, and pump operability. The replacement charge for the explosive valve shall be from the same manufactured batch as the one fired or from another batch which has been certified by having one of the batch successfully fired.

3. Both systems, including both explosive valves, shall be tested in the course of two operating cycles.

B. Normal System Requirements

1. Boron-10 Quantity: At least once per month calculate and record the quantity of Boron-10 stored in the Standby Liquid Control Solution Tank.
2. Concentration: At least once per month check and record. Also, check concentration anytime water or boron is added to the solution.

Unit 3

PBAPS

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd)

4.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd)

3. The Standby Liquid Control System conditions must satisfy the following equation:

$$\left(\frac{C}{13\% \text{ wt.}}\right) \left(\frac{Q}{86 \text{ gpm}}\right) \left(\frac{E}{19.8\% \text{ atom}}\right) \geq 1$$

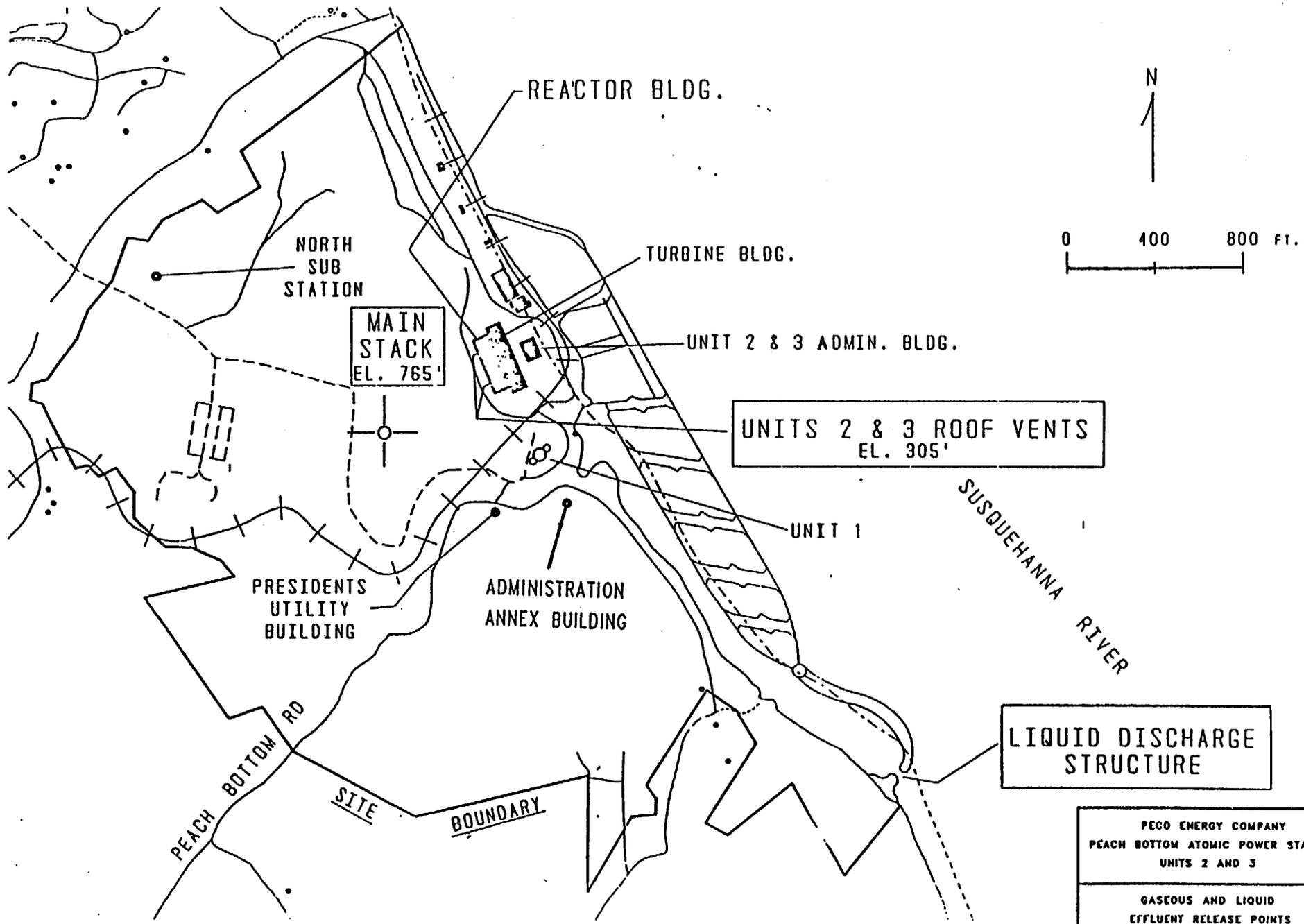
where,

C = Sodium Pentaborate Solution Concentration (% weight)

Q = Pump Flow Rate (gpm) against a system head of 1225 psig.

E = Boron-10 Enrichment (% atom Boron-10)

3. Pump Flow Rate: At least once per 92 days each pump loop shall be functionally tested by pumping boron solution to the test tank. At least once per quarter check and record pump flow rate against a system head of 1225 psig.
4. Enrichment: Following each addition of boron to the solution tank, calculate enrichment within 8 hours. Verify results by analysis within 30 days.
5. Solution Volume: At least once per day check and record.



PEGO ENERGY COMPANY
 PEACH BOTTOM ATOMIC POWER STATION
 UNITS 2 AND 3

GASEOUS AND LIQUID
 EFFLUENT RELEASE POINTS
 FIGURE 3.8.1

PBAPS

6.5.2 Nuclear Review Board Function

- 6.5.2.1 The Nuclear Review Board (NRB) shall function to provide independent review and audit of designated activities in the area of:
- a. nuclear power plant operations
 - b. nuclear engineering
 - c. chemistry and radiochemistry
 - d. metallurgy
 - e. instrumentation and control
 - f. radiological safety
 - g. mechanical and electrical engineering
 - h. quality assurance practices

The members of the NRB will be competent in the area of quality assurance practice and cognizant of the Quality Assurance requirements of 10 CFR 50, Appendix B. Additionally, they will be cognizant of the corporate Quality Assurance Program and will have the corporate Quality Assurance organization available to them.

Organization

- 6.5.2.2 The Chairman, members and alternate members of the NRB shall be appointed in writing by the Executive Vice President - Nuclear, and shall have an academic degree in an engineering or physical science field and in addition, shall have a minimum of five years technical experience, of which a minimum of three years shall be in one or more areas given in 6.5.2.1.

The NRB shall be composed of at least five members who are PECO Energy Company (PECO) employees and at least two outside members who are not PECO employees.

6.9.1 Routine Reports (cont'd)c. Annual Safety/Relief Valve Report

Describe all challenges to the primary coolant system safety and relief valves. Challenges are defined as the automatic opening of the primary coolant safety or relief valves in response to high reactor pressure.

d. Monthly Operating Report

Routine reports of operating statistics and shutdown experience and a narrative summary of the operating experience shall be submitted on a monthly basis. Each report shall be submitted no later than the 15th of the month following the calendar month covered by the report.

e. Core Operating Limits Report

(1) Core operating limits shall be established and shall be documented in the CORE OPERATING LIMITS REPORT prior to each Operating Cycle, or prior to any remaining portion of an Operating Cycle, for the following:

- a. The APLHGR for Specification 3.5.I,
- b. The MCPR for Specification 3.5.K,
- c. The core flow and power adjustment factors for Specification 3.5.K and 3.5.I,
- d. The LHGR for Specification 3.5.J,
- e. The upscale power biased Rod Block Monitor setpoints and corresponding power levels.

(2) The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents as amended and approved:

- a. NEDE-24011-P-A, "General Electric Standard Application for Reactor Fuel" (latest approved version)
- b. "Maximum Extended Load Line Limit and ARTS Improvement Program Analyses for Peach Bottom Atomic Power Station Units 2 and 3," NEDC-32162P, Revision 1, February, 1993
- c. PECO Energy Company Methodologies as described in:
 - (1) PECO-FMS-0001-A, "Steady-State Thermal Hydraulic Analysis of Peach Bottom Units 2 and 3 using the FIBWR Computer Code"

PBAPS

6.0 Environmental Surveillance and Special Study Programs

Studies described in this section will be terminated upon agreement by PECO Energy Company and the Nuclear Regulatory Commission that the intended purpose of the study has been satisfied.

6.1 (Deleted)

6.2 (Deleted)

6.3 (Deleted)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 199 AND 201 TO FACILITY OPERATING

LICENSE NOS. DPR-44 and DPR-56

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3

DOCKET NOS. 50-277 AND 50-278

1.0 INTRODUCTION

By letter dated April 15, 1994, the Philadelphia Electric Company (the licensee) submitted a request for changes to the Peach Bottom Atomic Power Station (PBAPS), Unit Nos. 2 and 3, Technical Specifications (TS). The requested changes would 1) correct a typographical error in the Unit 3 TS, 2) reflect the name change of Philadelphia Electric Company to PECO Energy Company, and 3) implement line-item TS improvements recommended by Generic Letter (GL) 93-05, "Line-Item Technical Specifications Improvements to Reduce Surveillance Requirements for Testing During Power Operation."

2.0 EVALUATION

The licensee's request to revise the Unit 3 TS Section 3.3.A.2.f corrects an error which occurred when Amendment 187 was issued on November 29, 1993. Specifically, due to a typographical error in its submittal, the licensee mistakenly requested and was granted a change to the TS (i.e., the word "inoperable" was inadvertently substituted for "operable" when the licensee retyped and submitted Unit 3 TS page 101 for an emergency amendment request). The licensee's proposal to correct this error restores the TS to the original wording and is therefore considered an acceptable editorial change.

The licensee also requested to change the name of the owner and licensee of PBAPS from Philadelphia Electric Company to PECO Energy Company. In a December 21, 1993 letter, the licensee informed the NRC that it was officially changing its name to the PECO Energy Company effective January 1, 1994. Therefore, the licensee's name change request is an acceptable editorial change which is necessary to ensure that the name of the PBAPS licensee corresponds to the company's new name.

In addition to the editorial changes discussed above, the licensee also requested three changes recommended in the staff's GL 93-05.

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P PDR

The licensee's first GL 93-05 change request would revise the frequency in TS Section 4.3.A.2.a to exercise each partially or fully withdrawn operable control rod from every 24 hours to within 24 hours when operating above the rod worth minimizer low power setpoint (this TS only applies if there are three or more inoperable control rods or if there is one fully or partially withdrawn rod which cannot be moved and for which control rod drive mechanism damage has not been ruled out). Specifically, the licensee proposed to revise the wording in TS 4.3.A.2.a from "...at least every 24 hours..." to "...within 24 hours...." This change request corresponds to GL 93-05 Item 4.2, which recommends that the applicable TS wording be revised from "At least once per 24 hours..." to "Within 24 hours...." Therefore, the staff finds that the licensee's proposal is equivalent to the GL 93-05 recommendation and is acceptable.

The licensee's second GL 93-05 change request would revise TS 4.4.A.2 to allow for the replacement charge on the explosive valve for the standby liquid control system to be from either the same manufactured batch as the one fired or another batch which has been certified by having one of the batch successfully fired. Specifically, the licensee proposed to revise the statement "The replacement charges to be installed will be selected from the same manufactured batch as the tested charge" to "The replacement charge for the explosive valve shall be from the same manufactured batch as the one fired or from another batch which has been certified by having one of the batch successfully fired." The staff's GL 93-05, Item 4.3 wording restates the following General Electric Boiling Water Reactors/4 standard TS (BWR/4 STS) wording: "The replacement charge for the explosive valve shall be from the same manufactured batch as the one fired or from another batch which has been certified by having one of the batch successfully fired." The licensee is proposing to utilize this sentence from the BWR/4 STS (and restated in GL 93-05). The staff reviewed the licensee's proposal against the recommended wording in the BWR/4 STS. The staff finds that the licensee's proposal for revising TS 4.4.A.2 is identical to the applicable BWR/4 STS (and GL 93-05) wording and is therefore acceptable.

The licensee's third GL 93-05 change request would revise TS Section 4.4.B.3 to functionally test each standby liquid control system pump loop from monthly to at least once per 92 days. Specifically, the licensee proposed to revise "At least once per month..." to "At least once per 92 days...." The staff's GL 93-05, Item 4.3 recommends that the Standby Liquid Control System pump test be required quarterly (i.e., every 92 days), in accordance with the American Society of Mechanical Engineers Code. The staff finds that the licensee's proposal is equivalent to the GL 93-05 recommendation and is therefore acceptable.

The three changes that the licensee is proposing, in accordance with the recommendations of GL 93-05, extend or change surveillance requirements. In order to demonstrate that the recommendations are compatible with a particular plant's operating experience, GL 93-05 requests licensees to include a statement in the license amendment request that "all proposed TS changes are compatible with plant operating experience and are consistent with this

guidance." To satisfy this request, the licensee included the following statement in its submittal: "The proposed changes to Surveillance Requirements for TS Sections 4.3.A.2.a, 4.4.A.2, and 4.4.B.3 are compatible with PBAPS operating experience and is [sic] consistent with the intent of the corresponding recommendations in NUREG-1366, 'Improvements to Technical Specifications Surveillance Requirements,' guidance in Generic Letter 93-05, and the format of the PBAPS TS." The staff finds that the licensee's statement is equivalent to the GL 93-05 recommendation and is therefore acceptable.

Therefore, the licensee has acceptably justified its editorial changes and has conformed to the guidance of GL 93-05 and the BWR/4 STS for the three non-editorial changes. Consequently, the licensee's proposal is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and administrative procedures or requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (59 FR 27064). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Dembek

Date: November 29, 1994