

Calvert Cliffs Nuclear Power Plant Steam Generator Replacement

NRC Presentation

April 25, 2001

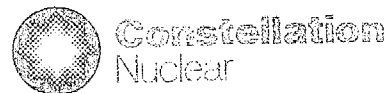
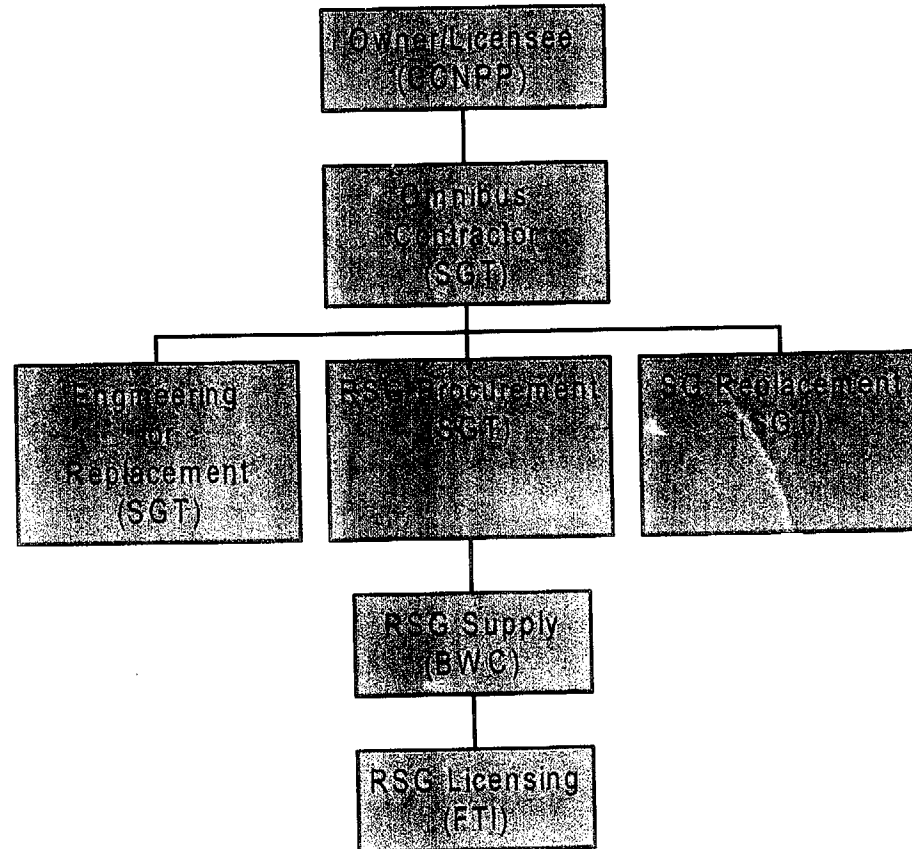


Reasons for Replacement

❖ Minimize:

- Extended Outages, Costly Inspections
- Mid-cycle Outages
- Expensive Repairs and Analysis
- Tube Leak Threat
- Reduced Power Operation

Functional Relationships



Project Milestone Schedule

- ❖ June 1998 - Contract for Replacement
- ❖ September 2001 - Unit I SGs Delivered
- ❖ February 2002 - Unit I Replacement Outage
- ❖ July 2002 - Unit II SGs Delivered
- ❖ March 2003 - Unit II Replacement Outage



Steam Generator Procurement Schedule

	NG1	NG2	NG3	NG4
Receipt of Tubesheet	13-Oct-98	14-Oct-98	22-Jan-99	22-Jan-99
Complete Gundrilling	28-May-99	21-Sep-99	1-Dec-99	15-Feb-00
Receipt of Primary Head	6-Aug-99	27-Aug-99	12-Oct-99	4-Nov-99
Complete Welding of Divider Plate to Primary Head	2-Sep-99	15-Nov-99	18-Jan-00	3-Apr-00
Complete Sec Shell Welds (except small nozzles)	16-May-00	21-Jun-00	21-Nov-00	3-Apr-01
Complete PWHT of Lower Section	29-Jun-00	29-Jul-00	10-Jan-01	24-May-01
Receipt of Tubes	14-Jul-00	15-Sep-00	22-Dec-00	22-Jan-01
Complete Installation of Tubes	9-Jan-01	30-Jan-01	4-Jun-01	30-Oct-01
Start Final Expansion of Tubes	19-Apr-01	28-May-01	12-Oct-01	1-Mar-02
Hydrotest	31-May-01	10-Aug-01	12-Dec-02	1-May-02
Ex-Cambridge	17-Aug-01	31-Aug-01	21-Jun-02	5-Jul-02
Arrive at Site	17-Sep-01	17-Sep-01	21-Jul-02	21-Jul-02



Replacement SG Design Features



❖ Primary

- Self Draining Channel Head
- 18" Primary Manways
- Chrome Plated Diaphragm Plates
- Electropolished Channel Head
- I690 Tubesheet Cladding
- I690TT Tubes with > 15:1 S/N Ratio
- I690 Divider Plate

Replacement SG Design Features

❖ Secondary

- Additional Handholes
- Improved Blowdown Capability
- Forged Secondary Shell
- Stainless Steel Tube Supports
- Tube Support Inspection Ports

Replacement SG Design Features

❖ Secondary

- ✓ ➤ Gooseneck Feedwater Nozzle Design
- Extended Thermal Sleeve
- Recirculation Nozzle
- Improved Moisture Separation Equipment
- Added Integral Main Steam Flow Restrictor /

Blow down feature.

*600 - 690
Manual*

*410 SS UBend bar
Lattas,
Non-Volume generation*

Reliability & Maintainability Features



RELIABILITY

ALL-WELDED SEPARATOR ASSEMBLY
MINIMIZES LOOSE PARTS RISK

SECONDARY SEPARATORS
- COMPACT
- LOW PRESSURE DROP
- PROVEN, WCO < 0.1%
(< 0.02% ACTUAL)
- NEVER NEEDS REPLACEMENT

CAP3 PRIMARY SEPARATORS
- LAB, FIELD PROVEN
- LOW CARRY OVER,
- INSENSITIVE TO WATER FLOW,
- WATER LEVEL

GOOSENECK FDM INLET
MINIMIZES STRATIFICATION,
ELIMINATES WATER HAMMER RISK

FLAT BAR U-BEND SUPPORTS

HIGH CIRCULATION RATIOS
- 4.5 OR GREATER
- TRADITIONAL B&W PRACTICE

LARGE MINIMUM RADIUS BENDS
- ELIMINATES PRIMARY SCC RISK

SELECTION OF TUBE MATERIAL
- FOR OPTIMUM CORROSION
RESISTANCE (ALLOY 690
RECOMMENDED)

LATTICE GRID TUBE SUPPORTS
- STAINLESS STEEL 410S MATL
- MORE OPEN TO FLOW
- MORE FLEXIBLE
- HIGHER STRENGTH
- LOWER PRESSURE DROP
- NOT PRONE TO CRUDDING
- NO TIE-RODS RECD

MAINTAINABILITY

SINGLE PIECE FORGED HEAD,
INTEGRAL NOZZLE FOR
REDUCED INSPECTION

ONE OR MORE 18" MANWAYS
HEAD OR SHELL INSTALLATION

ACCESS DUCT - FOR BUNDLE,
FDM HEADER, SECONDARY HEAD
INSPECTIONS

HANDHOLE - ACCESS TO OPENABLE
END OF FDM HEADER OR BUNDLE
TUBE FREE LANE

SECONDARY SHELL DESIGNED WITH
MINIMUM NUMBER OF WELDS, MOST
SUITABLE FOR AUTOMATED UT

EACH TUBE SUPPORT LEVEL
ACCESSIBLE BY INSPECTION PORT

RECESSED RIM IN TUBESHEET,
FULL DRAINABILITY, COLLECTOR
FOR LANCED SLUDGE

RECESSED BLOWDOWN MAXIMIZES
ACCESSIBILITY FOR TUBESHEET
SECONDARY FACE SLUDGE LANCING

FOUR HANDHOLES AT TUBESHEET FACE

EXTERNAL BLOWDOWN VALVING ALLOWS
SECONDARY SIDE DRAINING

GENEROUS CLEAR ACCESS, AT OUTERMOST
TUBE FOR MANIPULATORS, ETC

ELECTROPOLISHED CLADDING,
MINIMAL FIELDS

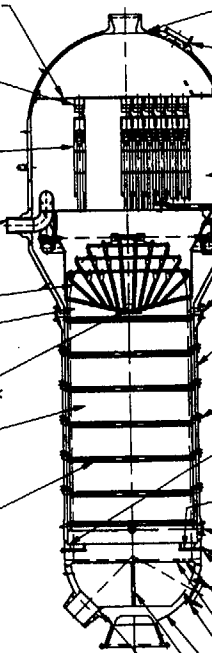
FLUSH SEAL WELDS

TWO 18" MANWAYS

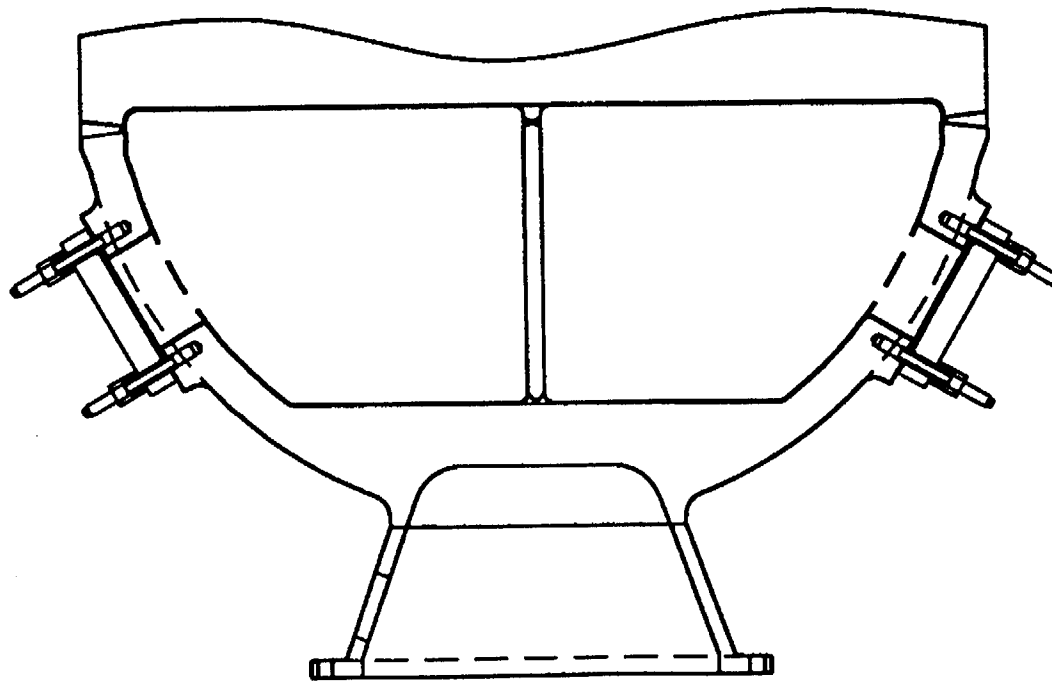
MONOBLOC FORGER HEAD,
NO WELDS TO INSPECT

WELDED DIVIDER PLATE,
NO CONTAINMENT TRAPS

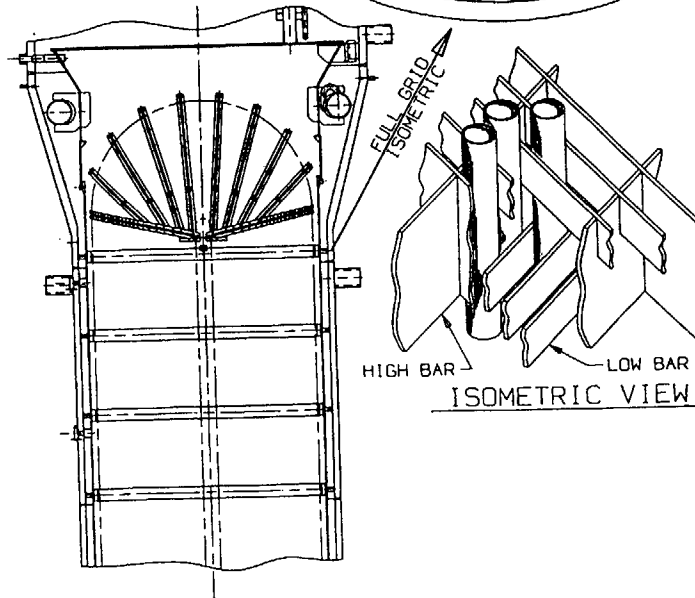
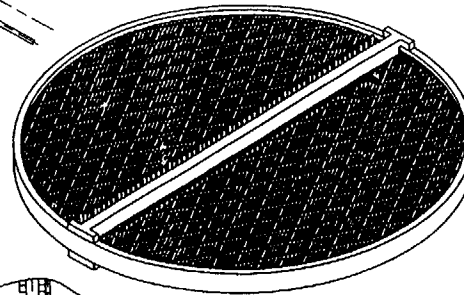
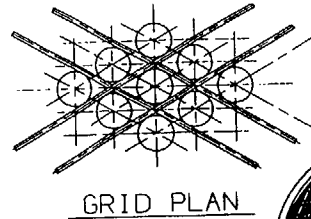
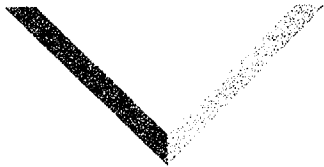
NOZZLE DAM GROOVES TO SUIT
PROPOSED EQUIPMENT



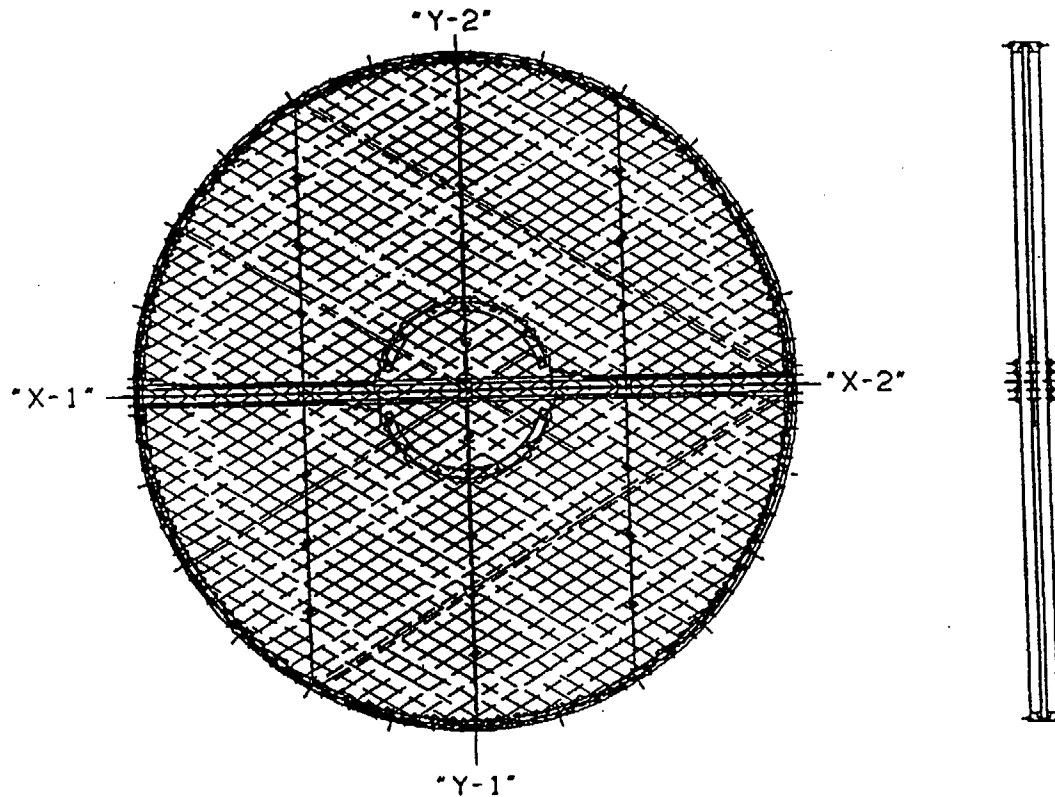
Primary Head Design



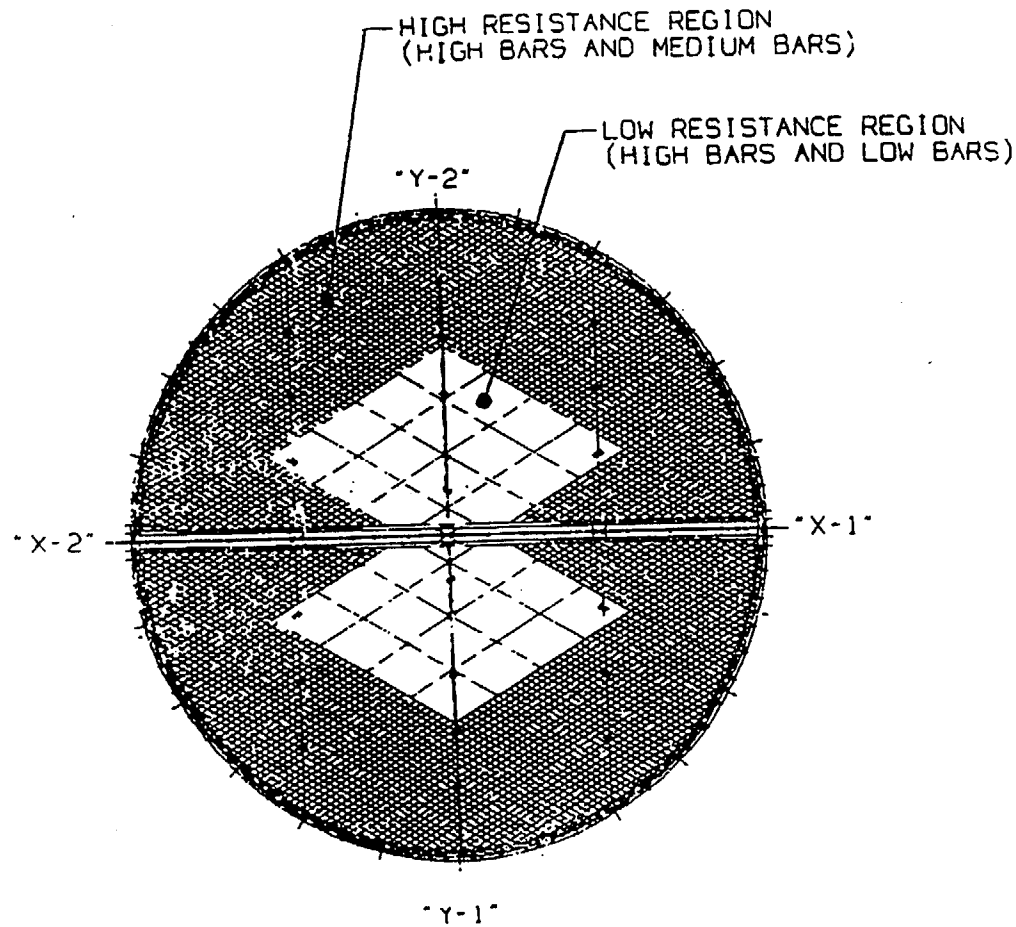
Lattice Grid Tube Supports



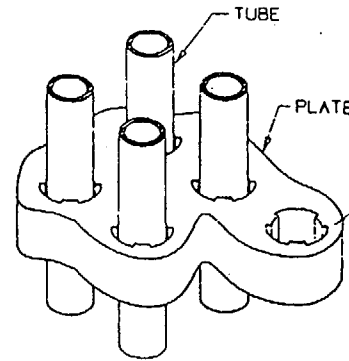
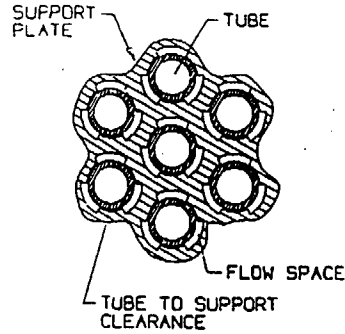
Lattice Grid Assembly



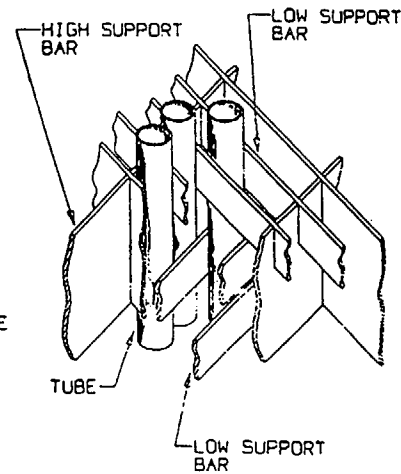
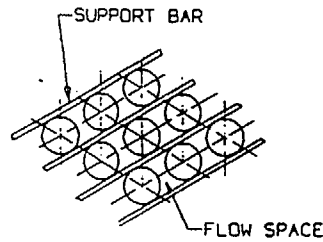
Differential Resistance Lattice



Lattice Grid Supports Free Flow Space

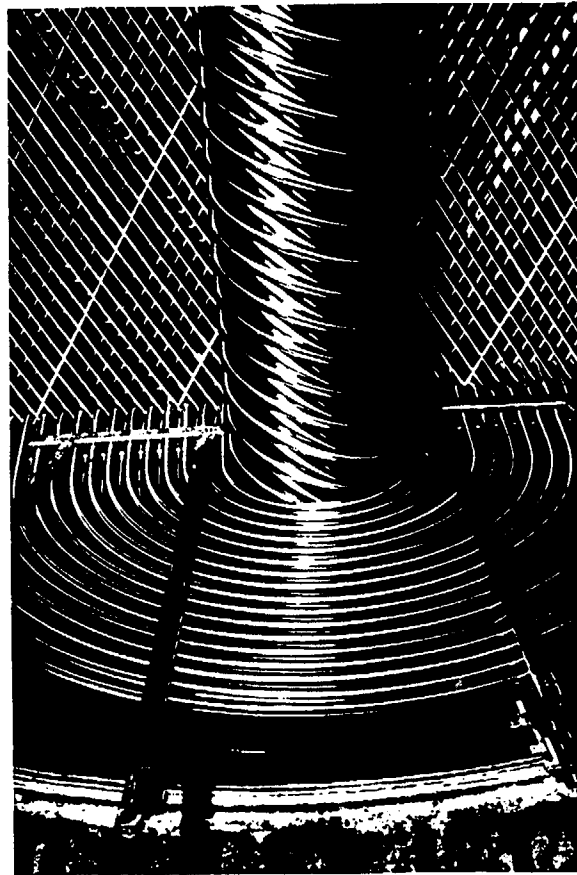


Comparison purpose

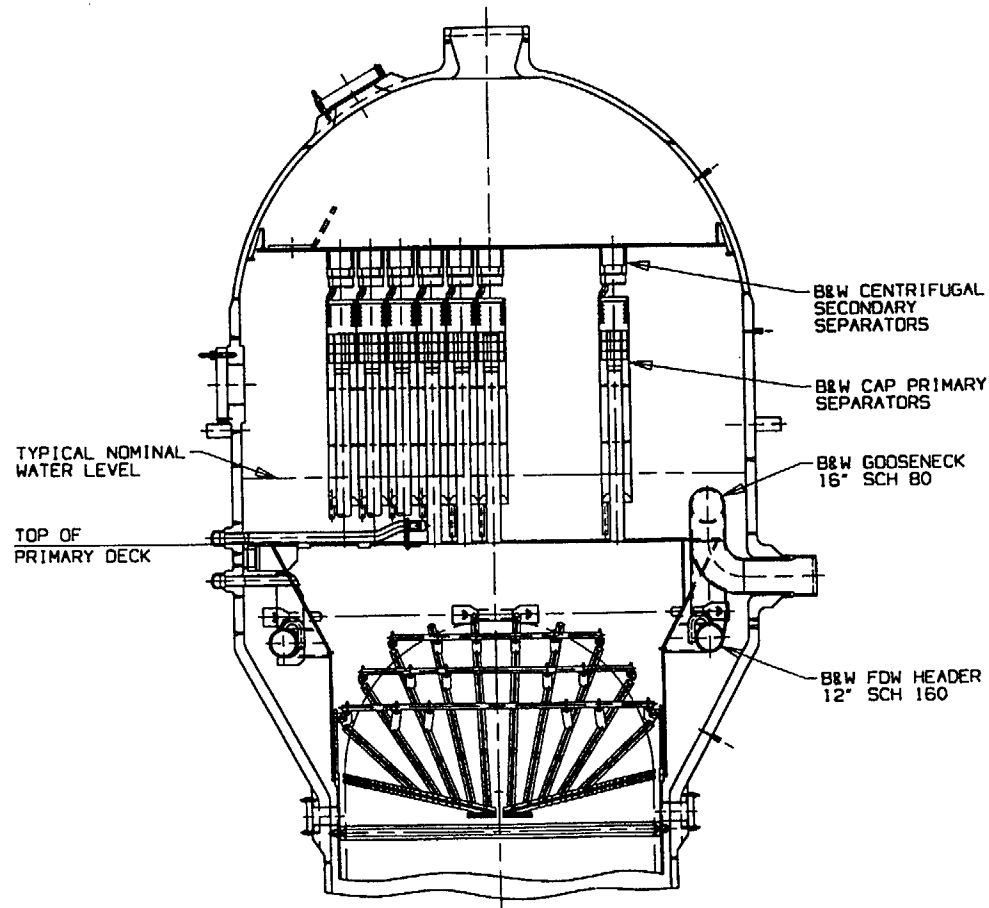


Lattice design.

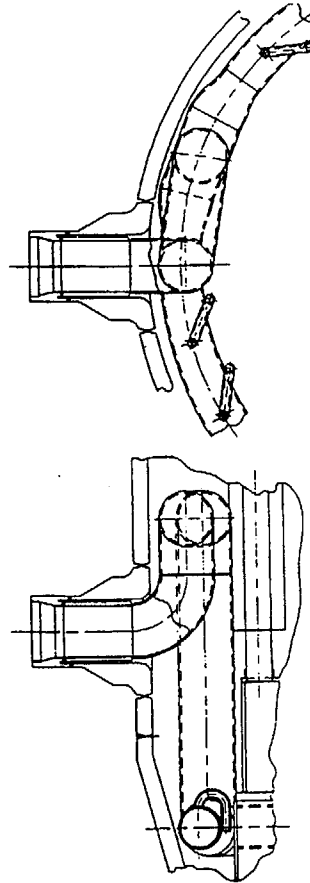
Type 410S Tube Support Material



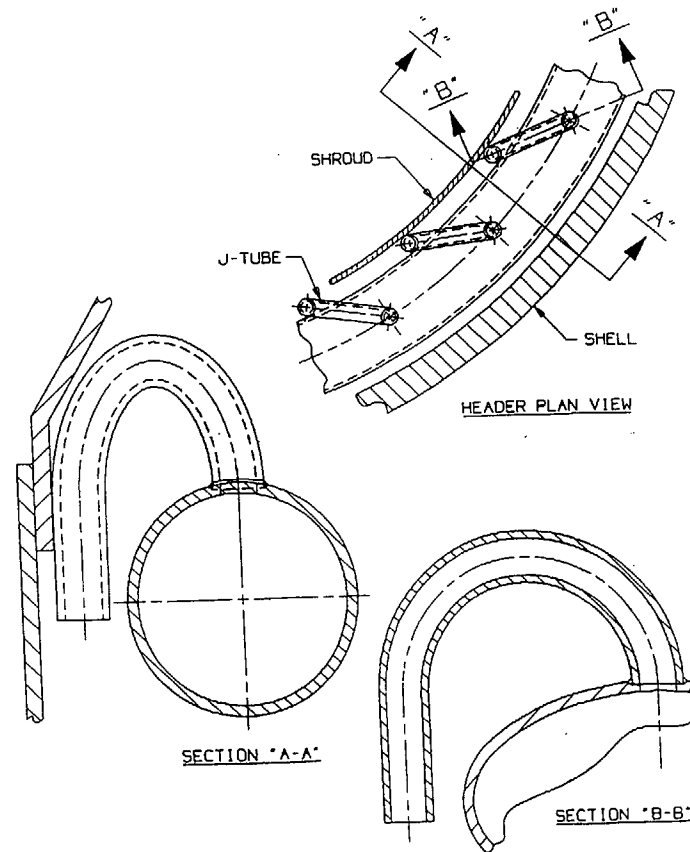
Steam Drum Arrangement



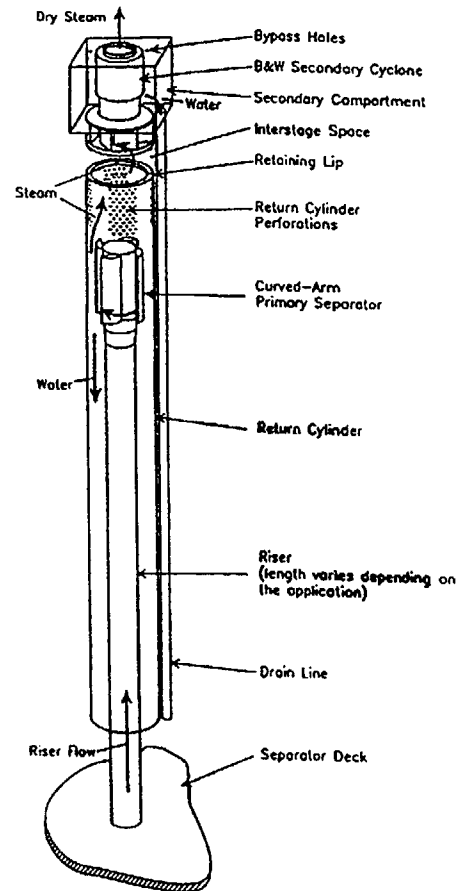
Feeding Header Distribution Systems



J-Tube Design



Primary & Secondary Separators



Replacement SG Design Features

❖ Secondary Performance Features

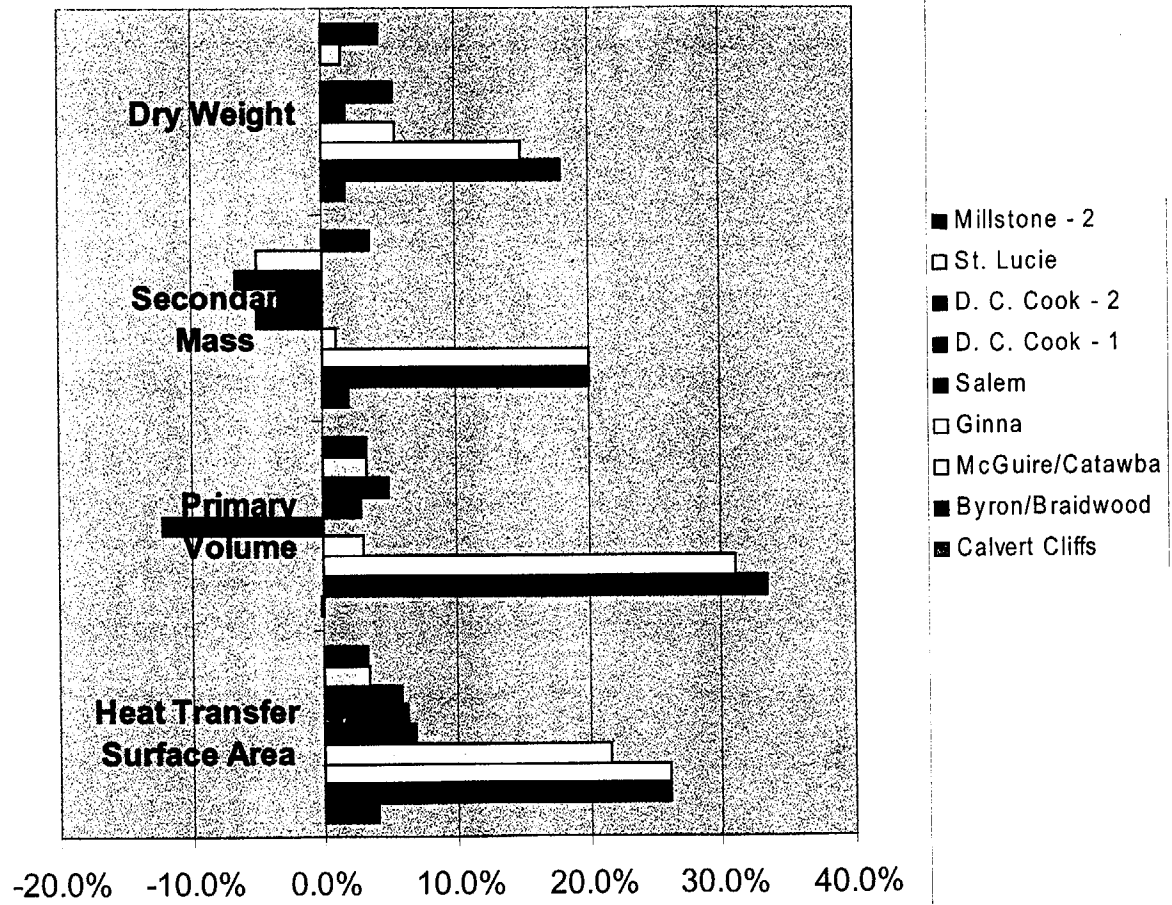
- Lower Void Fraction
- <0.05% Moisture Carryover
- Improved Circulation Ratio
- Improved Water Level Stability

System Operating Parameters

Will closely match present conditions

Parameter	Original Design	Replacement Design	% Change
Heat Transfer Area (sq ft)	90,200	92,008	+2.0%
Primary Volume (cu ft)	1,646	1,641	-0.30%
Secondary Inventory @100% (lbm)	135,986	138,525	+1.9%
Moisture Carryover @100% (%)	0.20	0.05	-75.0%
Circualtion Ratio	4.0	4.5	+12.5%
Tube Diameter (in)	0.75	0.75	-

Comparisons of SGRPs



% Change From Original

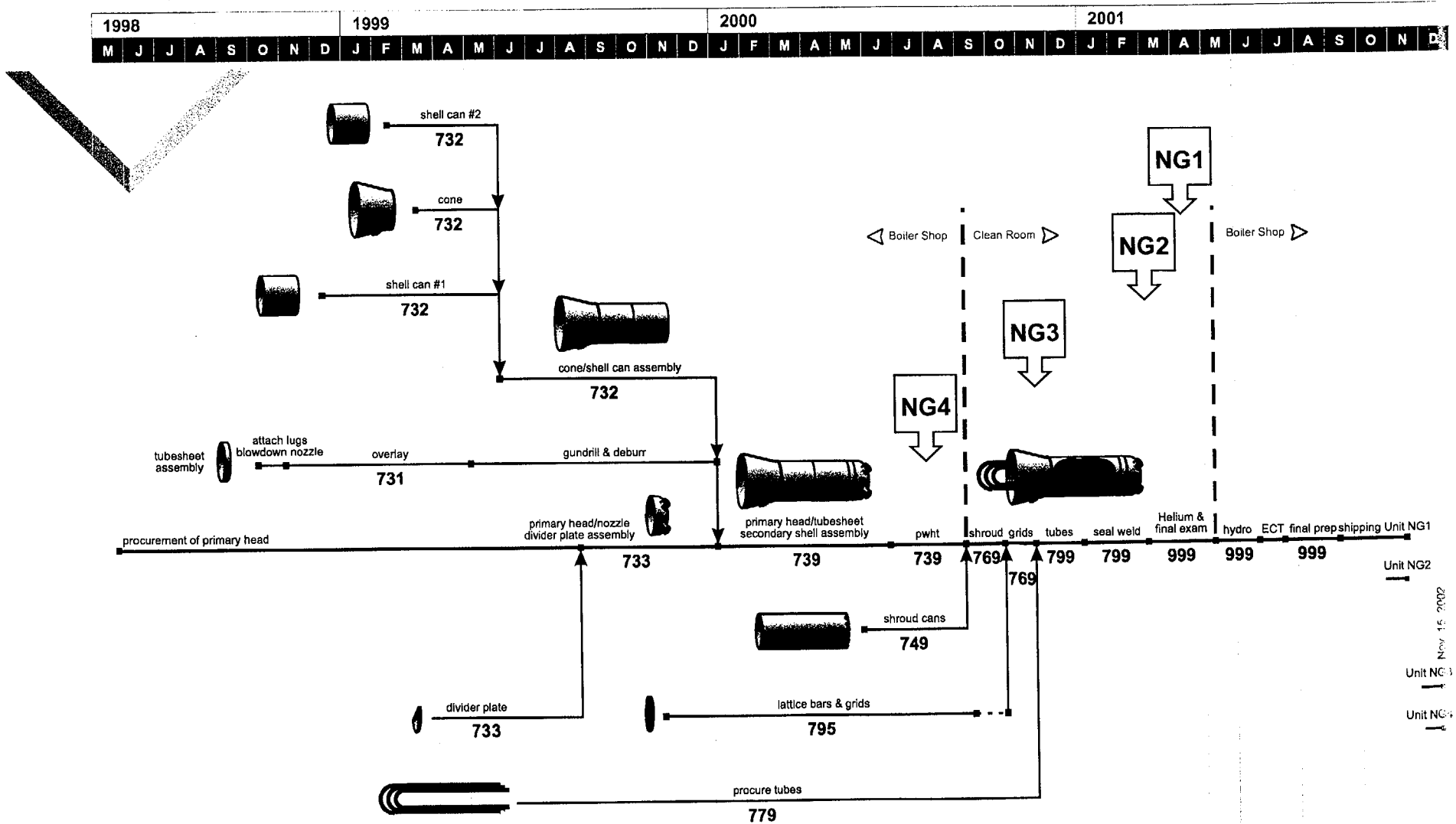


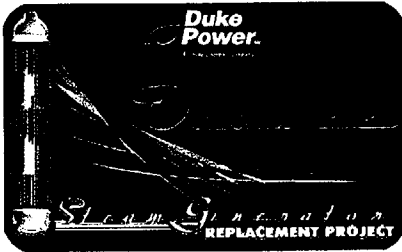
Transient Analyses

❖ RSG design minimizes reanalysis

- limit primary volume
- use existing level setpoints
- <1% increase in heat transfer capacity
- equivalent secondary operating pressure
- equivalent secondary volume
- equivalent primary dP

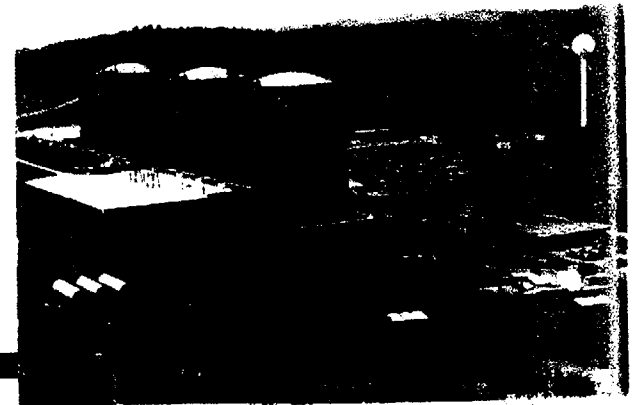
Calvert Cliffs Nuclear Power Plant Steam Generator Replacement Project

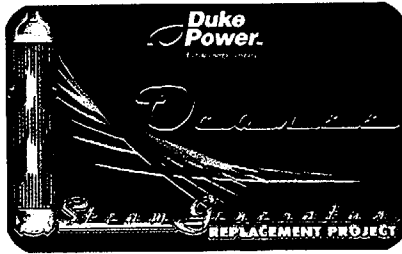




Duke Power's Project Overview Presentation to the Nuclear Regulatory Commission

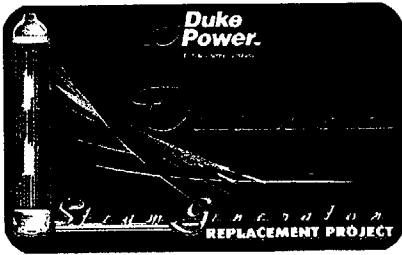
April 25, 2001



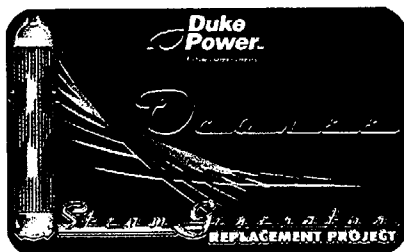


Agenda

- ◆ **Project Overview**
- ◆ **Steam Generator Procurement**
- ◆ **Replacement Steam Generator (ROTSG) Description**
- ◆ **Steam Generator Removal & Replacement**
- ◆ **Licensing**
- ◆ **Close**



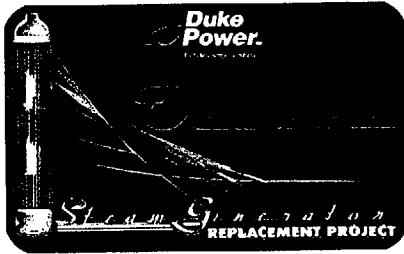
Project Overview



*75 days broken
to broken*

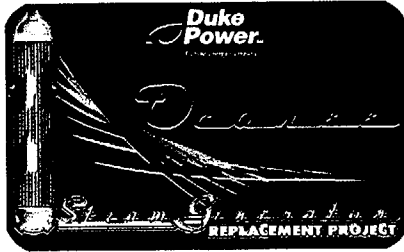
Meeting Objectives

- ◆ Inform the NRC of Duke's Plans to Replace Oconee's Steam Generators
- ◆ Present an Overview of the Project (Video)
- ◆ Discuss Anticipated Licensing Activities Related to the Project
- ◆ Present a Detailed Description of ROTSG Improvements



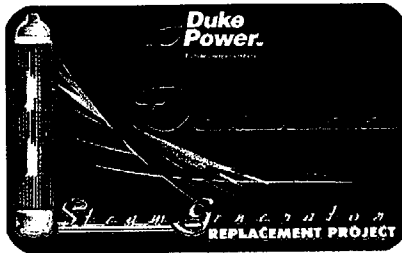
Background

- ◆ **Oconee Unit 1 Was First 177 FA Babcock & Wilcox Designed NSSS Placed in Service**
- ◆ **Commercial Operation**
 - ❖ Unit 1 - 7/15/73
 - ❖ Unit 2 - 9/9/74
 - ❖ Unit 3 - 12/16/74
- ◆ **2 Once Through Steam Generators Per Unit**
- ◆ **Unit 1 OTSGs Have Provided Longest Service of Any Currently Operating Steam Generators in U.S.**



Replacement Evaluation

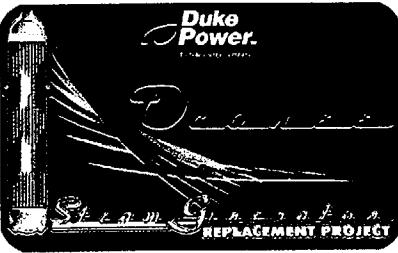
- ◆ Degradation of Tubes
- ◆ Current OTSGs Will Not Reach End of Current License
- ◆ License Extension Has Been Approved
- ◆ Cost Justified With or Without License Extension
- ◆ Evaluated New Vs. Existing OTSGs (Midland)
- ◆ Plan to Replace OTSGs for Units 1, 2 and 3 in Fall 2003, Spring 2004, and Fall 2004, Respectively



Total Tubes Repaired

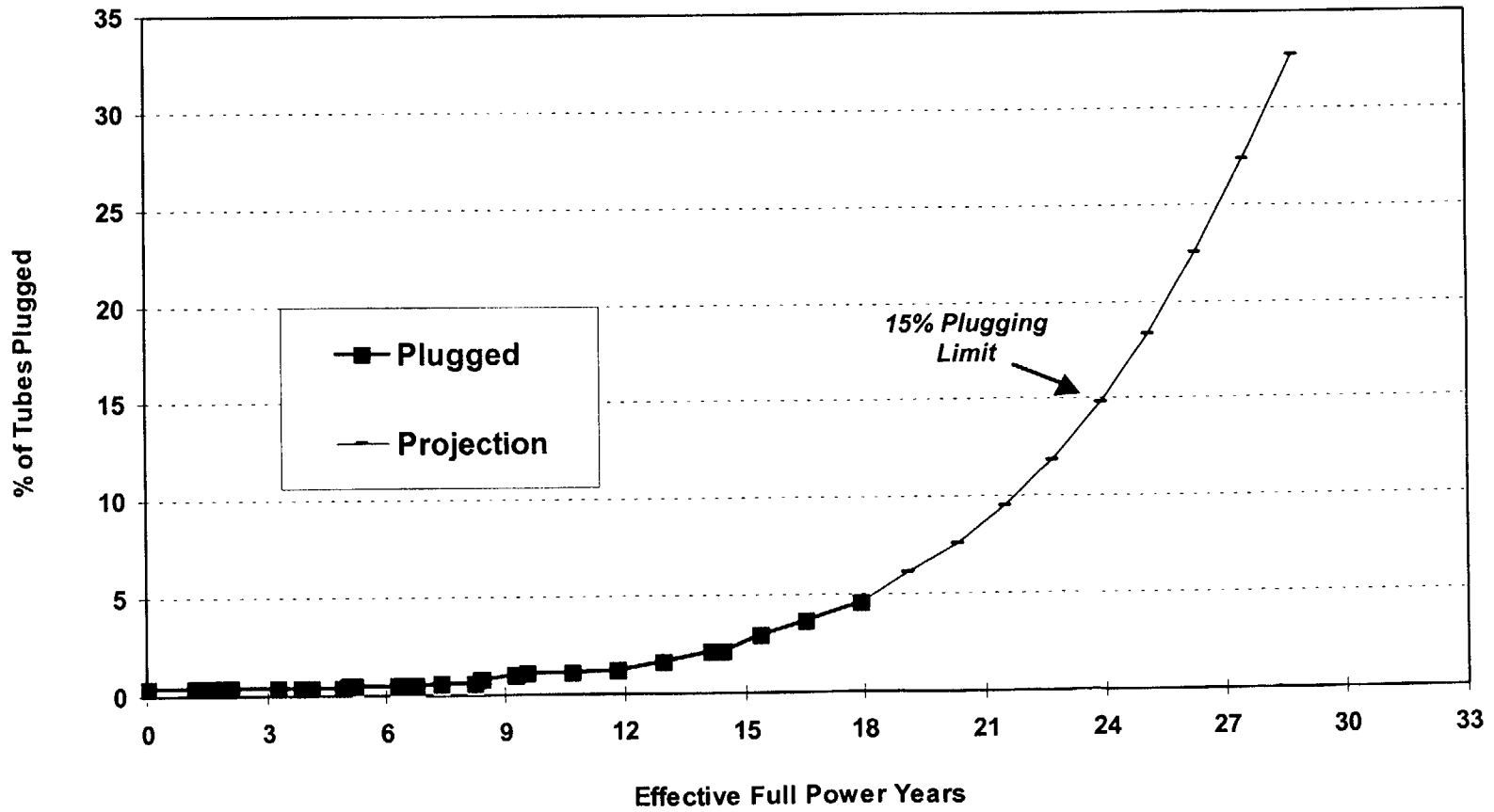
As of 1/4/01

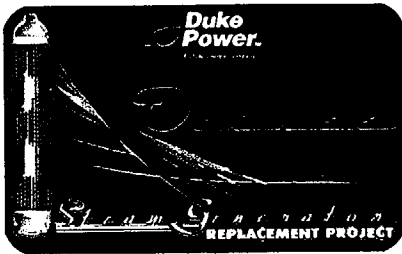
		<u>Total Tubes</u>	<u>Plugged</u>	<u>Percent Plugged</u>	<u>Rerolled</u>	<u>Sleeved In-Service</u>
ONS-1	1A	15,531	633	4.08%	2,618	247
	1B	15,531	1796	11.56%	2,380	180
ONS-1		31,062	2429	7.82%	4,795	427
ONS-2	2A	15,531	677	4.36%	117	275
	2B	15,531	893	5.75%	276	252
ONS-2		31,062	1,175	5.05%	256	533
ONS-3	3A	15,459	938	6.07%	126	281
	3B	15,531	686	4.42%	259	247
ONS-3		30,990	1,386	5.2%	385	528



Tube Degradation

Cumulative Tubes Plugged





Milestone Schedule

1998 1999 2000 2001 2002 2003 2004

ROTSG Bid Process



ROTSG Fabrication



Engineering



License Amendment Submittal



License Amendment Approval



Unit 1 Outage

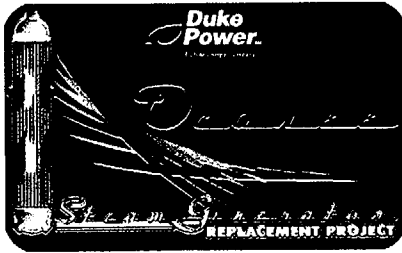


Unit 2 Outage



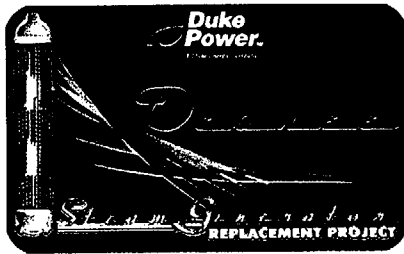
Unit 3 Outage





Key Project Strategies

- ◆ Life of Plant Design
- ◆ Like-for-Like Replacement
- ◆ Design Improvements
- ◆ Experienced Project Team



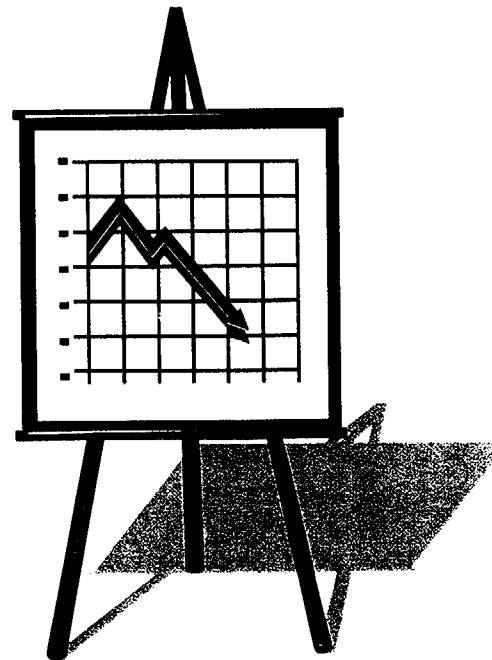
Project Goals

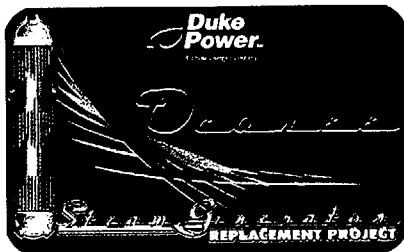
- ◆ Outage Duration
- ◆ Outage Dose

<75 Days

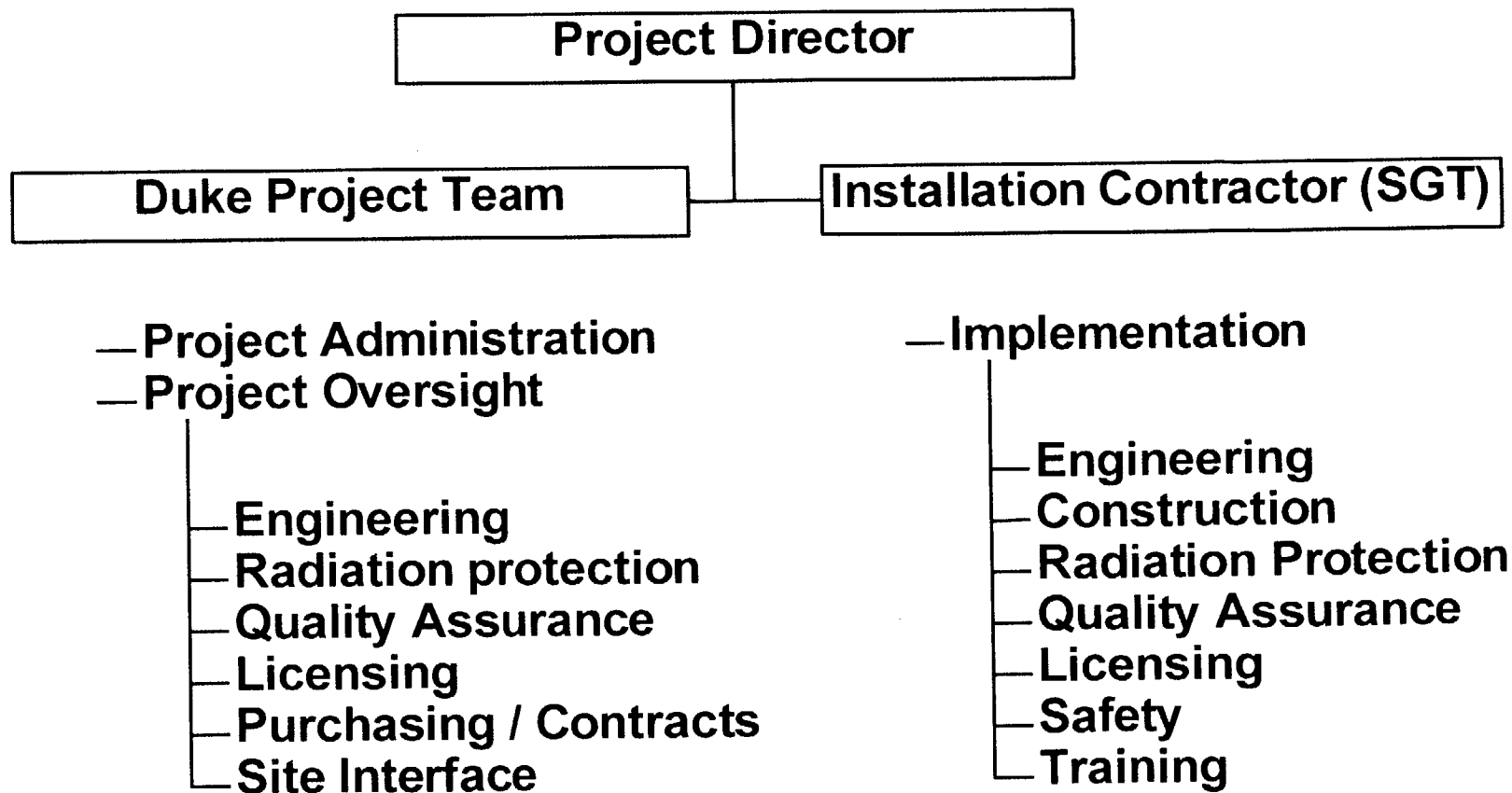
<147 Rem Per Unit

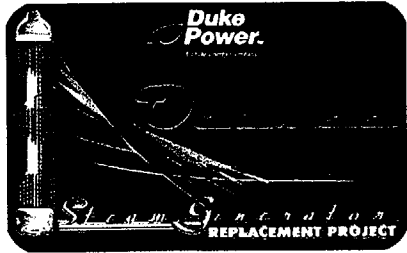
Now have to consider head replacement.



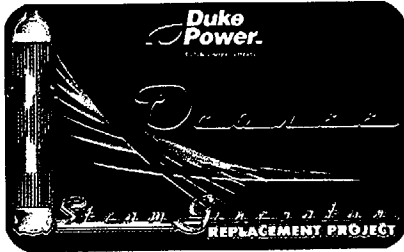


Project Organization



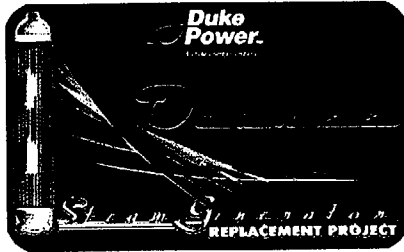


Steam Generator Procurement and Replacement Steam Generator Description



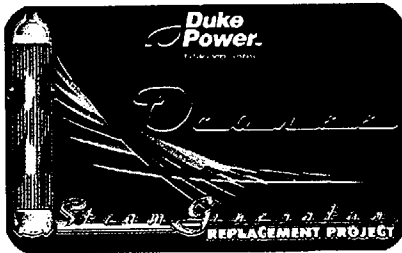
Replacement OTSG Procurement

- ◆ **Contract Awarded to B&W Canada (BWC) on March 31, 1999 for 6 Replacement Steam Generators**
- ◆ **Delivery**
 - ❖ Unit 1 - 6/1/03
 - ❖ Unit 2 - 1/1/04
 - ❖ Unit 3 - 7/1/04
- ◆ **Major Subsuppliers**
 - ❖ Tubes - Sumitomo
 - ❖ Heavy Forgings - Japan Steel Works
 - ❖ Tube Support Plates - Creusot Loire



Replacement OTSG Design Improvements

- ◆ Based on Duke and B&W Operating Experience
- ◆ Materials / Technology Improvements

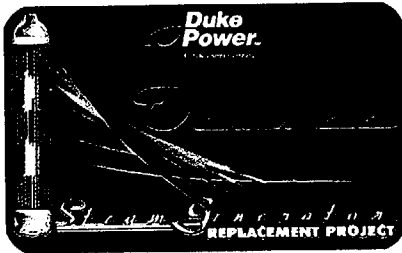


Replacement OTSG Design Improvements

22" tube sheet 2" thinner

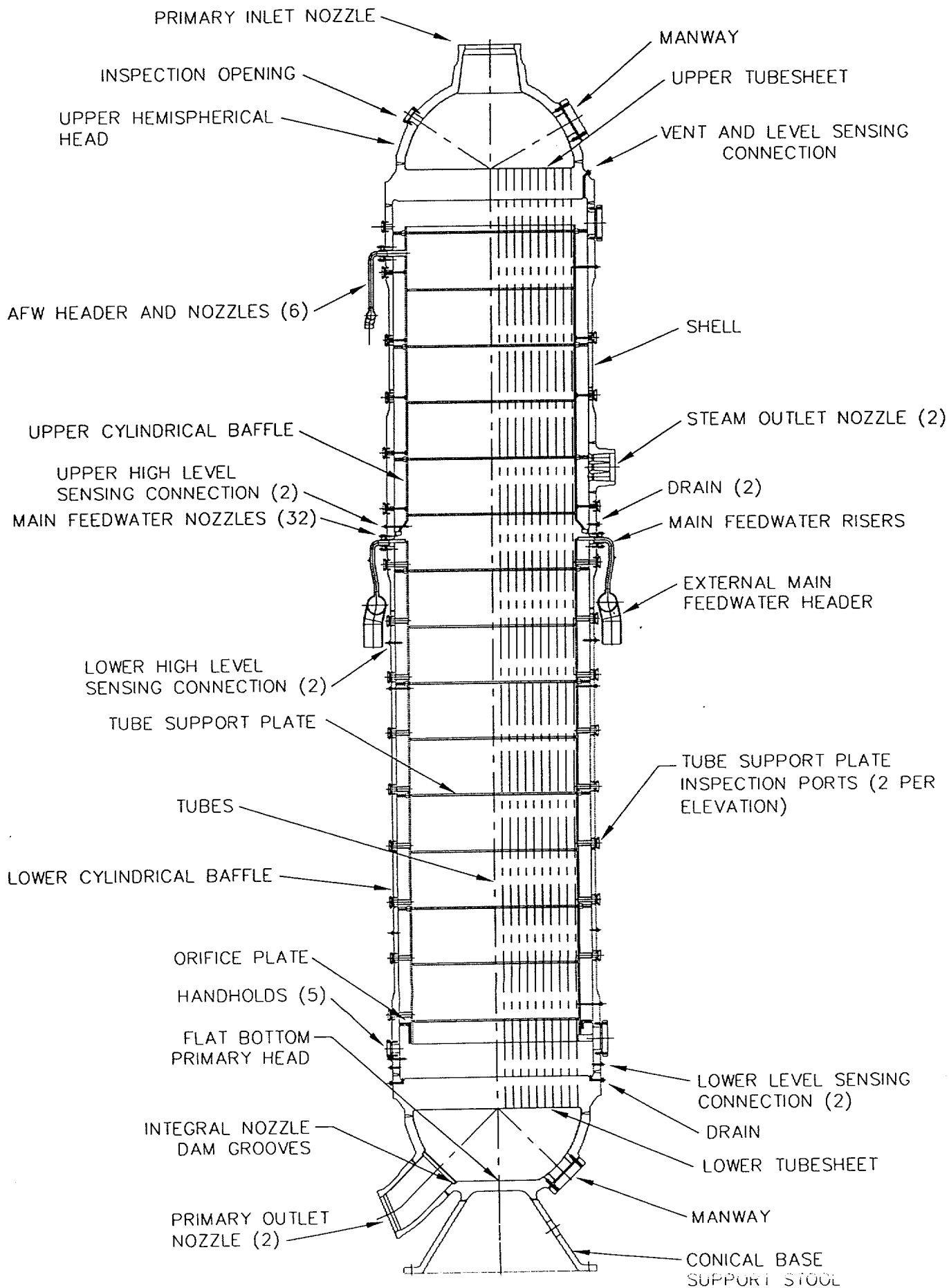
◆ Specific Improvements

- ❖ Erosion/corrosion resistant materials incorporated
- ❖ High strength steel forgings for shell, tubesheets and primary heads
- ❖ Alloy 690TT tubes
- ❖ Inconel main feedwater nozzles
- ❖ SS broached support plates
- ❖ Open tube lane tubed
- ❖ Hydraulically expanded tubes
- ❖ Steam outlet flow restrictor added *different from C.C.*
- ❖ Feedwater / Auxiliary Feedwater connections seal welded
- ❖ Increased secondary side design pressure
- ❖ Conical support pedestal
- ❖ Predetermined and controlled application of tube pre-stress
- ❖ Flat bottom, self-draining lower primary head

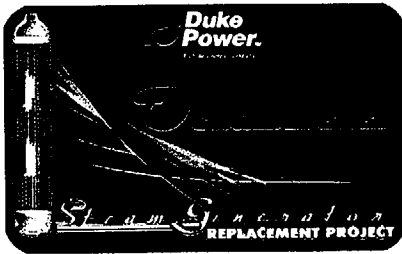


OTSG/ROTSG Comparison (Preliminary)

	<u>OTSG</u>	<u>ROTSG</u>
Thermal Output (MWt)	1292	1292
Steam Outlet Pressure (psia)	925	925
Primary Flow (lbm/hr x E6)	65.66	65.65
Thot/Tcold/Tavg (nominal)(deg F)	604/554/579	604/554/579
Number of Tubes	15,531	15,625
Surface Area (sqft)	132,966	134,546
Thermal Conductivity (BTU/hr-ft ² -degF @ 550 deg F)	10.83	9.85
Superheat (deg F)(BOL)	59.7	62.7
Operating Weight	637	551
Primary Side Volume (cuft)	2030	2007

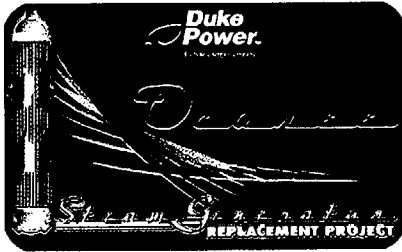


ROTSG LONGITUDINAL SECTION



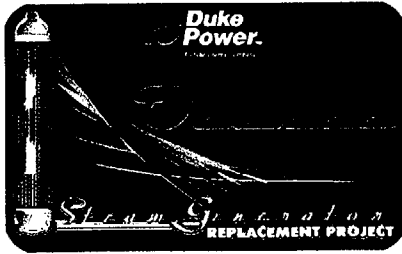
Engineering

- ◆ **Design Requirement Supported by Commercial Penalties**
 - ❖ No adverse change to unit operation or transient response caused by ROTSGs
- ◆ **No Significant Piping Modifications**
- ◆ **Existing Reflective Metal Insulation will be Replaced with New**



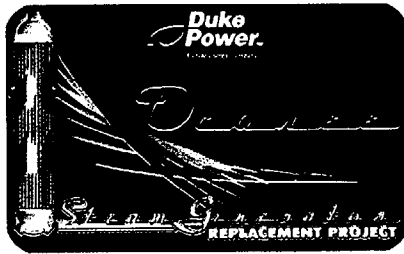
Reconstitution of NSSS Structural Design Basis

- ◆ Reanalysis by Framatome ANP
- ◆ Inputs Will Reflect Current Licensing Basis
- ◆ LBB Will Be Used as Input to Structural Design

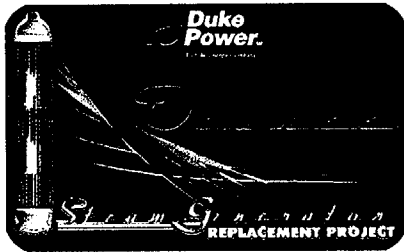


Steam Generator Disposal

- ◆ **Evaluated Off-site Disposal Vs. On-site Storage**
- ◆ **Current Plan Is For Original Steam Generators To Be Stored on Site**
- ◆ **Following Construction, Storage Facility Will Be Used As Prep-facility For Replacement Steam Generators**



Steam Generator Removal and Replacement



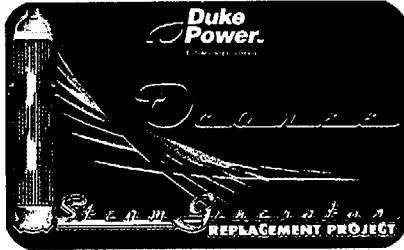
Installation Contractor

◆ Steam Generating Team (SGT)

- ❖ Joint Partnership of Washington Group and Duke Engineering & Services
- ❖ Washington Group, Duke, SGT Steam Generator Replacement Project Experience
 - Point Beach Units 1 & 2
 - DC Cook Unit 2
 - Catawba Unit 1
 - McGuire Units 1 & 2
 - St. Lucie Unit 1
 - Indian Point Unit 2
 - Calvert Cliffs Units 1 & 2
 - Prairie Island Unit 1
 - Callaway

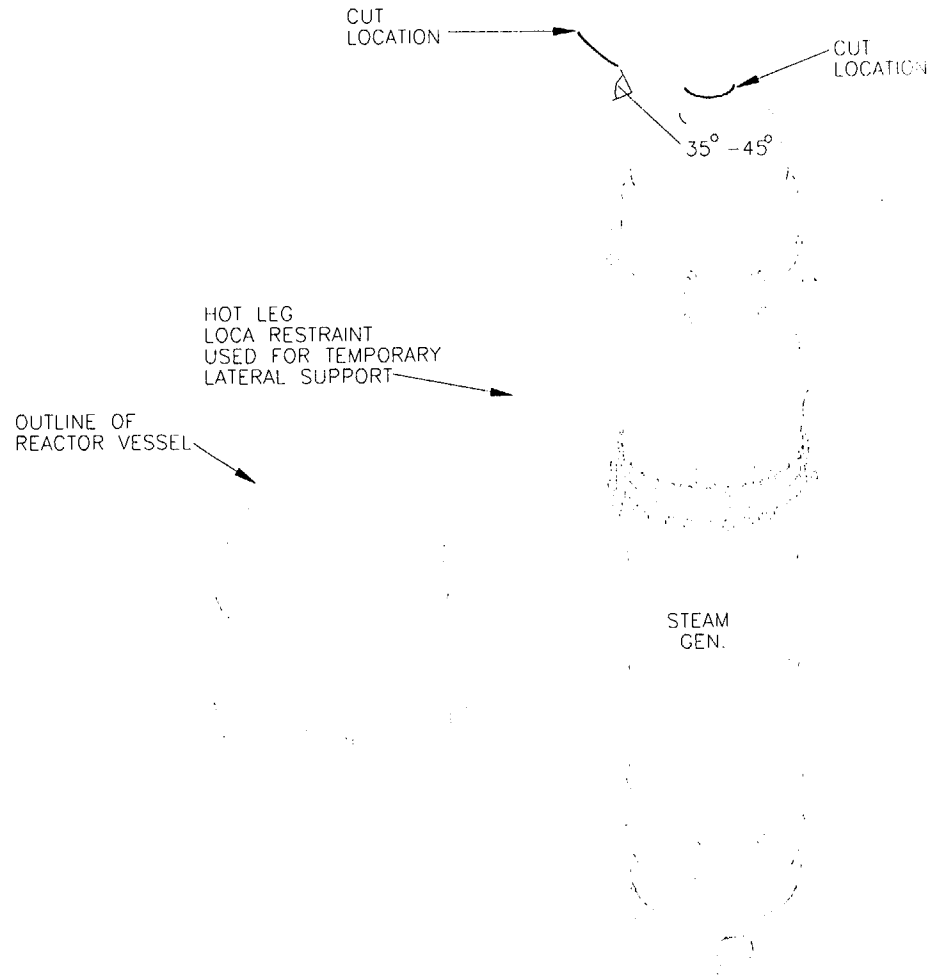
The Steam Generating Team
SGT Ltd.

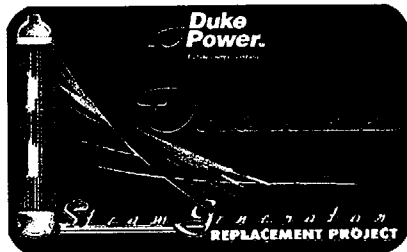
*A Washington Group International, Inc. /
Duke Engineering & Services, Inc. Company*



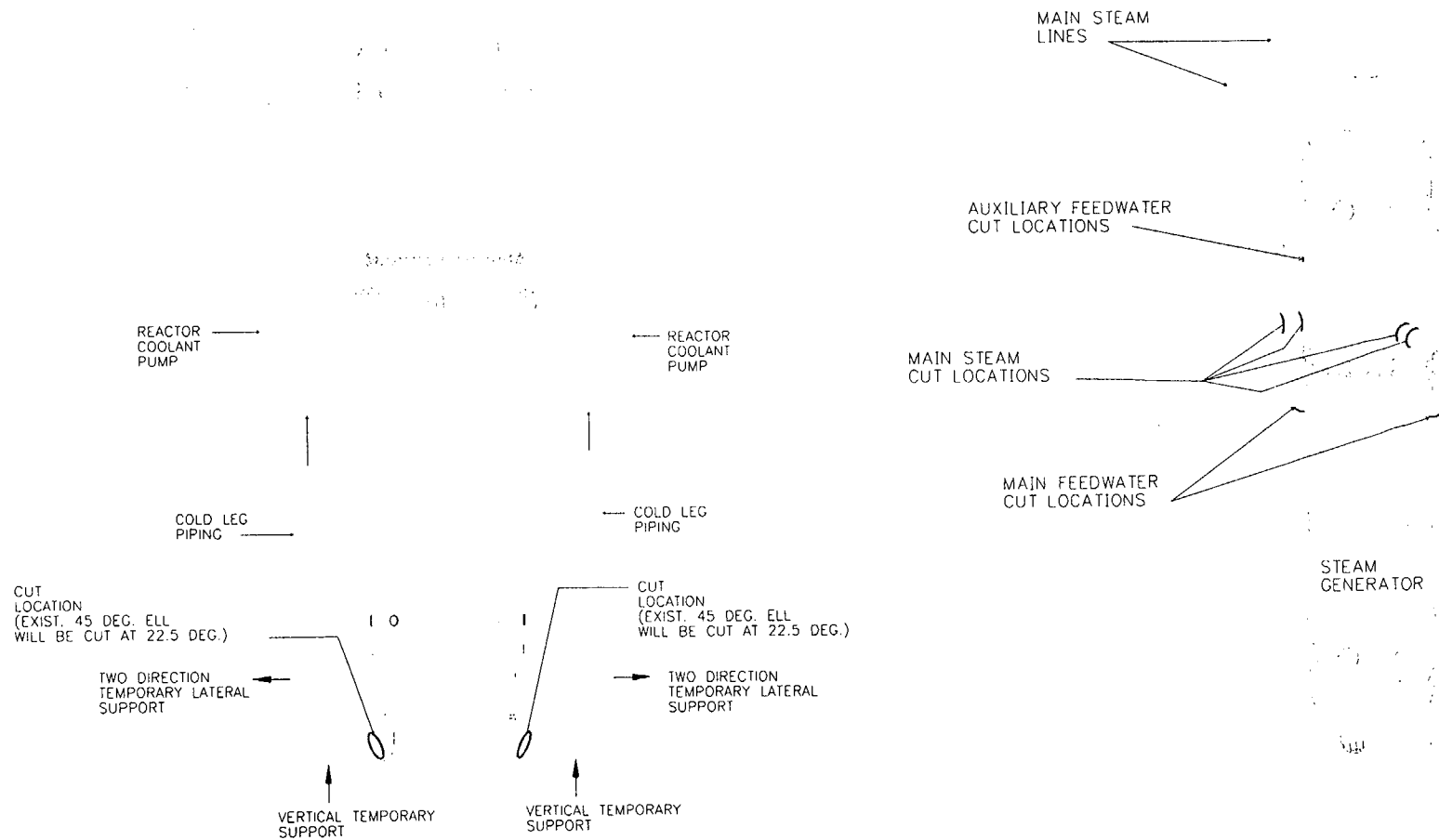
Steam Generator Removal/Replacement

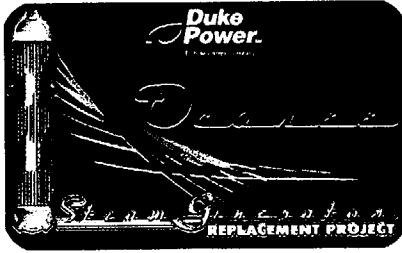
- ◆ **Steam Generator Cut Locations Optimized to Facilitate Steam Generator Removal and Installation of Replacement Steam Generators**





Cut Location Figure





Removal of Steam Generators From Containment

Two Options Considered:

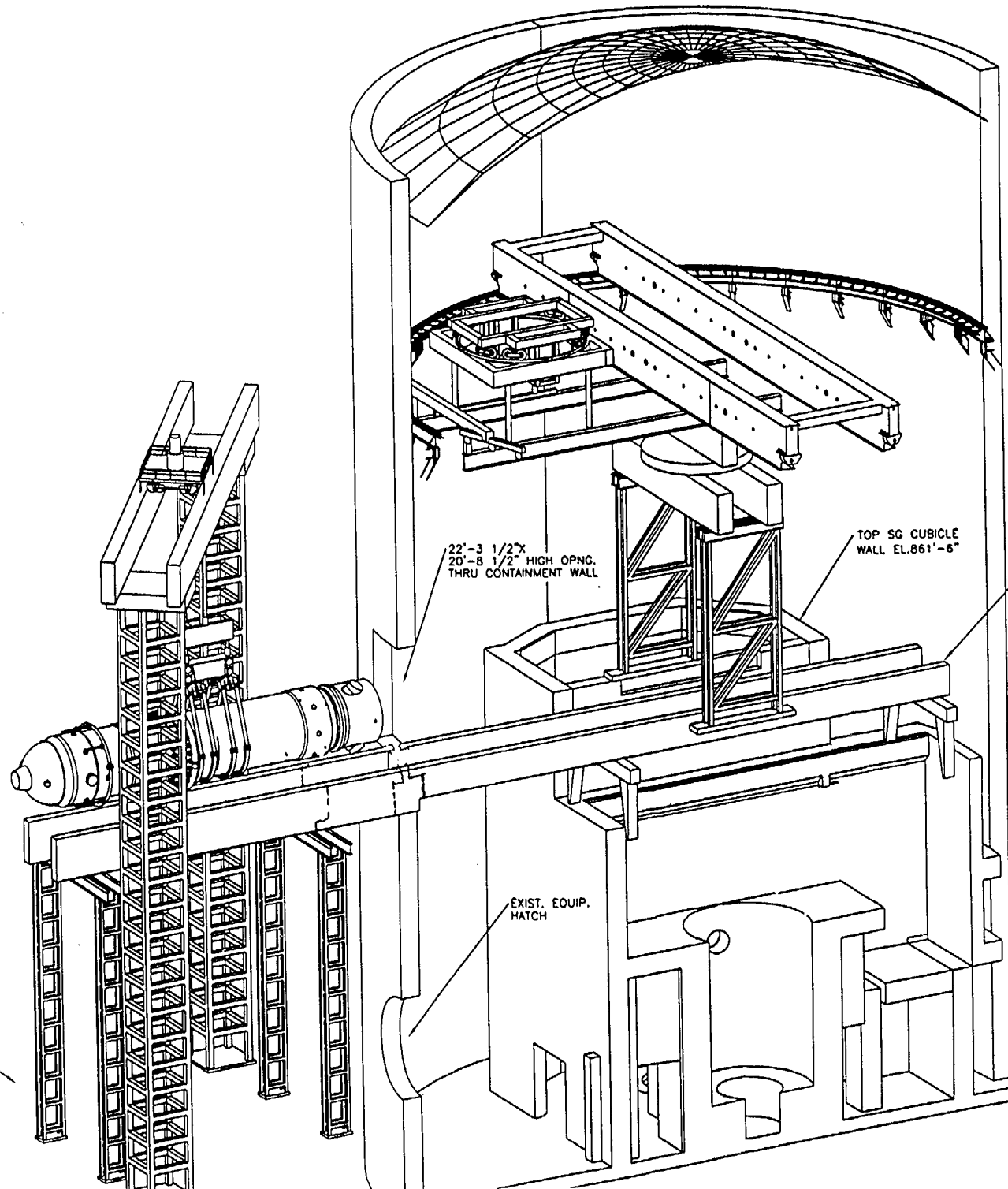
◆ Existing Equipment Hatch

- ❖ Requires extensive removal/replacement of structures inside Containment and equipment including a portion of the fuel transfer canal and a section of CCW piping outside Containment

◆ Containment Wall Opening

- ❖ Involves cutting 20 X 25 foot opening in Containment
- ❖ Successfully completed at other plants
- ❖ Less critical path time

GRADE EL.796'-0"



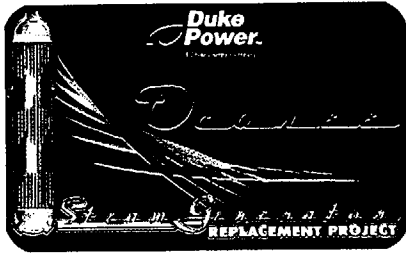
22'-3 1/2"x
20'-8 1/2" HIGH OPNG.
THRU CONTAINMENT WALL

TOP SG CUBICLE
WALL EL.861'-6"

TOP GIRDERS
EL.860'-9 1/2"

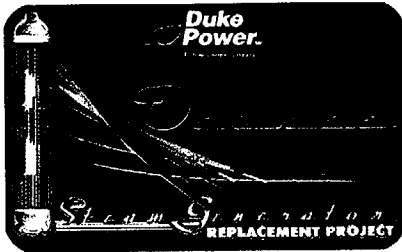
EXIST. EQUIP.
HATCH

EASEMENT FLOOR
EL.777'-1 1/2"

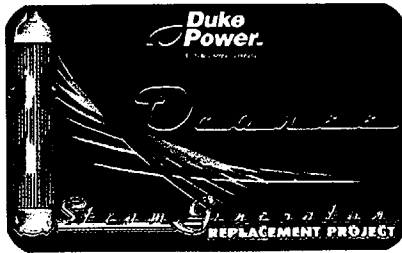


Licensing

Licensing Approach



- ◆ **Steam Generators Are a “Like-for-Like” Replacement**
- ◆ **Minimal Impact on Containment (Ch 6) or Accident (Ch 15) Analysis**
- ◆ **Majority of Replacement Activities Accomplished Under 50.59 and 50.65**
- ◆ **License Amendment Needed to “Clean Up” Steam Generator Surveillance Tech Spec and Revise Topical Report References**
 - ❖ **Delete references to sleeving, re-rolled tubes and open tube lane**



Other Licensing Actions

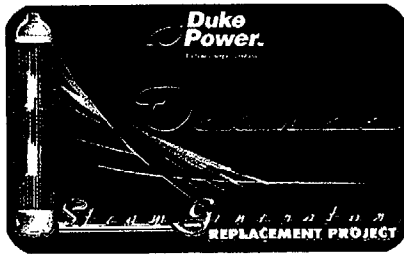
◆ Other NRC Approvals / License Amendments

- ❖ ASME Code Cases 2142-1, 2143-2, and N-20-4 (Complete)
- ❖ Loop Analysis Methodology (In Progress)
- ❖ Duke Topical Report Revisions

◆ Large Break LOCA Thermal Transient

- ❖ Pursuing resolution for ROTSGs through B&W Owners Group
- ❖ Preliminary analysis has demonstrated that the tubes will not fail

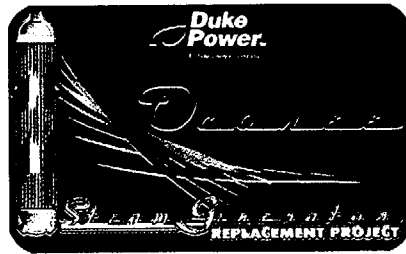
Chapter 16



Modifications/Temporary Alterations

- ◆ Steam Generator Supports
- ◆ Steam Generator Structural Interference
- ◆ RCS Piping Supports#
- ◆ Mainsteam piping
- ◆ Feedwater Piping
- ◆ Emergency Feedwater Piping
- ◆ Scaffolding/Shielding
- ◆ Nitrogen Injection System#
- ◆ Temporary Services
- ◆ Steam Generator Replacement
- ◆ Blowdown#
- ◆ Auxiliary Cranes
- ◆ Chemical Addition#
- ◆ Vent Line#
- ◆ Insulation
- ◆ RCS Instrumentation Lines#
- ◆ Secondary Instrumentation Lines
- ◆ Facilities
- ◆ Original Steam Generator Retirement Facility#
- ◆ Containment Opening
- ◆ Rigging & Handling
- ◆ Steam Generator Transport
- ◆ Reactor Cavity Decking
- ◆ Prior Outage Supports#
- ◆ Temporary Electrical Power
- ◆ Nitrogen Injection#

- Modifications



Licensing Schedule

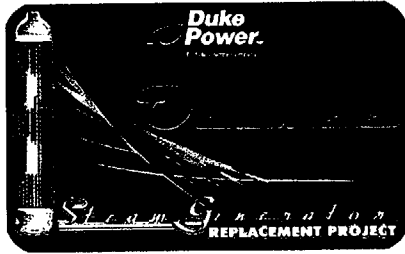
◆ Code Case Approval Requests

- ❖ 2142-1 and 2143-1 Approved 9/10/99
- ❖ N-20-4 Approved 12/22/99

◆ License Amendment

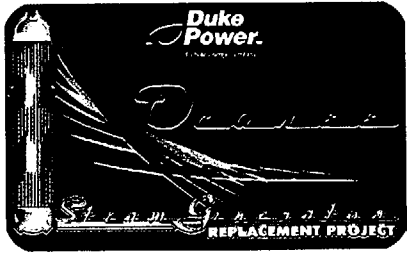
- ❖ Anticipated submittal - August 2002
- ❖ Requested approval - August 2003 (first replacement outage starts Sept 2003)
- ❖ NRC/NEI proposal to revise steam generator surveillance Technical Specifications would eliminate need for SGRP specific license amendment except for Topical Report references

NEI 97-06

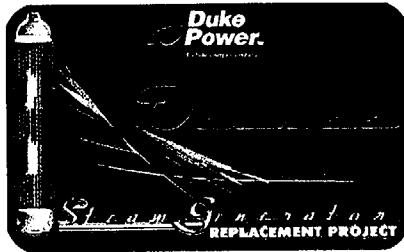


NRC Interface

- ◆ **Project Status Updates to NRR and/or Region on Periodic Basis**
- ◆ **Tour of BWC Shop (Cambridge, Ontario) During Steam Generator Fabrication**
- ◆ **Weekly Meetings With Region / Resident Inspectors During Replacement Outage**



Closing



Summary

◆ Experienced Project Team (Duke/SGT/BWC)



◆ Goal - Continued Safe and Economical Operation of Oconee