UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

March 13, 1984

Dockets Nos. 50-277
and 50-278

Posted
Amdt. 93
to DPR-44

Mr. Edward G. Bauer, Jr.
Vice President and General Counsel
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

SUBJECT: TECHNICAL SPECIFICATION AMENDMENT PERTAINING TO SURVEILLANCE REQUIREMENTS FOR MODIFICATION TO THE SUPPRESSION POOL TEMPERATURE MONITORING SYSTEM

The Commission has issued the enclosed Amendments Nos. 93 and 95 to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated October 15, 1982.

The changes to the TSs provide for operability and surveillance requirements to reflect required modifications to the suppression pool temperature monitoring system as part of the Mark I Containment Long Term Program. Requested minor revisions to the TSs have also been provided by these amendments.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next Monthly Federal Register notice.

Sincerely,

Gerry Gears, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:
1. Amendment No. 93 to DPR-44
2. Amendment No. 95 to DPR-56
3. Safety Evaluation

cc w/enclosures:
See next page

Philadelphia Electric Company

cc w/enclosure(s):

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 93
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated October 15, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 93, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 13, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 93

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Remove	Insert
77	77
78	78
86	86
165	165
165a	165a
190	190

TABLE 3.2.F - SURVEILLANCE INSTRUMENTATION

Minimum No. of Operable Instrument Channels	Instrument	Type Indication and Range	Action***
2	Reactor Water Level	Recorder 0-60" Indicator 0-60"	(6) (7)
2	Reactor Pressure	Recorder 0-1500 psig Indicator 0-1200 psig	(1) (2) (3)
2	Drywell Pressure	Recorder 0-70 psig	(1) (2) (3)
2	Drywell Temperature	Recorder 0-400°F Indicator 0-400°F	(1) (2) (3)
2	Suppression Chamber Water Temperature*	Recorder 30-230°F Indicator 30-230°F	(1) (2) (3) (9)
2	Suppression Chamber Water Temperature**	Recorder 0-600°F Indicator 0-400°F	(1) (2) (3)
2	Suppression Chamber Water Level	Recorder 0-2 ft. Indicator 0-2 ft.	(1) (5)
1	Control Rod Position	28 Volt Indicating Lights	
1	Neutron Monitoring	SRM, IRM, LPRM 0-100%	(1) (2) (3) (4)
1	Safety-Relief Valve Position Indication	Acoustic or thermocouple	(8)

* Effective when modification associated with this amendment request is complete.
** Delete when modification associated with this amendment request is complete.
*** Notes for Table 3.2.F appear on page 78.

NOTES FOR TABLE 3.2.F

1) From and after the date that one of these parameters is reduced to one indication, continued operation is permissible during the succeeding thirty days unless such instrumentation is sooner made operable.

2) From and after the date that one of these parameters is not indicated in the control room, continued operation is permissible during the succeeding seven days unless such instrumentation is sooner made operable.

3) If the requirements of notes (1) and (2) cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold condition within 24 hours.

4) These surveillance instruments are considered to be redundant to each other.

5) In the event that all indications of this parameter are disabled and such indication cannot be restored in six (6) hours, an orderly shutdown shall be initiated and the reactor shall be in a Hot Shutdown condition in six (6) hours and a Cold Shutdown condition in the following eighteen (18) hours.

6) With the number of operable channels less than the minimum number of instrumentation channels shown in Table 3.2.F, either restore the inoperable channel to an operable status within 7 days, or be in at least hot shutdown within the next 12 hours.

7) If this parameter is not indicated in the control room, either restore at least one inoperable channel to operable status within 48 hours or be in at least hot shutdown within the next 12 hours.

8) If this parameter is not indicated in the control room, either restore at least one channel to operable status within thirty days or be in at least hot shutdown within the next 12 hours.

9) A suppression Chamber Water Temperature instrument channel will be considered operable if there are at least ten (10) resistance temperature detector inputs operable and no two (2)adjacent resistance temperature detector inputs are inoperable.

TABLE 4.2.F Unit 2

MINIMUM TEST AND CALIBRATION FREQUENCY FOR SURVEILLANCE INSTRUMENTATION

Instrument Channel	Calibration Frequency	Instrument Check
1) Reactor Level	Once/Operating cycle	Once Each Shift
2) Reactor Pressure	Once/6 months	Once Each Shift
3) Drywell Pressure	Once/6 months	Once Each Shift
4) Drywell Temperature	Once/6 months	Once Each Shift
5) Suppression Chamber Water Temperature	Once/operating cycle** Once/6 months***	Once Each Day** Once Each Shift***
6) Suppression Chamber Water Level	Once/6 months	Once Each Shift
7) Control Rod Position	NA	Once Each Shift
8) Neutron Monitoring (APRM)	Twice Per Week	Once Each Shift
9) Safety/Relief Valve Position Indicator (acoustics)	Once/Operating cycle	Once/Month
10) Safety/Relief Valve Position Indicator (thermocouple)	NA*	Once/month
11) Safety Valve Position Indicator (Acoustics)	Once/operating cycle	Once/month
12) Safety Valve Position Indicator (thermocouple)	NA*	Once/month

* Perform instrument functional check once per operating cycle
** Effective when modification associated with this amendment request is complete.
***Delete when modification associated with this amendment request is complete.

LIMITING CONDITIONS FOR OPERATION | SURVEILLANCE REQUIREMENTS

3.7 CONTAINMENT SYSTEMS

Applicability:

Applies to the operating status of the primary and secondary containment systems.

Objective:

To assure the integrity of the primary and secondary containment system.

Specification:

A. Primary Containment

1. Whenever the nuclear system is pressurized above atmospheric pressure or work is being done which has the potential to drain the vessel, the pressure suppression pool water volume and temperature shall be maintained within the following limits except as specified by 3.7.A.2, or when inoperability of the core spray systems, the LPCI and containment cooling subsystems is permissible as provided for in 3.5.F.3 and 3.5.F.4.b.

   a. Minimum water volume- 122,900 ft^3

   b. Maximum water volume- 127,300 ft^3

4.7 CONTAINMENT SYSTEMS

Applicability:

Applies to the primary and secondary containment integrity.

Objective:

To verify the integrity of the primary and secondary containment.

Specification:

1. The suppression chamber water level and temperature shall be checked once per day.

2. Whenever there is indication of relief valve operation (except when the reactor is being shutdown and torus cooling is being established) or testing which adds heat to the suppression pool, the pool temperature shall be continually monitored and also observed and logged every 5 minutes until the heat addition is terminated.

3. Whenever there is indication of relief valve operation with the local suppression pool temperature reaching 200°F or more, an external visual examination of the suppression chamber shall be conducted before resuming power operation.

4. A visual inspection of the suppression chamber interior, including water line regions shall be made at each major refueling outage.

LIMITING CONDITIONS FOR OPERATION | SURVEILLANCE REQUIREMENTS

3.7.A Primary Containment (Cont'd)

c. Maximum average suppression pool temperature limits:

(1) During startup/hot standby and run modes, with the suppression pool temperature greater than 95°F, except as permitted below, restore the temperature to less than 95°F within 24 hours or be in hot shutdown within the next 12 hours and cold shutdown within the following 24 hours.

(2) During testing which adds heat to the suppression pool, the pool temperature shall not exceed 105°F. Should the pool temperature exceed 105°F, such testing shall be stopped and the pool temperature must be reduced to below the limit specified in (1) above within 24 hours or be in hot shutdown within the next 12 hours and cold shutdown within the following 24 hours.

(3) The reactor shall be scrammed from any operating condition if the pool temperature reaches 110°F. Power operation shall not be resumed until the pool temperature is reduced below the limit specified in (1) above.

(4) During reactor isolation conditions, the reactor pressure vessel shall be depressurized to less than 200 psig at normal cooldown rates if the pool temperature reaches 120°F.

3.7.A & 4.7.A BASES (Cont'd)

The maximum allowable volume assures the integrity and functional capability of the Suppression Chamber (torus) during postulated LOCA pool swell effects on the torus support system. The majority of the Bodega tests were run with a submerged length of 4 feet and with complete condensation. Thus, with respect to downcomer submergence, this specification is adequate. The maximum temperature at the end of blowdown tested during the Humboldt Bay and Bodega Bay tests was 170°F and this is conservatively taken to be the limit for complete condensation of the reactor coolant, although condensation would occur for temperatures above 170°F.

Should it be necessary to drain the suppression chamber, this should only be done when there is no requirement for core standby cooling systems operability as explained in basis 3.5.F.

Experimental data indicates that excessive steam condensing loads can be avoided if the peak temperature of the suppression pool water at the quencher discharge is maintained below 200°F during any period of relief valve operation discharging through tee quenchers.

Because of the large volume and thermal capacity of the suppression pool, the volume and temperature changes very slowly and monitoring these parameters daily is sufficient to establish any temperature trends. By requiring the suppression pool temperature to be continually monitored and frequently logged during periods of testing which add significant heat, the temperature trends will be closely followed so that appropriate action can be taken if required. Logging is not required during inadvertent relief valve operation since during such periods operator action is actively and directly involved in operations relating to controlling torus temperature and monitoring of temperature trends is a natural part of the operations. Additionally, torus temperature is monitored by a recorder during these periods so that an historical record is available.

Operating procedures define the action to be taken in the event a relief valve inadvertently opens or sticks open. As a minimum this action shall include: (1) use of all available means to close the valve, (2) initiate suppression pool water cooling heat exchangers, (3) initiate reactor shutdown, and (4) if other relief valves are used to depressurize the reactor, their discharge shall be separated from that of the stuck-open relief valve to assure mixing and uniformity of energy insertion to the pool.

The requirement for an external visual examination following any event where potentially high loadings could occur provides assurance that no significant damage was encountered. Particular attention should be focused on structural discontinuities in the vicinity of the relief valve discharge since these are expected to be the points of highest stress.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 95
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated October 15, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 95 , are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 13, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 95

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Remove	Insert
77	77
78	78
86	86
165	165
165a	165a
190	190

TABLE 3.2.F - SURVEILLANCE INSTRUMENTATION

Minimum No. of Operable Instrument Channels	Instrument	Type Indication and Range	Action***
2	Reactor Water Level	Recorder 0-60" Indicator 0-60"	(6) (7)
2	Reactor Pressure	Recorder 0-1500 psig Indicator 0-1200 psig	(1) (2) (3)
2	Drywell Pressure	Recorder 0-70 psig	(1) (2) (3)
2	Drywell Temperature	Recorder 0-400°F Indicator 0-400°F	(1) (2) (3)
2	Suppression Chamber Water Temperature*	Recorder 30-230°F Indicator 30-230°F	(1) (2) (3) (9)
2	Suppression Chamber Water Temperature**	Recorder 0-600°F Indicator 0-400°F	(1) (2) (3)
2	Suppression Chamber Water Level	Recorder 0-2 ft. Indicator 0-2 ft.	(1) (5)
1	Control Rod Position	28 Volt Indicating Lights	(1) (2) (3) (4)
1	Neutron Monitoring	SRM,IRM,LPRM 0-100%	
1	Safety-Relief Valve Position Indication	Acoustic or thermocouple	(8)

* Effective when modification associated with this amendment request is complete.
** Delete when modification associated with this amendment request is complete.
*** Notes for Table 3.2.F appear on page 78.

NOTES FOR TABLE 3.2.F

1) From and after the date that one of these parameters is reduced to one indication, continued operation is permissible during the succeeding thirty days unless such instrumentation is sooner made operable.

2) From and after the date that one of these parameters is not indicated in the control room, continued operation is permissible during the succeeding seven days unless such instrumentation is sooner made operable.

3) If the requirements of notes (1) and (2) cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold condition within 24 hours.

4) These surveillance instruments are considered to be redundant to each other.

5) In the event that all indications of this parameter are disabled and such indication cannot be restored in six (6) hours, an orderly shutdown shall be initiated and the reactor shall be in a Hot Shutdown condition in six (6) hours and a Cold Shutdown condition in the following eighteen (18) hours.

6) With the number of operable channels less than the minimum number of instrumentation channels shown in Table 3.2.F, either restore the inoperable channel to an operable status within 7 days, or be in at least hot shutdown within the next 12 hours.

7) If this parameter is not indicated in the control room, either restore at least one inoperable channel to operable status within 48 hours or be in at least hot shutdown within the next 12 hours.

8) If this parameter is not indicated in the control room, either restore at least one channel to operable status within thirty days or be in at least hot shutdown within the next 12 hours.

9) A suppression Chamber Water Temperature instrument channel will be considered operable if there are at least ten (10) resistance temperature detector inputs operable and no two (2)adjacent resistance temperature detector inputs are inoperable.

TABLE 4.2.F Unit 3

MINIMUM TEST AND CALIBRATION FREQUENCY FOR SURVEILLANCE INSTRUMENTATION

Instrument Channel	Calibration Frequency	Instrument Check
1) Reactor Level	Once/Operating cycle	Once Each Shift
2) Reactor Pressure	Once/6 months	Once Each Shift
3) Drywell Pressure	Once/6 months	Once Each Shift
4) Drywell Temperature	Once/6 months	Once Each Shift
5) Suppression Chamber Water Temperature	Once/operating cycle** Once/6 months***	Once Each Day** Once Each Shift***
6) Suppression Chamber Water Level	Once/6 months	Once Each Shift
7) Control Rod Position	NA	Once Each Shift
8) Neutron Monitoring (APRM)	Twice Per Week	Once Each Shift
9) Safety/Relief Valve Position Indicator (acoustics)	Once/Operating cycle	Once/Month
10) Safety/Relief Valve Position Indicator (thermocouple)	NA*	Once/month
11) Safety Valve Position Indicator (Acoustics)	Once/operating cycle	Once/month
12) Safety Valve Position Indicator (thermocouple)	NA*	Once/month

* Perform instrument functional check once per operating cycle,
** Effective when modification associated with this amendment request is complete.
***Delete when modification associated with this amendment request is complete.

LIMITING CONDITIONS FOR OPERATION | SURVEILLANCE REQUIREMENTS

3.7 CONTAINMENT SYSTEMS

Applicability:

Applies to the operating status of the primary and secondary containment systems.

Objective:

To assure the integrity of the primary and secondary containment system.

Specification:

A. Primary Containment

1. Whenever the nuclear system is pressurized above atmospheric pressure or work is being done which has the potential to drain the vessel, the pressure suppression pool water volume and temperature shall be maintained within the following limits except as specified by 3.7.A.2, or when inoperability of the core spray systems, the LPCI and containment cooling subsystems is permissible as provided for in 3.5.F.3 and 3.5.F.4.b.

   a. Minimum water volume- 122,900 ft^3

   b. Maximum water volume- 127,300 ft^3

4.7 CONTAINMENT SYSTEMS

Applicability:

Applies to the primary and secondary containment integrity.

Objective:

To verify the integrity of the primary and secondary containment.

Specification:

1. The suppression chamber water level and temperature shall be checked once per day.

2. Whenever there is indication of relief valve operation (except when the reactor is being shutdown and torus cooling is being established) or testing which adds heat to the suppression pool, the pool temperature shall be continually monitored and also observed and logged every 5 minutes until the heat addition is terminated.

3. Whenever there is indication of relief valve operation with the local suppression pool temperature reaching 200°F or more, an external visual examination of the suppression chamber shall be conducted before resuming power operation.

4. A visual inspection of the suppression chamber interior, including water line regions shall be made at each major refueling outage.

LIMITING CONDITIONS FOR OPERATION | SURVEILLANCE REQUIREMENTS

3.7.A Primary Containment (Cont'd)

c. Maximum average suppression pool temperature limits:

(1) During startup/hot standby and run modes, with the suppression pool temperature greater than 95°F, except as permitted below, restore the temperature to less than 95°F within 24 hours or be in hot shutdown within the next 12 hours and cold shutdown within the following 24 hours.

(2) During testing which adds heat to the suppression pool, the pool temperature shall not exceed 105°F. Should the pool temperature exceed 105°F, such testing shall be stopped and the pool temperature must be reduced to below the limit specified in (1) above within 24 hours or be in hot shutdown within the next 12 hours and cold shutdown within the following 24 hours.

(3) The reactor shall be scrammed from any operating condition if the pool temperature reaches 110°F. Power operation shall not be resumed until the pool temperature is reduced below the limit specified in (1) above.

(4) During reactor isolation conditions, the reactor pressure vessel shall be depressurized to less than 200 psig at normal cooldown rates if the pool temperature reaches 120°F.

3.7.A & 4.7.A BASES (Cont'd)

The maximum allowable volume assures the integrity and functional capability of the Suppression Chamber (torus) during postulated LOCA pool swell effects on the torus support system. The majority of the Bodega tests were run with a submerged length of 4 feet and with complete condensation. Thus, with respect to downcomer submergence, this specification is adequate. The maximum temperature at the end of blowdown tested during the Humboldt Bay and Bodega Bay tests was 170°F and this is conservatively taken to be the limit for complete condensation of the reactor coolant, although condensation would occur for temperatures above 170°F.

Should it be necessary to drain the suppression chamber, this should only be done when there is no requirement for core standby cooling systems operability as explained in basis 3.5.F.

Experimental data indicates that excessive steam condensing loads can be avoided if the peak temperature of the suppression pool water at the quencher discharge is maintained below 200°F during any period of relief valve operation discharging through tee quenchers.

Because of the large volume and thermal capacity of the suppression pool, the volume and temperature changes very slowly and monitoring these parameters daily is sufficient to establish any temperature trends. By requiring the suppression pool temperature to be continually monitored and frequently logged during periods of testing which add significant heat, the temperature trends will be closely followed so that appropriate action can be taken if required. Logging is not required during inadvertent relief valve operation since during such periods operator action is actively and directly involved in operations relating to controlling torus temperature and monitoring of temperature trends is a natural part of the operations. Additionally, torus temperature is monitored by a recorder during these periods so that an historical record is available.

Operating procedures define the action to be taken in the event a relief valve inadvertently opens or sticks open. As a minimum this action shall include: (1) use of all available means to close the valve, (2) initiate suppression pool water cooling heat exchangers, (3) initiate reactor shutdown, and (4) if other relief valves are used to depressurize the reactor, their discharge shall be separated from that of the stuck-open relief valve to assure mixing and uniformity of energy insertion to the pool.

The requirement for an external visual examination following any event where potentially high loadings could occur provides assurance that no significant damage was encountered. Particular attention should be focused on structural discontinuities in the vicinity of the relief valve discharge since these are expected to be the points of highest stress.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENTS NOS. 93 AND 95 TO FACILITY OPERATING LICENSES

NOS. DPR-44 AND DPR-56

PHILADELPHIA ELECTRIC COMPANY

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3

DOCKETS NOS. 50-277 AND 50-278

1.0 INTRODUCTION

Pursuant to Section 210 of the Energy Reorganization Act of 1974, the capability of the boiling water reactor (BWR) Mark I containment suppression chamber to withstand suppression pool hydrodynamic loads which were not considered in the original design of the structures was designated an "Unresolved Safety Issue" (Task Action Plan A-7). The objectives of the Mark I Containment Long Term Program were to establish design-basis loads that are appropriate for the anticipated life of each Mark I BWR facility and to restore the originally intended design safety margins for each Mark I Containment System. The principal thrust of the program has been the development of generic methods for the definition of suppression pool hydrodynamic loading events and the associated structural assessment techniques for the Mark I containment configurations. Generic analysis techniques were used to perform plant unique analyses (PUA) for each Mark I facility. The NRC staff, in the "Mark I Containment Long Term Program Safety Evaluation Report" (NUREG-0661, including Supplement 1), concluded that the load definition procedures utilized by the BWR Owners' Group, as modified by the NRC staff requirements, provide conservative estimates of these loading conditions and that the structural acceptance criteria are consistent with the requirements of the applicable codes and standards.

Some of the modifications made by Philadephia Electric Company (the licensee) to satisfy the criteria of Appendix A of NUREG-0661 require the licensee to request revisions to the Peach Bottom Technical Specifications.

2.0 EVALUATION

By letter dated October 15, 1982, the licensee requested Technical Specification changes that reflect the following: 1) modifications to the suppression pool temperature monitoring system that meet the NUREG-0661 criteria; 2) changes to the testing and calibration frequencies for the suppression chamber water

temperature that are consistent with the BWR Standard Technical Specifications (NUREG-0123, Revision 3); and 3) revisions to the suppression pool temperature limit to conform with the NUREG-0661 criteria.

Since the modifications as stated by the licensee restore the originally intended safety margins and the proposed revisions to the Technical Specifications conform with the criteria of Appendix A of NUREG-0661, we find these revisions to be acceptable. However, we wish to point out that approval of these Technical Specification changes should not be interpreted as our approval of the Peach Bottom Mark I Containment Long Term Program modifications. The acceptability of this program will be determined from our post-implementation audit review of the licensee's PUA report submittal. If any portions of the PUA report are found to be unacceptable in this post-implementation audit, then this could affect these Technical Specification changes and the licensee would have to submit revised Technical Specifications for our review and approval.

The licensee also proposed deletion of obsolete specifications and footnotes for Peach Bottom Unit No. 3 since the modifications authorized by Amendment No. 67 have been completed. In addition, several minor changes in Section 3.7.A.1.c were proposed to remove a typographical error and to provide greater clarity to this section. We find these changes to be acceptable.

3.0 SUMMARY

We conclude that the proposed changes to the Technical Specifications are consistent with the Mark I Containment Long Term Program and, therefore, are acceptable. The other requested revisions involving deletion of obsolete specifications and footnotes, editorial corrections and minor word changes to provide additional clarity are also acceptable.

4.0 ENVIRONMENTAL CONSIDERATION

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 13, 1984

The following NRC personnel have contributed to this Safety Evaluation: G. Gears, B. Siegel.