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U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Mail Station O-P1-17  
Washington, D.C. 20555

SUBJECT: James A. FitzPatrick Nuclear Power Plant  
Docket No. 50-333  
**2001 Final Safety Analysis Report**

Dear Sir:

The update of the Final Safety Analysis Report (FSAR) for the James A. FitzPatrick Nuclear Power Plant is being submitted as required by 10 CFR 50.71(e). As discussed in NRC Regulatory Issue Summary 2001-05, this submittal will consist of a single CD-ROM. The CD-ROM is in pdf document format as requested in the Regulatory Issue Summary and includes the List of Effective Pages and replacement pages for all text, table and figure changes.

This update includes the designation of certain information as historical and the removal of certain redundant information pursuant to the guidance of NEI-98-03 Rev. 1 as endorsed by the NRC in Regulatory Guide 1.181. These changes are described in Attachment 1, Table 1.

This update includes changes to Chapter 17 "Quality Assurance Program" which are also described in Attachment 1, Table 2. These changes do not reduce the commitments in the QA Program description previously accepted by the NRC and demonstrate Entergy's continued compliance with Appendix B to 10 CFR 50. The revised Chapter 17 fulfills the requirement in 10 CFR 50.54(a) to submit an update of the Quality Assurance Program description.

This Update also includes changes to the FSAR that removed text and figures based on the process controls of AP-20.06. In accordance with that procedure those changes and their bases are summarized in Attachment 1, Table 3.

There are no commitments contained in this letter. If you have any questions, please contact Mr. Art Zaremba at (315) 349-6005.

Very Truly Yours,

Ted Sullivan  
Vice President, Operations

Cc: USNRC Regional Administrator  
USNRC Office of the Resident Inspector  
USNRC Project manager  
New York State Energy, Research, and Development Authority

Attachment: 2001 FSAR Changes

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**ATTACHMENT 1  
2001 FSAR Changes**

<b>Table 1: NEI-98-03 Rev. 1 2001 FSAR Changes</b>		
<b>FSAR Page / Table / Figure</b>	<b>Description of Change</b>	<b>Justification for Change</b>
P4.10-13, P4.10-14, P4.10-15, P7.3-5, P7.3-6, P7.3-19, P7.3-20, P7.11-2, F7.3-5, F7.3-7, F7.3-8, F7.3-9, F4.7-1	Delete redundant figures, revise references to those figures to provide reference to other figures where the information is available, correct descriptive text.	Deleted several simplified schematics and revised references to those schematics to indicate where the same information could be found in the FSAR. This change is consistent with the guidance, for removing redundant information from the FSAR, in NEI-98-03 Rev. 1 as endorsed by the NRC in Reg. Guide 1.181.
P7.1-43, P7.1-44, P7.1-45, P7.1-46, P7.1-47, P7.1-48, P7.1-49, P7.1-50, P8.ii, P8.vii, P8.3-7, F8.3-6, F8.3-7, F8.3-8	Reconstitute original AEC Q&A and move the associated text to a new section.	This information was added to the text in 1982, it was taken from answers to original Q&A. The response were fragmented during the 1982 update and certain supporting schematics were not incorporated into the Updated FSAR. As a result of this fragmentation and failure to keep the reference schematics in place it was difficult to understand the text. Reconstituting the information so that it can be viewed in its original context and designating this information as historical is consistent with the guidance in NEI-98-03 Rev.1 as endorsed by the NRC in Reg. Guide 1.181.
T7.1-2, T12.4-4	Restored T7.1-2 values to original FSAR values and designated T7.1-2 and T12.4-4 as historical information.	These tables were added to Chapters 7 and 12 in 1982. Originally they were identical in content were taken from original Q&A regarding the original Seismic Testing / Qualification. The response to the Q&A included a summary of information from GE. GE had performed testing to determine actual seismic failure points for various pieces of equipment. The values that they determined were never intended to be JAF Specific requirements. Restoration of this information to its original content and designation as historical is consistent with the guidance in NEI-98-03 Rev.1 as endorsed by the NRC in Reg. Guide 1.181.
P7.10-6, P10.8-2, P10.8-3, P10.8-4, P10.8-5	Revise section 7.10.3.6 to more accurately discuss the startup and operation of the feed and condensate systems and delete duplicate text in section 10.8.3	Deleted redundant text describing the startup and operation of the condensate and feedwater systems. The change provided reference to indicate where the information could be found in the FSAR. This change is consistent with the guidance, for removing redundant information from the FSAR, in NEI-98-03 Rev. 1 as endorsed by the NRC in Reg. Guide 1.181.
P7.10-8, P10.8-5	Identified text regarding leakage rates as historical information.	This information was added to the text in 1982, it was taken from answers to original Q&A. Designating this information as historical is consistent with the guidance in NEI-98-03 Rev.1 as endorsed by the NRC in Reg. Guide 1.181.
T12.4-5	Designated T12.4-5 as historical information.	This table was added to Chapter 12 in 1982. It was taken from original Q&A regarding the original Seismic Testing / Qualification for certain panels. Designating this information as historical is consistent with the guidance in NEI-98-03 Rev.1 as endorsed by the NRC in Reg. Guide 1.181.

**ATTACHMENT 1  
2001 FSAR Changes**

<b>Table 2: 2001 Chapter 17 QA Program Changes</b>		
<b>FSAR Page / Table / Figure</b>	<b>Description of Change</b>	<b>Justification for Change</b>
P17.2E-1, P17.2E-2	Change QA program description regarding audit frequencies to reflect the maturity of the JAF Program.	NRC acceptance of QA Program change proposed in correspondence JPN-98-049, acceptance provided on NRC letter dated 03/25/99 assigned Records Management System number NRPA-990325.
F17.2-1	Revised internal reporting relationships within the NYPA Organization.	Subject changes were evaluated in 50.59 evaluation JAF-SE-99-010, Revision 0.
P17.2-29	Revised address for submitting Part 21 Reports to be consistent with current NRC organization.	Editorial change to be consistent with the Code of Federal Regulations.
P17-v, P17.2-10, P17.2-17, F17.2-5	Delete cross-reference matrix of implementing documents.	Subject changes were evaluated in 50.59 evaluation JAF-SE-00-002, Revision 0.
P17.2-2, P17.2-3, P17.2-4, P17.2-5, P17.2-8, P17.2-10, P17.2-31, F17.2-1, F17.2-2, F17.2-3, F17.2-4	Revised titles and internal reporting relationships to reflect the updated NYPA Organization.	Subject changes were evaluated in 50.59 evaluation JAF-SE-00-005, Revision 0.
P17.2E-2	Adds a note to S17.2, Appendix E, S17.2E.2.1 to allow a 90 day grace period for completion of internal QA audits.	Subject changes were evaluated in 50.59 evaluation JAF-SE-00-042, Revision 0.
P17-i, P17-ii, P17-iii, P17-v, P17.2-1 – 2-32, P17.2A-1, P17.2B-3, -6, P17.2D-1, P17.2D-2, P17.2E-2, F17.2-1, F17.2-2, F17.2-3, F17.2-4	Revised titles and internal reporting relationships to reflect the organizational transition from The New York Power Authority to Entergy Nuclear Northeast.	Subject changes were evaluated in 50.59 evaluation JAF-SE-00-053, Revision 0.

**ATTACHMENT 1  
2001 FSAR Changes**

<b>Table 3: Other 2001 Changes</b>		
<b>FSAR Page / Table / Figure</b>	<b>Description of Change</b>	<b>Justification for Change</b>
Figures 3.2-3 and 3.2-4	Deleted Figures	These figures depicted fuel assemblies removed from the core during RFO-14
Figures 3.6-14 and 3.6-15	Deleted Figures	These figures illustrated SCRAM Reactivity and SCRAM Characteristics for the initial core. Due to text changes there were no longer references to the figures so they were removed.
Figure 3.7-2	Deleted Figure	Figures 3.7-1 and 3.7-2 presented essentially the same information (the core operating map), update of the FSAR to incorporate the ELLLA guidelines consolidated the information on Figure 3.7-1 and deleted Figure 3.7-2
Figure 4.2-2	Deleted Figure	This figure provided information regarding Charpy V Transition Temperature, that information was identified as being superseded by R.G. 1.99 Rev. 2 during the incorporation Tech Spec Amendment 258.
Pages 12.2A-1 through 12.2A-23 and Figures 12.2A-1 through 12.2A-60	Removed text and figures in their entirety.	Section 12.2A presented information regarding the SRFA for the James A. Fitzpatrick Nuclear Power Plant. The SRFA was added to the FSAR without adequate technical review. The response to DER-98-0340 determined that while the NRC had endorsed the SRFA methodology they had not reviewed the JAF Site specific SRFA for adequacy, and the JAF NSE that was used as the basis for the earlier FASR Change to incorporate the SRFA also failed to address the technical content of the JAF SRFA. Therefore, no basis existed for its inclusion in the FSAR and its removal was recommended.
Figure 13.2-2	Deleted Figure	This organization chart was deleted as part of the organization re-alignment after the transfer of the JAF Operating License the information was incorporated into Figure 13.2-1.
Figures 14.5-14 and 14.5-15	Deleted Figures	Figures presented information regarding Feedwater transients during fuel cycle 14; the information for these transients during cycle 15 is presented on figures 14.5-16, 14.5-17, and 14.5-18.
Figure 16.5-1 Sheet 3 of 3	Deleted Figure	Update to include the current P-T Curves in accordance with Tech Spec Amendment 258 resulted in consolidating information on 2 sheets.