

Mr. George J. Beck
Manager-Licensing, MC 52A-5
Philadelphia Electric Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box No. 195
Wayne, Pennsylvania 19087-0195

Dear Mr. Beck:

SUBJECT: SNUBBER VISUAL INSPECTION INTERVAL REQUIREMENTS, GENERIC LETTER
90-09, PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3
(TAC NOS. M84178 AND M84179)

The Commission has issued the enclosed Amendments Nos. 171 and 175 to Facility Operating License Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Unit Nos. 2 and 3. These amendments consist of changes to the Technical Specifications in response to your application dated August 5, 1992 and supplemented by letter dated September 11, 1992.

These amendments revise Section 4.11.D of the Peach Bottom Atomic Power Station technical specifications regarding the visual inspection of snubbers. The changes revise the method of determining inspection intervals for snubbers in accordance with the guidance in NRC Generic Letter 90-09, "Alternative Requirements for Snubber Visual Inspection Intervals and Corrective Actions."

Please notify the staff, in writing, when you have fully implemented these amendments.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

/s/

Joseph W. Shea, Acting Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

9209290360 920921
PDR ADOCK 05000277
P PDR

Enclosures:

- 1. Amendment No. 171 to DPR-44
- 2. Amendment No. 175 to DPR-56
- 3. Safety Evaluation

cc w/enclosures:

See next page

DISTRIBUTION:

| | | | |
|------------------|-------------------|-------------------|----------------|
| Docket File | MO'Brien(2) | CGrimes, 11E21 | BRuland, RGN-I |
| NRC & Local PDRs | JShea/RClark | JRajan | |
| PDI-2 Reading | OGC | ACRS(10) | |
| SVarga | DHagan, 3206 | OPA | |
| JCalvo | GHill(8), P1-22 | OC/LFMB | |
| CMiller | Wanda Jones, 7103 | EWenzinger, RGN-I | |

*Previously Concurred

| | | | | | |
|------|------------|------------|------------|-----------|---|
| OFC | : PDI-2/LA | : PDI-2/PM | : OGC* | : PDI-2/D | : |
| NAME | : MO'Brien | : JShea | : | : CMiller | : |
| DATE | : 9/11/92 | : 9/16/92 | : 08/19/92 | : 9/17/92 | : |

MAILED 10:00 AM SEP 17 1992

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

September 21, 1992

Docket Nos. 50-277
and 50-278

Mr. George J. Beck
Manager-Licensing, MC 52A-5
Philadelphia Electric Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box No. 195
Wayne, Pennsylvania 19087-0195

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Sincerely,

A handwritten signature in black ink, appearing to read "Joseph W. Shea".

Joseph W. Shea, Acting Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 171 to DPR-44
2. Amendment No. 175 to DPR-56
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. George J. Beck
Philadelphia Electric Company

Peach Bottom Atomic Power Station,
Units 2 and 3

cc:

J. W. Durham, Sr., Esquire
Sr. V.P. & General Counsel
Philadelphia Electric Company
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Bureau of Radiation Protection
Pennsylvania Department of
Environmental Resources
P. O. Box 2063
Harrisburg, Pennsylvania 17120

Philadelphia Electric Company
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Peach Bottom Atomic Power Station
Route 1, Box 208
Delta, Pennsylvania 17314

Board of Supervisors
Peach Bottom Township
R. D. #1
Delta, Pennsylvania 17314

Philadelphia Electric Company
ATTN: Regulatory Engineer, A1-2S
Peach Bottom Atomic Power Station
Route 1, Box 208
Delta, Pennsylvania 17314

Public Service Commission of Maryland
Engineering Division
ATTN: Chief Engineer
231 E. Baltimore Street
Baltimore, MD 21202-3486

Resident Inspector
U.S. Nuclear Regulatory Commission
Peach Bottom Atomic Power Station
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Delta, Pennsylvania 17314

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Review Division
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Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
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King of Prussia, Pennsylvania 19406

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Baltimore, Maryland 21201

Carl D. Schaefer
External Operations - Nuclear
Delmarva Power & Light Company
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Wilmington, DE 19899



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 171
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et. al. (the licensee) dated August 5, 1992, and supplemented by letter dated September 11, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

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P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 171, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Charles L. Miller

Charles L. Miller, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 21, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 171

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

| <u>Remove</u> | <u>Insert</u> |
|---------------|---------------|
| vii | vii |
| 234a | 234a |
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| - | 234t |
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| 3.15 | Seismic Monitoring Instrumentation | 240u |
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LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS

3.11.D

Shock Suppressors (Snubbers)
on Safety Related Systems

3.11.D.1

During all modes of operation all snubbers on safety-related systems shall be operable except as noted in 3.11.D.2 and 3.11.D.3 below. Snubbers on non-safety related systems are excluded from this requirement if their failure or failure of the system on which installed has no adverse effect on a safety-related system.

3.11.D.2

During operation in the cold shutdown or refueling modes, snubbers located on systems required to be operable shall be operable except as noted in 3.11.D.3.

3.11.D.3

With one or more snubbers inoperable under the requirements of 3.11.D.1, within 72 hours, replace or restore the inoperable snubber to the operable status and perform an engineering evaluation per specification 4.11.D.6. If these requirements cannot be met, declare the supported system inoperable and follow the applicable Limiting Condition for Operation for that System.

4.11.D

Shock Suppressors (Snubbers)
on Safety Related Systems

4.11.D.1

Snubbers required to be operable under the provisions of 3.11.D.1 shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.6.6.

4.11.D.2

Snubbers required to be operable under the provisions of 3.11.D.1 shall be visually inspected according to the schedule determined by Table 4.11.D-1. The visual inspection interval for each category shall be determined based upon criteria provided in Table 4.11.D-1 and the first inspection interval determined by using this criteria shall be based upon the previous inspection interval as established by the requirements in effect before amendment (NRC will include the number of license amendment that implements this change).

Snubbers may be categorized in two groups, "accessible" or "inaccessible", based on their accessibility for inspection during reactor operation. These two groups may be inspected independently according to Table 4.11.D-1.

4.11.D.3

Visual inspection of snubbers required to be operable under the provisions of 3.11.D.1 shall verify that 1) there are no indications of damage or impaired operability, 2) attachments to the foundations or supporting structure are functional, and 3) fasteners for the attachment of the snubber to the component and to the snubber anchorage are functional.

Snubbers which appear to be inoperable as a result of visual inspections shall be classified as unacceptable and may be reclassified acceptable for the purpose of establishing the next visual inspection interval, providing that 1) the cause of the rejection is clearly established and remedied for that particular snubber and for other generically susceptible snubbers; and 2) the affected snubber is functionally tested in the as found condition and determined operable per Specification 4.11.D.7 or 4.11.D.8, as applicable. All snubbers found connected to an inoperable common hydraulic fluid reservoir shall be counted as unacceptable for determining the next inspection interval. A review and evaluation shall be performed and documented to justify continued operation with an unacceptable snubber. If continued operation cannot be justified, the snubber shall be declared inoperable and the Limiting Conditions for Operation shall be met.

4.11.D.4

Functional Test

a) Once each operating cycle, during shutdown, a representative sample of 10% of each type of (mechanical or hydraulic) snubber required to be operable under the provisions of 3.11.D.1 shall be functionally tested either in place or in a bench test. For every unit found to be inoperable an additional 10% of that type of snubber shall be functionally tested until no more failures are found or all snubbers of that type have been tested. The functional test requirements for mechanical

TABLE 4.11.D-1
SNUBBER VISUAL INSPECTION INTERVAL

| Population or Category (Notes 1 and 2) | NUMBER OF UNACCEPTABLE SNUBBERS | | |
|---|--|--|--|
| | Column A Extend Interval (Notes 3 and 6) | Column B Repeat Interval (Notes 4 and 6) | Column C Reduce Interval (Notes 5 and 6) |
| 1 | 0 | 0 | 1 |
| 80 | 0 | 0 | 2 |
| 100 | 0 | 1 | 4 |
| 150 | 0 | 3 | 8 |
| 200 | 2 | 5 | 13 |
| 300 | 5 | 12 | 25 |
| 400 | 8 | 18 | 36 |
| 500 | 12 | 24 | 48 |
| 750 | 20 | 40 | 78 |
| 1000 or greater | 29 | 56 | 109 |

Note 1: The next visual inspection interval for a snubber population or category size shall be determined based upon the previous inspection interval and the number of unacceptable snubbers found during that interval. Snubbers may be categorized, based upon their accessibility during power operation, as accessible or inaccessible. These categories may be examined separately or jointly. However, the licensee must make and document that decision before any inspection and shall use that decision as the basis upon which to determine the next inspection interval for that category.

Note 2: Interpolation between population or category sizes and the number of unacceptable snubbers is permissible. Use next lower integer for the value of the limit for Columns A, B, or C if that integer includes a fractional value of unacceptable snubbers as determined by interpolation.

Note 3: If the number of unacceptable snubbers is equal to or less than the number in Column A, the next inspection interval may be twice the previous interval but not greater than 48 months.

Note 4: If the number of unacceptable snubbers is equal to or less than the number in Column B but greater than the number in Column A, the next inspection interval shall be the same as the previous interval.

Note 5: If the number of unacceptable snubbers is equal to or greater than the number in Column C, the next inspection interval shall be two-thirds of the previous interval. However, if the number of unacceptable snubbers is less than the number in Column C but greater than the number in Column B, the next interval shall be reduced proportionally by interpolation, that is, the previous interval shall be reduced by a factor that is one-third of the ratio of the difference between the number of unacceptable snubbers found during the previous interval and the number in Column B to the difference in the numbers in Columns B and C.

Note 6: The provisions of Section 1.0 ("Definitions") are applicable for all inspection intervals up to and including 48 months.

PBAPS

4.11 BASESB. Emergency Heat Sink Facility

The testing of the ESW Booster Pumps and the ECW pump is in accordance with existing ASME codes and applicable addenda except where relief has been granted and assures the required availability of the equipment.

C. Emergency Shutdown-Control Panels

Once per week verification of the panels being properly secured is considered adequate. The associated equipment is proven operable during surveillance testing of that equipment. An operability verification by electrical test at each refueling outage is adequate to assure that the panels are available and can perform their design function.

D. Shock Suppressors (Snubbers) on Safety Related Systems

All safety related snubbers shall be, as a minimum, visually inspected to verify that (1) the snubber has no visible indications of damage or impaired operability, (2) attachments to the foundation or supporting structure are functional, (3) fasteners for the attachment of the snubber to the component and to the snubber anchorage are functional, and (4) proper hydraulic fluid level for hydraulic snubbers. Snubbers are categorized into two groups, "accessible" or "inaccessible", based on their accessibility for inspection during reactor operation and drywell inertment. As discussed in Generic Letter 90-09, "Alternative Requirements for Snubber Visual Inspection Intervals and Corrective Actions", the method for determining the next interval for the visual inspection of snubbers is provided based upon the number of unacceptable snubbers found during the previous inspection, the total population or category size and the previous inspection interval. A snubber is considered unacceptable if it fails to satisfy the acceptance criteria of the visual inspection. Snubbers may be categorized, based upon their accessibility during power operation, as accessible or inaccessible. These categories may be examined separately or jointly. Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection. However, the results of such early inspections will only be used to shorten the required interval and not to lengthen it.

When a snubber is found inoperable an engineering evaluation is performed to determine (a) snubber mode of failure and, (b) if there is any adverse effect or degradation on the supported piping or equipment due to the failure.

To further increase the assurance of snubber reliability, functional tests will be performed once each operating cycle.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 174
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et. al. (the licensee) dated August 5, 1992, and supplemented by letter dated September 11, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 175, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Charles L. Miller

Charles L. Miller, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 21, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 175

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

| <u>Remove</u> | <u>Insert</u> |
|---------------|---------------|
| vii | vii |
| 234a | 234a |
| 234b | 234b |
| - | 234t |
| - | 234u |
| 236a | 236a |

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LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS

3.11.D

Shock Suppressors (Snubbers)
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During all modes of operation all snubbers on safety-related systems shall be operable except as noted in 3.11.D.2 and 3.11.D.3 below. Snubbers on non-safety related systems are excluded from this requirement if their failure or failure of the system on which installed has no adverse effect on a safety-related system.

3.11.D.2

During operation in the cold shutdown or refueling modes, snubbers located on systems required to be operable shall be operable except as noted in 3.11.D.3.

3.11.D.3

With one or more snubbers inoperable under the requirements of 3.11.D.1, within 72 hours, replace or restore the inoperable snubber to the operable status and perform an engineering evaluation per specification 4.11.D.6. If these requirements cannot be met, declare the supported system inoperable and follow the applicable Limiting Condition for Operation for that System.

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Shock Suppressors (Snubbers)
on Safety Related Systems

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Snubbers required to be operable under the provisions of 3.11.D.1 shall be visually inspected according to the schedule determined by Table 4.11.D-1. The visual inspection interval for each category shall be determined based upon criteria provided in Table 4.11.D-1 and the first inspection interval determined by using this criteria shall be based upon the previous inspection interval as established by the requirements in effect before amendment (NRC will include the number of license amendment that implements this change).

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Note 1: The next visual inspection interval for a snubber population or category size shall be determined based upon the previous inspection interval and the number of unacceptable snubbers found during that interval. Snubbers may be categorized, based upon their accessibility during power operation, as accessible or inaccessible. These categories may be examined separately or jointly. However, the licensee must make and document that decision before any inspection and shall use that decision as the basis upon which to determine the next inspection interval for that category.

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Note 5: If the number of unacceptable snubbers is equal to or greater than the number in Column C, the next inspection interval shall be two-thirds of the previous interval. However, if the number of unacceptable snubbers is less than the number in Column C but greater than the number in Column B, the next interval shall be reduced proportionally by interpolation, that is, the previous interval shall be reduced by a factor that is one-third of the ratio of the difference between the number of unacceptable snubbers found during the previous interval and the number in Column B to the difference in the numbers in Columns B and C.

Note 6: The provisions of Section 1.0 ("Definitions") are applicable for all inspection intervals up to and including 48 months.

4.11 BASES

B. Emergency Heat Sink Facility

The testing of the ESW Booster Pumps and the ECW pump is in accordance with existing ASME codes and applicable addenda except where relief has been granted and assures the required availability of the equipment.

C. Emergency Shutdown-Control Panels

Once per week verification of the panels being properly secured is considered adequate. The associated equipment is proven operable during surveillance testing of that equipment. An operability verification by electrical test at each refueling outage is adequate to assure that the panels are available and can perform their design function.

D. Shock Suppressors (Snubbers) on Safety Related Systems

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When a snubber is found inoperable an engineering evaluation is performed to determine (a) snubber mode of failure and, (b) if there is any adverse effect or degradation on the supported piping or equipment due to the failure.

To further increase the assurance of snubber reliability, functional tests will be performed once each operating cycle.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 171 AND 175 TO FACILITY OPERATING

LICENSE NOS. DPR-44 and DPR-56

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3

DOCKET NOS. 50-277 AND 50-278

1.0 INTRODUCTION

By letter dated August 5, 1992, and supplemented by letter dated September 11, 1992, the Philadelphia Electric Company (PECo or the licensee) submitted a request for changes to the Peach Bottom Atomic Power Station (PBAPS), Units 2 and 3, Technical Specifications (TS). The requested changes would revise TS Section 4.11.D and the associated Bases to incorporate the recommendations on snubber visual inspection frequencies contained in Generic Letter (GL) 90-09, "Alternative Requirements for Snubber Visual Inspection Intervals and Corrective Actions" dated December 11, 1990. Specifically, the revisions would remove the snubber visual examination schedule in the existing TS and replace it with a refueling outage based visual examination schedule as listed in Table 1 of GL 90-09. The information provided in the supplemental letter was not outside the scope of the original Federal Register notice and did not change the initial no significant hazards consideration determination.

2.0 EVALUATION

The snubber visual examination schedule in the existing Technical Specifications is based on the permissible number of inoperable snubbers found during the visual examination. Because the existing snubber visual examination schedule is based only on the absolute number of inoperable snubbers found during the licensee's visual examination regardless of the total population of snubbers, licensees with a large snubber population find the visual examination schedule excessively restrictive. The purpose of the alternative visual examination schedule is to allow the licensee to perform visual examinations and corrective actions during plant outages without reduction of the confidence level provided by the existing visual examination schedule. The new visual examination schedule specifies the permissible number of inoperable snubbers for various snubber populations. The basic examination interval is the normal fuel cycle up to 24 months. This interval may be extended to as long as twice the fuel cycle depending on the number of

acceptable snubbers found during the visual examination. The examination interval may vary by ± 25 percent to coincide with the actual outage.

In the event one or more snubbers are found inoperable during a visual examination, the Limiting Condition for Operation (LCO) in the present TS require the licensee to restore or replace the inoperable snubber(s) to operable status within 72 hours or declare the attached system inoperable and follow the appropriate action statement for that system. This LCO will remain in the TS; however, the permissible number of inoperable snubber(s) and the subsequent visual inspection interval will now be determined in accordance with the visual examination schedule (Table 1 of Generic Letter 90-09, dated December 11, 1990). As noted in the guidance for this line item TS improvement, certain corrective actions may have to be performed depending on the number of inoperable snubbers found. All requirements, for corrective actions and evaluations associated with the use of the visual examination schedule and stated in the footnotes 1 through 7 (Table 1 of Generic Letter 90-09), shall be included in the TS.

The licensee has provided changes to Specification 4.11.D that are consistent with the guidance provided in Generic Letter 90-09 for the replacement of the snubber visual examination schedule with Table 1 (including footnotes 1 through 7) of Generic Letter 90-09. On the basis of its review of this matter, the staff finds that the proposed changes to the TS for Peach Bottom Atomic Power Station, Units 2 and 3 are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (57 FR 37570). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Rajan

Date: September 21, 1992