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Vice President

May 16, 2001

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Subject: Oconee Nuclear Site Docket No. 50-269
Core Operating Limits Report (COLR)

Gentlemen:

Attached, pursuant to Oconee Technical Specifications 5.6.5, is an information copy of a revision to the Core Operating Limits Report for Oconee Unit 1, Cycle 20, Rev. 16.

Very truly yours,

W. R. McCollum, Site Vice President
Oconee Nuclear Site

Attachment

Ⓢ A001

NRC Document Control Desk
May 16, 2001
Page 2

xc w/att: Mr. L. A. Reyes, Regional Administrator
U. S. Nuclear Regulatory Commission, Region II

Mr. D. E. LaBarge, Project Manager
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Page 1 of 1

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Duke Power Company

Oconee 1 Cycle 20

Core Operating Limits Report

REVIEWED AND APPROVED BY CFAM 3.13

QA Condition 1

FOR INFORMATION ONLY

~~Not Reviewed or Approved by CFAM 3.13~~

REVIEWED AND APPROVED BY CFAM 3.13

Prepared By: David W. Harris

Date: May 2, 2001

Checked By: G. Michael Sennell

Date: May 3, 2001

CDR By: R. C. Harvey

Date: May 4, 2001

Approved By: R. R. St. Clair

Date: May 4, 2001

Oconee 1 Cycle 20
Core Operating Limits Report

Insertion Sheet for Revision 16

This revision is valid from May, 2001, until the end of operation for Oconee 1 Cycle 20.

Remove these revision 15 pages

Insert these revision 16 pages

1 - 4

1 - 4

Revision Log

Revision	Effective Date	Pages Revised	Pages Added	Pages Deleted	Total Effective Pages
Oconee 1 Cycle 20 revisions below					
16	May-01	1 - 4	-	-	31
15	Nov-00	1 - 31	-	31	31
Oconee 1 Cycle 19 revisions below					
14	Oct-00	1,2,3,5	-	-	31
13	Feb-00	1,2,3,4	-	-	31
12	Jul-99	1, 2, 3, 8, 10, 13, 31	-	-	31
11	May-99	1 - 31	-	1-31	31
Oconee 1 Cycle 18 revisions below					
10	Mar-99	1 - 31	-	32-38	31
9	Feb-98	1,2,3,5,13, 16,17,32,36	-	-	38
8	Nov-97	1,2,3,5,10, 32	37	-	38
7	Aug-97	1 - 38	-	-	38

Oconee 1 Cycle 20

The Core Operating Limits Report for O1C20 has been prepared in accordance with the requirements of ITS 5.6.5. The core operating limits within this report have been developed using NRC approved methodology identified in references 1, 2, 3, 4, 5, 6, 7, 8, 9 and 10. The RPS protective limits and maximum allowable setpoints are documented in references 11 and 12. These limits are validated for use in O1C20 by references 13, 14, 15, and 16. The O1C20 analyses assume a design flow of 107.5% of 88,000 gpm per RCS pump, radial local peaking ($F\Delta h$) of 1.714, and axial peaking factor (Fz) of 1.5.

The error adjusted core operating limits included in section 1 of the report incorporate all necessary uncertainties and margins required for operation of the O1C20 reload core.

1.1 References

1. Nuclear Design Methodology Using CASMO-3 / SIMULATE-3P, DPC-NE-1004A, Revision 0, SER dated November 23, 1992.
2. Oconee Nuclear Station Reload Design Methodology II, DPC-NE-1002A, Revision 1, SER dated October 1, 1985.
3. Oconee Nuclear Station Reload Design Methodology, NFS-1001A, Revision 4, SER dated July 29, 1981.
4. ONS Core Thermal Hydraulic Methodology Using VIPRE-01, DPC-NE-2003P-A, SER dated July 19, 1989.
5. Thermal Hydraulic Statistical Core Design Methodology, DPC-NE-2005P-A, Revision 2, SER dated June 8, 1999.
6. Fuel Mechanical Reload Analysis Methodology Using TACO3, DPC-NE-2008P-A, SER dated April 3, 1995.
7. UFSAR Chapter 15 Transient Analysis Methodology, DPC-NE-3005-PA, Revision 1, SER dated May 25, 1999.
8. DPC-NE-3000P-A, Thermal Hydraulic Transient Analysis Methodology, Rev. 2, SER dated October 14, 1998.
9. BAW-10192P-A, BWNT LOCA - BWNT Loss of Coolant Accident Evaluation Model for Once-Through Steam Generator Plants, SER dated February 18, 1997.
10. BAW-10227-PA, Evaluation of Advanced Cladding and Structural Material (M5) in PWR Reactor Fuel, SER dated February 4, 2000.
11. Variable Low Pressure Safety Limit, OSC-4048, Revision 3, July 1998.
12. Power Imbalance Safety Limits and Tech Spec Setpoints Using Error Adjusted Flux-Flow Ratio of 1.094, OSC-5604, Revision 1, November 1998.
13. O1C20 Maneuvering Analysis, OSC-7528, Revision 3, May 2001.
14. O1C20 Specific DNB Analysis, OSC-7536, Revision 1, May 2000.
15. O1C20 Reload Safety Evaluation, OSC-7537, Revision 2, May 2001.
16. ΔT_c and EOC Reduced Tav_g Operation, OSC-7265, Rev. 0, Duke Power Co., April 2001.

