

August 24, 1994

Docket Nos. 50-277
and 50-278

Mr. George A. Hunger, Jr.
Director-Licensing, MC 52A-5
PECO Energy Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box No. 195
Wayne, Pennsylvania 19087-0195

Dear Mr. Hunger:

SUBJECT: REMOVAL OF FIRE PROTECTION REQUIREMENTS FROM TECHNICAL SPECIFICATIONS, PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3 (TAC NOS. M89157 AND M89158)

The Commission has issued the enclosed Amendments Nos. 194 and 198 to Facility Operating License Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Unit Nos. 2 and 3. These amendments consist of changes to the Technical Specifications (TS) in response to your application dated March 28, 1994, as supplemented on June 27, 1994, and July 8, 1994.

These amendments would relocate the TS Fire Protection requirements to the Updated Final Safety Analysis Report in accordance with Generic Letter (GL) 86-10, "Implementation of Fire Protection Requirements," and GL 88-12, "Removal of Fire Protection Requirements from Technical Specifications." Please inform the staff, in writing, when you have implemented these amendments.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

/s/
Stephen Dembek, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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PDR ADOCK 05000277
P PDR

Enclosures:

1. Amendment No. 194 to DPR-44
2. Amendment No. 198 to DPR-56
3. Safety Evaluation

cc w/enclosures:

See next page

DISTRIBUTION:

Docket File	MO'Brien(2)	CGrimes, 0-11E21
NRC & Local PDRs	SDembek	CMcCracken, 0-8D1
PDI-2 Reading	OGC	ACRS(10)
SVarga	DHagan, 3206	OPA
JCalvo	GHill(4), P1-22	OC/LFDCB
MThadani	EWenzinger, RGN-I	CAnderson, RGN-I

*Previously Concurred

NRC FILE CENTER COPY

OFC	: PDI-2/LA	: PDI-2/PM	: SPLB/BC*	: OGC*	: PDI-2/D	:
NAME	: MO'Brien	: SDembek	: tc:CMcCracken:		: MThadani	:
DATE	: 8/13/94	: 8/23/94	: 08/16/94	: 08/18/94	: 8/23/94	:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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PECO Energy Company
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The Commission has issued the enclosed Amendments Nos. 194 and 198 to Facility Operating License Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Unit Nos. 2 and 3. These amendments consist of changes to the Technical Specifications (TS) in response to your application dated March 28, 1994, as supplemented on June 27, 1994, and July 8, 1994.

These amendments relocate the TS Fire Protection requirements to the Updated Final Safety Analysis Report in accordance with Generic Letter (GL) 86-10, "Implementation of Fire Protection Requirements," and GL 88-12, "Removal of Fire Protection Requirements from Technical Specifications." Please inform the staff, in writing, when you have implemented these amendments.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "Stephen Dembek".

Stephen Dembek, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 194 to DPR-44
2. Amendment No. 198 to DPR-56
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. George A. Hunger, Jr.
PECO Energy Company

Peach Bottom Atomic Power Station,
Units 2 and 3

cc: --

J. W. Durham, Sr., Esquire
Sr. V.P. & General Counsel
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Division of Nuclear Safety
Pennsylvania Department of
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Harrisburg, Pennsylvania 17105-8469

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Delta, Pennsylvania 17314

Board of Supervisors
Peach Bottom Township
R. D. #1
Delta, Pennsylvania 17314

PECO Energy Company
ATTN: Regulatory Engineer, A1-2S
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Delta, Pennsylvania 17314

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Baltimore, MD 21202-6806

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U.S. Nuclear Regulatory Commission
Peach Bottom Atomic Power Station
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Annapolis, Maryland 21401

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
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King of Prussia, Pennsylvania 19406

Mr. John Doering, Chairman
Nuclear Review Board
PECO Energy Company
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Mail Code 63C-5
Wayne, Pennsylvania 19087

Mr. Roland Fletcher
Department of Environment
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Baltimore, Maryland 21201

Carl D. Schaefer
External Operations - Nuclear
Delmarva Power & Light Company
P.O. Box 231
Wilmington, DE 19899



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PHILADELPHIA ELECTRIC COMPANY

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 194
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated March 28, 1994, as supplemented on June 27, 1994, and July 8, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. DPR-44 paragraph 2.C.(5) is hereby amended to read as follows:
 - 2.C.(5) The licensee shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility, and as approved in the NRC SER dated May 23, 1979 and Supplements dated

August 14, September 15, October 10 and November 24, 1980, and in the NRC SER dated September 16, 1993, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. Further, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 194, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Mohan C. Thadani, Acting Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachments:

1. Page 4a of License*
2. Changes to the Technical Specifications

Date of Issuance: August 24, 1994

*Page 4a is attached, for convenience, for the composite license to reflect this change.

- 2.C.(5) The licensee shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility, and as approved in the NRC SER dated May 23, 1979 and Supplements dated August 14, September 15, October 10 and November 24, 1980, and in the NRC SERs dated September 16, 1993 and August 24, 1994 subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. This amended license is subject to the following conditions for the protection of the environment:
- (a) Upon completion of the licensees' "smoke tests," the licensees' shall release all procedures, data, and analysis bearing on the calculation of χ/Q to the Regulatory Staff as soon as possible but no later than initial fuel loading for Peach Bottom Unit 3. Using the "smoke test" results to re-calculate χ/Q and using the other assumptions previously used by the Regulatory Staff (or realistic refinements thereof), the Regulatory Staff shall re-calculate the potential radioiodine thyroid dose to a child via the milk pathway at the points of maximum concentration at or beyond the site boundary where dairy cows are present or could be pastured. If this calculated dose based on the combined operation of Peach Bottom Units 2 and 3 exceeds 15 mrem/year, then the licensees' shall install for operation no later than the next refueling cycle for each unit the necessary equipment to reduce the projected dose to such levels.
- (b) If the actual milk sample measurements taken at the nearby farms in accordance with the technical specifications, predict a dose to a child's thyroid, based on actual combined operation of

ATTACHMENT TO LICENSE AMENDMENT NO. 194

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

<u>Remove</u>	<u>Insert</u>
240c	240c
240d through 240s	240d through 240s
244	244
246	246
248	248
252	252
253	253

PBAPS

The Fire Protection Limiting Conditions for Operation and Surveillance Requirements have been relocated to the Fire Protection Program, which is part of the UFSAR. These requirements are still mandated by the Facility Operating License.

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PBAPS

Continued

6.2.2 Facility Staff

The facility organization shall be subject to the following:

- a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2.1, except that the shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
- b. At least one licensed operator shall be in the control room and assigned to each reactor that contains fuel.
- c. At least two licensed operators, excluding the operator on the second unit, shall be present in the control room during reactor startup, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be onsite when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibility during this operation.
- f. Deleted.
- g. In addition to the requirements of 6.2.2.a, the Senior Manager-Operations shall hold a Senior Reactor Operator license.

6.3 Facility Staff Qualifications

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) Senior - Health Physicist (radiation protection manager) who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall meet the qualifications specified by the 1985 NRC Policy Statement on Engineering Expertise on Shift.

6.4 Training

- 6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Superintendent-Training and shall meet the requirements of Section 5.5 of ANSI N18.1-1971 and 10 CFR 55, Appendix A.

6.4.2 DELETED

6.5 Review and Audit

6.5.1 Plant Operations Review Committee (PORC)

6.5.1.1 Function

The Plant Operations Review Committee shall function to advise the Plant Manager on all matters related to nuclear safety.

6.5.1.2 Composition

The Plant Operations Review Committee is composed of nine regular members from Peach Bottom Atomic Power Station staff. Members shall collectively have experience in the following areas:

Plant Operations
Engineering
Maintenance
Instrumentation and Controls
Planning
Radiation Safety
Chemistry
Experience Assessment

Members shall meet the requirements of ANSI N18.1-1971, Sections 4.2, 4.4, or 4.6 for applicable required experience and be appointed in writing by the Plant Manager. The Chairman and alternate Chairmen of the PORC shall be drawn from the PORC members and appointed in writing by the Plant Manager.

6.5.1.3 Alternates

Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternate members shall be used to satisfy a PORC quorum (See 6.5.1.5).

PBAPS

6.5.1.6 Continued

- i. Review of the Plant Security Plan, and shall submit recommended changes to the Plan to the Plant Manger and the Nuclear Review Board.
- j. Review of the Emergency Plan, and shall submit recommended changes to the Plan to the Plant Manager and the Nuclear Review Board.
- k. Review of every unplanned release reportable under 10CFR 50.72 and 50.73, of radioactive material to the environs; evaluate the event; specify remedial action to prevent recurrence; and document the event description, evaluation, and corrective action and the disposition of the corrective action in the plant records.
- l. Review of the facility Fire Protection Program, and shall submit recommended changes to the Program to the Plant Manager and the Nuclear Review Board.

Authority

6.5.1.7 The Plant Operations Review Committee shall:

- a. Recommend in writing, to the Plant Manager, for his approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(b) through (e) above constitutes an unreviewed safety question, as defined in 10 CFR 50.59.
- c. Provide immediate written notification to the Vice President, Peach Bottom Atomic Power Station, or in his absence, the Senior Vice President - Nuclear and the Nuclear Review Board of disagreement between the PORC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

PBAPS

6.5.2.8 Continued

- e. The Facility Emergency Plan and implementing procedures at least once per year.
- f. The Facility Security Plan and implementing procedures at least once per two years.
- g. The Offsite Dose Calculation Manual and implementing procedures at least once per two years.
- h. The performance of activities required by the Quality Assurance Program regarding the radiological monitoring program to meet the provisions of Regulatory Guide 4.1, Revision 1, April 1975, at least once per calendar year.
- i. Any other area of facility operation considered appropriate by the NRB or the Executive Vice President - Nuclear.
- j. The facility Fire Protection Program and implementing procedures at least once per year.

Authority

- 6.5.2.9 The NRB shall report to and advise the Executive Vice President - Nuclear on those areas of responsibility specified in Section 6.5.2.7 and 6.5.2.8.

Records

- 6.5.2.10 Records of NRB activities shall be prepared, approved and distributed as indicated below:
- a. Minutes of each NRB meeting shall be prepared, approved and forwarded to the Executive Vice President - Nuclear within 10 working days following each meeting.
 - b. Reports of reviews encompassed by Section 6.5.2.7.e, f, g, and h above shall be prepared, approved and forwarded to the Executive Vice President - Nuclear, within 10 working days following completion of the review.

PBAPS

6.6 Reportable Event Action

6.6.1 The following actions shall be taken for Reportable Events:

- a. The Commission shall be notified pursuant to the requirements of Section 50.73 to 10 CFR 50.
- b. Each Reportable Event Report submitted to the Commission shall be reviewed by the PORC and submitted to the NRB and the Vice President, Peach Bottom Atomic Power Station.

6.7 Safety Limit Violation

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Vice President, Peach Bottom Atomic Power Station, Plant Manger, and the NRB shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NRB and the Vice President, Peach Bottom Atomic Power Station within 10 working days of the violation.

6.8 Procedures

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained as follows, except as provided in Technical Specifications 6.8.2 and 6.8.3:

- a. Procedures that meet the requirements of Sections 5.1 and 5.3 of ANSI N18.7-1972
- b. Procedures that meet the requirements of Appendix "A" of USAEC Regulatory Guide 1.33 (November 1972)
- c. Procedures covering radiological effluent technical specification activities, including the Offsite Dose Calculation Manual and Quality Assurance Program for environmental monitoring using the guidance in Regulatory Guide 4.1, Revision 1, April 1975.
- d. Procedures for the implementation of the Fire Protection Program.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PHILADELPHIA ELECTRIC COMPANY

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 198
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated March 28, 1994, as supplemented on June 27, 1994, and July 8, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. DPR-56 paragraph 2.C.(4) is hereby amended to read as follows:
 - 2.C.(4) The licensee shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility, and as approved in the NRC SER dated May 23, 1979 and Supplements dated August 14, September 15, October 10 and November 24, 1980, and in

the NRC SER dated September 16, 1993, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. Further, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 198, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Mohan C. Thadani, Acting Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachments:

1. Page 4a of the License*
2. Changes to the Technical Specifications

Date of Issuance: August 24, 1994

*Page 4a is attached, for convenience, for the composite license to reflect this change.

- 2.C.(4) The licensee shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility, and as approved in the NRC SER dated May 23, 1979 and Supplements dated August 14, September 15, October 10 and November 24, 1980, and in the NRC SERs dated September 16, 1993 and August 24, 1994 subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

- 2.C.(5) Operation beyond the end-of-cycle (all rods out condition) thermal power is limited to seventy (70) percent minimum. Increasing core power level via reduced feedwater heating, once operation in the coastdown mode has begun, is not permitted unless the licensee has performed an analysis of this operating condition that confirms that this condition is bounded by the analysis for the particular cycle of operation.

3. This license is subject to the following conditions for the protection of the environment:

- A. If the actual milk sample measurements taken at the nearby farms in accordance with the Technical Specifications, predict a dose to a child's thyroid, based on actual combined operation of Unit 2 and Unit 3, that exceeds 15 mrem/year, licensees shall install for operation no later than the next refueling cycle for each unit the necessary equipment to reduce the projected dose to such levels.

ATTACHMENT TO LICENSE AMENDMENT NO. 198

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

<u>Remove</u>	<u>Insert</u>
240c	240c
240d through 240s	240d through 240s
244	244
246	246
248	248
252	252
253	253

PBAPS

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PBAPS

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6.2.2 Facility Staff

The facility organization shall be subject to the following:

- a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2.1, except that the shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
- b. At least one licensed operator shall be in the control room and assigned to each reactor that contains fuel.
- c. At least two licensed operators, excluding the operator on the second unit, shall be present in the control room during reactor startup, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be onsite when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibility during this operation.
- f. Deleted.
- g. In addition to the requirements of 6.2.2.a, the Senior Manager-Operations shall hold a Senior Reactor Operator license.

6.3 Facility Staff Qualifications

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) Senior - Health Physicist (radiation protection manager) who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall meet the qualifications specified by the 1985 NRC Policy Statement on Engineering Expertise on Shift.

6.4 Training

- 6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Superintendent-Training and shall meet the requirements of Section 5.5 of ANSI N18.1-1971 and 10 CFR 55, Appendix A.

6.4.2 DELETED

6.5 Review and Audit

6.5.1 Plant Operations Review Committee (PORC)

6.5.1.1 Function

The Plant Operations Review Committee shall function to advise the Plant Manager on all matters related to nuclear safety.

6.5.1.2 Composition

The Plant Operations Review Committee is composed of nine regular members from Peach Bottom Atomic Power Station staff. Members shall collectively have experience in the following areas:

Plant Operations
 Engineering
 Maintenance
 Instrumentation and Controls
 Planning
 Radiation Safety
 Chemistry
 Experience Assessment

Members shall meet the requirements of ANSI N18.1-1971, Sections 4.2, 4.4, or 4.6 for applicable required experience and be appointed in writing by the Plant Manager. The Chairman and alternate Chairmen of the PORC shall be drawn from the PORC members and appointed in writing by the Plant Manager.

6.5.1.3 Alternates

Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternate members shall be used to satisfy a PORC quorum (See 6.5.1.5).

PBAPS

6.5.1.6 Continued

- i. Review of the Plant Security Plan, and shall submit recommended changes to the Plan to the Plant Manger and the Nuclear Review Board.
- j. Review of the Emergency Plan, and shall submit recommended changes to the Plan to the Plant Manager and the Nuclear Review Board.
- k. Review of every unplanned release reportable under 10CFR 50.72 and 50.73, of radioactive material to the environs; evaluate the event; specify remedial action to prevent recurrence; and document the event description, evaluation, and corrective action and the disposition of the corrective action in the plant records.
- l. Review of the facility Fire Protection Program, and shall submit recommended changes to the Program to the Plant Manager and the Nuclear Review Board.

Authority

6.5.1.7 The Plant Operations Review Committee shall:

- a. Recommend in writing, to the Plant Manager, for his approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(b) through (e) above constitutes an unreviewed safety question, as defined in 10 CFR 50.59.
- c. Provide immediate written notification to the Vice President, Peach Bottom Atomic Power Station, or in his absence, the Senior Vice President - Nuclear and the Nuclear Review Board of disagreement between the PORC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

PBAPS

6.5.2.8 Continued

- e. The Facility Emergency Plan and implementing procedures at least once per year.
- f. The Facility Security Plan and implementing procedures at least once per two years.
- g. The Offsite Dose Calculation Manual and implementing procedures at least once per two years.
- h. The performance of activities required by the Quality Assurance Program regarding the radiological monitoring program to meet the provisions of Regulatory Guide 4.1, Revision 1, April 1975, at least once per calendar year.
- i. Any other area of facility operation considered appropriate by the NRB or the Executive Vice President - Nuclear.
- j. The facility Fire Protection Program and implementing procedures at least once per year.

Authority

- 6.5.2.9 The NRB shall report to and advise the Executive Vice President - Nuclear on those areas of responsibility specified in Section 6.5.2.7 and 6.5.2.8.

Records

- 6.5.2.10 Records of NRB activities shall be prepared, approved and distributed as indicated below:
- a. Minutes of each NRB meeting shall be prepared, approved and forwarded to the Executive Vice President - Nuclear within 10 working days following each meeting.
 - b. Reports of reviews encompassed by Section 6.5.2.7.e, f, g, and h above shall be prepared, approved and forwarded to the Executive Vice President - Nuclear, within 10 working days following completion of the review.

PBAPS

6.6 Reportable Event Action

6.6.1 The following actions shall be taken for Reportable Events:

- a. The Commission shall be notified pursuant to the requirements of Section 50.73 to 10 CFR 50.
- b. Each Reportable Event Report submitted to the Commission shall be reviewed by the PORC and submitted to the NRB and the Vice President, Peach Bottom Atomic Power Station.

6.7 Safety Limit Violation

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Vice President, Peach Bottom Atomic Power Station, Plant Manger, and the NRB shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NRB and the Vice President, Peach Bottom Atomic Power Station within 10 working days of the violation.

6.8 Procedures

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained as follows, except as provided in Technical Specifications 6.8.2 and 6.8.3:

- a. Procedures that meet the requirements of Sections 5.1 and 5.3 of ANSI N18.7-1972
- b. Procedures that meet the requirements of Appendix "A" of USAEC Regulatory Guide 1.33 (November 1972)
- c. Procedures covering radiological effluent technical specification activities, including the Offsite Dose Calculation Manual and Quality Assurance Program for environmental monitoring using the guidance in Regulatory Guide 4.1, Revision 1, April 1975.
- d. Procedures for the implementation of the Fire Protection Program.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 194 AND 198 TO FACILITY OPERATING

LICENSE NOS. DPR-44 and DPR-56

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3

DOCKET NOS. 50-277 AND 50-278

1.0 INTRODUCTION

By letter dated March 28, 1994, as supplemented on June 27, 1994 and July 8, 1994, the Philadelphia Electric Company (the licensee) submitted a request for changes to the Peach Bottom Atomic Power Station, Unit Nos. 2 and 3, Technical Specifications (TS). The requested changes would relocate the TS Fire Protection requirements to the Updated Final Safety Analysis Report (UFSAR) in accordance with the guidance in Generic Letter (GL) 86-10, "Implementation of Fire Protection Requirements," and GL 88-12, "Removal of Fire Protection Requirements from Technical Specifications." The licensee's supplemental letters provided additional information which did not change the staff's original no significant hazards consideration determination.

2.0 BACKGROUND

Section 50.36 of Title 10 of the Code of Federal Regulations established the regulatory requirements related to the content of technical specifications (TS). The rule requires that technical specifications include items in specific categories, including safety limits, limiting conditions for operation, and surveillance requirements; however, the rule does not specify the particular requirements to be included in a plant's TS. The NRC developed criteria, as described in the "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors," (58 FR 39132) to determine which of the design conditions and associated surveillances need to be located in the TS because the requirement is "necessary to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety." Briefly, those criteria are (1) detection of abnormal degradation of the reactor coolant pressure boundary, (2) boundary conditions for design basis accidents and transients, (3) primary success paths to prevent or mitigate design basis accidents and transients, and (4) functions determined to be important to risk or operating experience.

The Commission's final policy statement acknowledged that its implementation may result in the relocation of existing technical specification requirements to licensee controlled documents and programs.

The following changes were proposed by the licensee:

- (1) Replace the existing Fire Protection license condition on page 4a of the facility operating license (FOL), (FOL Section 2.C.(5) for Unit 2, Section 2.C.(4) for Unit 3), with the following statements:

"PECO Energy Company shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility, and as approved in the NRC SER [Safety Evaluation Report] dated May 23, 1979 and Supplements dated August 14, September 15, October 10 and November 24, 1980, and in the NRC SERs dated September 16, 1993 and August 24, 1994, subject to the following provision:

PECO Energy Company may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire."

This change would replace the existing FOL fire protection license condition with the standard fire protection license condition discussed in GL 86-10.

- (2) Delete current page 240c and insert new page 240c with the following statement:

"The Fire Protection Limiting Condition for Operation and Surveillance Requirements have been relocated to the Fire Protection Program [FPP], which is part of the UFSAR. These requirements are still mandated by the Facility Operating License."

This change provides clarification on the relocation of the Fire Protection requirements from TS to the FPP.

- (3) Delete pages 240d through 240s in their entirety and replace them with a single page which states:

"LEFT BLANK INTENTIONALLY"

These pages, which currently include TS sections 3.14, "Fire Protection Limiting Conditions for Operation (LCO)" and 4.14, "Surveillance Requirements (SRs)" along with the associated Bases, are proposed to be relocated to the FPP which is in the UFSAR.

- (4) Delete 6.2.2(f) of Section 6.2.2, "Facility Staff," on page 244, which states:

"A Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of the unit(s) (3 members) or any personnel required for other essential functions during a fire emergency."

This change would relocate the requirement to the FPP.

- (5) Delete 6.4.2 of Section 6.4, "Training," on page 246, which states:

"A training program for the Fire Brigade shall be conducted such that Fire Brigade members complete an instruction program within a two year period. Regularly planned meetings will be held every 3 months."

This change would relocate the requirement to the FPP.

- (6) Insert proposed 6.5.1.6(1) of Section 6.5.1.6, "Responsibilities," which states:

"Review of the facility Fire Protection Program, and shall submit recommended changes to the Program to the Plant Manager and the Nuclear Review Board."

This addition is consistent with the guidance of GL 88-12.

- (7) Insert proposed 6.5.2.8(j) of Section 6.5.2.8, "Audit," which states:

"The facility Fire Protection Program and implementing procedures at least once per year."

The licensee's original submittal requested a once per two year frequency for this audit. However, the licensee's June 27, 1994, supplemental letter changed the requested audit frequency to at least once per year. This revision from the original submittal was made in order for the new TS to be consistent with the fire protection inspection required by TS 6.12.a.

- (8) Insert proposed 6.8.1(d) of Section 6.8, "Procedures," which states:

"Procedures for the implementation of the Fire Protection Program."

This change requires that written procedures be established, implemented and maintained for the implementation of the FPP.

3.0 EVALUATION

The staff reviewed the licensee's request against the guidance provided in GLs 86-10 and 88-12, and the policy statement discussed above. Generic Letter 86-10 requested that licensees incorporate the NRC-approved FPP in their UFSAR, and specified a standard fire protection license condition. Generic Letter

88-12 addressed the elements a licensee should include in a license amendment request to remove the fire protection requirements from the TS. These elements are (1) the NRC-approved FPP must be incorporated into the UFSAR; (2) the Limiting Conditions for Operations and Surveillance Requirements associated with the fire detection systems, fire suppression systems, fire barriers, and the administrative controls that address fire brigade staffing would be deleted from the TS (the existing administrative controls related to fire protection audits would be retained in the TS); (3) all operational conditions, remedial actions, and test requirements presently included in the TS for these systems, as well as the fire brigade staffing requirements, shall be incorporated into the FPP; (4) the standard fire protection license condition specified in GL 86-10 must be included in the facility operating license; (5) the Unit Review Group (Onsite Review Group) shall be given responsibility for the review of the FPP and implementing procedures and for the submittal of recommended changes to the Company Nuclear Review and Audit group (Off-site or Corporate Review Group); and (6) the FPP implementation shall be added to the list of elements for which written procedures shall be established, implemented, and maintained.

The licensee previously incorporated the NRC-approved FPP into the UFSAR (see UFSAR Section 10.12). In a June 29, 1994 letter, the licensee also incorporated the current TS operability and surveillance requirements for the fire detection system, fire suppression systems, and fire rated assemblies into its UFSAR. Additionally, the June 29, 1994 letter incorporated the TS requirements related to fire brigade staffing into the UFSAR. Therefore, the licensee has satisfied Elements 1, 2, and 3 of GL 88-12.

The licensee proposed the standard fire protection license condition specified in GL 86-10. The licensee has, therefore, satisfied Element 4 of GL 88-12. The inclusion of the standard license condition also ensures that the criteria of the Commission's Policy Statement will be met. Since the FPP requirements are included in the UFSAR, the change control process (under which the licensee would evaluate whether a change affected safe shutdown capability) is 10 CFR 50.59. However, as noted in the Background section above, the licensee's request included reference to the "PECO Energy Company" as the licensee. The staff has not completed the licensee's April 15, 1994 amendment request to change the licensed name from "Philadelphia Electric Company" to "PECO Energy Company." In a July 8, 1994 telephone call, a representative of the licensee stated that the FPP amendment had a higher priority than the name change amendment. Therefore, the NRC staff proposed to replace "PECO Energy Company" with "The licensee" in the new license condition. The licensee's representative concurred with the staff's proposal.

To satisfy Elements 5 and 6 of GL 88-12, the licensee is adding TSs 6.5.1.6.1 and 6.8.1.d. The licensee has, therefore, satisfied Elements 5 and 6 of GL 88-12.

Thus, all of the changes proposed by the licensee are consistent with the guidance provided in GLs 86-10 and 88-12.

On this basis, the staff concludes that the FPP TS do not need to be controlled by TS, and changes to the FPP are adequately controlled by 10 CFR 50.59, "Changes, tests, and experiments." Should the licensee's determination conclude that an unreviewed safety question is involved, due to either (1) an increase in the probability or consequences of accidents or malfunctions of equipment important to safety, (2) the creation of a possibility for an accident or malfunction of a different type than any evaluated previously, or (3) a reduction in the margin of safety, NRC approval and a license amendment would be required prior to implementation of the change. NRC inspection and enforcement programs also enable the staff to monitor facility changes and licensee adherence to UFSAR commitments and to take any remedial action that may be appropriate.

The staff has concluded, therefore, that relocation of FPP TS is acceptable because (1) their inclusion in technical specifications is not specifically required by 10 CFR 50.36 or other regulations, (2) the FPP is not required to avert an immediate threat to the public health and safety, and (3) changes that are deemed to involve an unreviewed safety question, will require prior NRC approval in accordance with 10 CFR 50.59(c).

Separately, while reviewing the licensee's request the staff found that the PBAPS TS do not include TS for alternative shutdown equipment as specified in GL 81-12, "Fire Protection Rule (45 FR 76602, November 19, 1980)." The staff will pursue implementation of alternative safe shutdown equipment TS, consistent with GL 81-12, independent of this license amendment.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and surveillance requirements. The amendments also relate to changes in recordkeeping, reporting, or administrative procedures or requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (59 FR 22012). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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Date: August 24, 1994