

February 18, 1988

Dockets Nos. 50-277/278

Mr. Edward G. Bauer, Jr.
Vice President and General Counsel
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

SUBJECT: VERIFICATION OF VACUUM BREAKER CLOSURE
(TAC NOS. 46787 AND 46788)

RE: PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3

The Commission has issued the enclosed Amendments Nos. 127 and 130 to Facility Operating License Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Unit Nos. 2 and 3. These amendments consist of changes to the Technical Specifications in response to your application dated August 6, 1981 as supplemented on April 2, 1984, December 2, 1985, October 29, 1986 and July 7, 1987.

These amendments make changes to the methods of verifying drywell - suppression chamber vacuum breaker closure and include changes of an administrative nature to correct errors, to establish consistency and for editorial clarity.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

/s/

Robert E. Martin, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 127 to DPR-44
2. Amendment No. 130 to DPR-56
3. Safety Evaluation

cc w/enclosures:
See next page

8803030008 880218
PDR ADOCK 05000277
P PDR

PDI
MO'Brien
V 2/20/88

PDI-2/RM
REMartin:mr
6/1/22/88

OGC *KL*
S H Lewis
2/1/88

PDI-2/D
WButler
2/18/88 *WB*

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MO'Brien(2)		



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555
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RE: PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3

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Sincerely,

A handwritten signature in cursive script that reads "Robert E. Martin".

Robert E. Martin, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 127 to DPR-44
2. Amendment No. 130 to DPR-56
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. E. G. Bauer, Jr.
Philadelphia Electric Company

Peach Bottom Atomic Power Station,
Units 2 and 3

cc:

Mr. J. S. Kemper, Senior Vice President
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Mr. R. A. Heiss, Coordinator
Pennsylvania State Clearinghouse
Governor's Office of State Planning
and Development
P. O. Box 1323
Harrisburg, Pennsylvania 17120

Troy B. Conner, Jr., Esq.
1747 Pennsylvania Avenue, N.W.
Washington, D.C. 20006

Mr. Thomas M. Gerusky, Director
Bureau of Radiation Protection
Pennsylvania Department of
Environmental Resources
P. O. Box 2063
Harrisburg, Pennsylvania 17120

Philadelphia Electric Company
ATTN: Mr. D. M. Smith, Vice President
Peach Bottom Atomic Power Station
Route 1, Box 208
Delta, Pennsylvania 17314

Mr. Albert R. Steel, Chairman
Board of Supervisors
Peach Bottom Township
R. D. #1
Delta, Pennsylvania 17314

Mr. W. M. Alden
Engineer-In-Charge-Licensing
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Mr. Gary Mock
P. O. Box 09181
Columbus, Ohio 43209

Morgan J. Morris, III
General Manager - Operating Services
Atlantic Electric
P. O. Box 1500
1199 Black Horse Pike
Pleasantville, New Jersey 08232

Mr. Jack Urban
General Manager, Production
Delmarva Power and Light Company
800 King Street
Wilmington, Delaware 19899

Resident Inspector
U. S. Nuclear Regulatory Commission
Peach Bottom Atomic Power Station
P. O. Box 399
Delta, Pennsylvania 17314

Mr. Tom Magette
Power Plant Research Program
Department of Natural Resources
B-3
Tawes State Office Building
Annapolis, Maryland 21401

Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Mr. Roland Fletcher
Department of Environment
201 West Preston Street
Baltimore, Maryland 21201



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 127
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated August 6, 1981 as supplemented on April 2, 1984, December 2, 1985, October 29, 1986 and July 7, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

8803030014 880218
PDR ADDCK 05000277
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 127, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective within 60 days of its date of issuance to accommodate the writing and approval of implementing procedures.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 18, 1988

PDI-2/D
MQB:ven
1/20/88

PDI-2/RM
REMartin:mr
01/24/88

OGC ✓ WZ
S H Lewis
2/1/88

PDI-2/D
WButler
2/18/88

WB

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 127, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective within 60 days of its date of issuance to accommodate the writing and approval of implementing procedures.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 18, 1988

ATTACHMENT TO LICENSE AMENDMENT NO.127

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

<u>Remove</u>	<u>Insert</u>
128	128
170	170
171	171

Replace the following page of the Appendix B Technical Specifications with the enclosed page. The revised area is indicated by marginal lines.

51	51
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LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENT				
<p>3.5.B <u>Containment Cooling Subsystem (cont'd.)</u></p>	<p>4.5.B <u>Containment Cooling Subsystem (cont'd.)</u></p>				
<p>3. From and after the date that any 3 HPSW pumps are made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding fifteen days unless such pumps are sooner made operable provided all remaining components of the containment cooling system are operable.</p>	<p>3. When it is determined that any 3 HPSW pumps are inoperable, the remaining components of both containment cooling subsystems shall be demonstrated to be operable immediately and weekly thereafter.</p>				
<p>4. From and after the date that 3 containment cooling subsystem loops are made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding seven days unless such subsystem loop is sooner made operable, provided that all active components of the other containment cooling subsystem loop, including its associated diesel generators, are operable.</p>	<p>4. When 3 containment cooling subsystem loops become inoperable, the operable subsystem loop and its associated diesel-generator shall be demonstrated to be operable immediately and the operable containment cooling subsystem loop daily thereafter.</p>				
<p>5. If the requirements of 3.5.B cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a Cold Shutdown Condition within 24 hours.</p>					
<p>C. <u>HPCI Subsystem</u></p>	<p>C. <u>HPCI Subsystem</u></p>				
<p>1. The HPCI Subsystem shall be operable whenever there is irradiated fuel in the reactor vessel, reactor pressure is greater than 105 psig, and prior to reactor startup from a Cold Condition, except as specified in 3.5.C.2 and 3.5.C.3 below.</p>	<p>1. HPCI Subsystem testing shall be performed as follows:</p> <table border="1" data-bbox="885 1612 1421 1774"> <thead> <tr> <th data-bbox="987 1612 1057 1640"><u>Item</u></th> <th data-bbox="1214 1612 1365 1640"><u>Frequency</u></th> </tr> </thead> <tbody> <tr> <td data-bbox="885 1667 1133 1774">(a) Simulated Automatic Actuation Test</td> <td data-bbox="1214 1667 1398 1724">Once/operating cycle</td> </tr> </tbody> </table>	<u>Item</u>	<u>Frequency</u>	(a) Simulated Automatic Actuation Test	Once/operating cycle
<u>Item</u>	<u>Frequency</u>				
(a) Simulated Automatic Actuation Test	Once/operating cycle				

~~April 1973~~
Amendment No. 127

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS| 3.7.A Primary Containment (Cont'd.)4.7.A Primary Containment (Cont'd.) |3. Pressure Suppression Chamber -
Reactor Building Vacuum Breakers

- a. Except as specified in 3.7.A.3.b below, two pressure suppression chamber-reactor building vacuum breakers shall be operable at all times when primary containment integrity is required. The setpoint of the differential pressure instrumentation which actuates the pressure suppression chamber-reactor building vacuum breakers shall be 0.5 ± 0.25 psid.

- b. From and after the date that one of the pressure suppression chamber-reactor building vacuum breakers is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding seven days unless such vacuum breaker is sooner made operable provided that the repair procedure does not violate primary containment integrity.

4. Drywell-Pressure Suppression Chamber Vacuum Breakers

- a. When primary containment is required, all drywell-suppression chamber vacuum breakers shall be operable and positioned in the fully closed position (except during testing) except as specified in 3.7.A.4.b and c below.

- b. Drywell-suppression chamber vacuum breaker(s) may be "not fully seated" as shown by position indication if testing confirms that the bypass area is less than or equivalent to a one-inch diameter hole. Testing shall be initiated within 8 hours of initial detection of a "not fully seated" position

3. Pressure Suppression Chamber -
Reactor Building Vacuum Breakers

- a. The pressure suppression chamber-reactor building vacuum breakers and associated instrumentation including setpoint shall be checked for proper operation every refueling outage.

4. Drywell-Pressure Suppression Chamber Vacuum Breakers

- a. Each drywell-suppression chamber vacuum breaker shall be exercised through an opening-closing cycle once a month.
- b. When it is determined that a vacuum breaker is inoperable for opening at a time when operability is required, all other operable vacuum breakers shall be exercised immediately and every 15 days thereafter until the inoperable vacuum breaker has been returned to normal service.

- c. Once per operating cycle each vacuum breaker shall be visually inspected

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.7.A Primary Containment (Cont'd)

4.7.A Primary Containment (Cont'd.)

indication and shall be performed periodically thereafter as follows:

- (1) Once every 15 days (only if "not fully seated" valve(s) are indicated).
- (2) Within 24 hours following vacuum breaker exercising required by 4.7.A.4.a and 4.7.A.4.B. (only if "not fully seated" valve(s) are indicated).

- c. Two drywell-suppression chamber vacuum breakers may be inoperable for opening.
- d. If specifications 3.7.A.4.a, b, or c cannot be met, the situation shall be corrected within 24 hours or the unit shall be placed in a cold shutdown condition in an orderly manner.

5. Oxygen Concentration

- a. The primary containment atmosphere shall be reduced to less than 4% oxygen with nitrogen gas during reactor power operation with reactor coolant pressure above 100 psig, except as specified in 3.7.A.5.b.
- b. Within the 24-hour period, subsequent to placing the reactor in the RUN mode following a shutdown, the containment atmosphere oxygen concentration shall be reduced to less than 4% by volume and maintained in this condition. De-inerting may commence 24 hours prior to a shutdown.

to insure proper maintenance and operation.

- d. A leak test of the drywell to suppression chamber structure shall be conducted at each refueling outage to assure no bypass larger than or equivalent to a one-inch diameter hole exists between the drywell and suppression chamber.

5. Oxygen Concentration

The primary containment oxygen concentration shall be measured and recorded at least twice weekly.

7.4 Plant Reporting Requirements

7.4.1 Routine Reports

In addition to the environmental monitoring information required by Section 6.9.3.h of Appendix A to the Operating License, the following information shall be submitted in an annual report:

- A. Records of special study programs data and analysis thereof.
- B. Records of changes to the plant which affect the environmental impact of the facility.
- C. Records of changes to environmental permits and certificates.

7.4.2 Non-Routine Reports

A. Environmental Deviation Reports

In the event of an environmental deviation as defined in the environmental technical specifications, notification shall be made within 24 hours by telephone or telegraph to the Director of the NRC Regional Inspection and Enforcement Office. A written report shall follow within 10 working days to the Director of Nuclear Reactor Regulation (copy to the Director of Regional Inspection and Enforcement Office).

The written report on an environmental deviation, and to the extent possible, the preliminary telephone and telegraph notification, should: (a) describe, analyze, and evaluate implications, (b) determine the cause of the occurrence and (c) indicate the corrective action (including any significant changes made in procedures) taken to preclude repetition of the occurrence and to prevent similar occurrences involving similar components or systems.

B. Reporting of Changes to the Plant or Permits

A written report, including an evaluation of the environmental impact resulting from a change, shall be forwarded to the Director, Office of Nuclear Reactor Regulation (copy to the Director of the Regional Inspection and Enforcement Office) in the event of:

- 1. Changes to the plant that affect the environmental impact evaluation contained in the Environmental Report or the



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 130
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated August 6, 1981 as supplemented on April 2, 1984, December 2, 1985, October 29, 1986 and July 7, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 130, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

- 3. This license amendment is effective within 60 days of its date of issuance to accommodate the writing and approval of implementing procedures.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 18, 1988

PDI-2/MA
M...
1/20/88

PDI-2/PM
RE Martin:mr
01/22/88

OGC
S H Lewis
2/1/88

PDI-2/D
WButler
2/1/88

WB

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 130, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective within 60 days of its date of issuance to accommodate the writing and approval of implementing procedures.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 18, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 130

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

<u>Remove</u>	<u>Insert</u>
128	128
170	170
171	171

Replace the following page of the Appendix B Technical Specifications with the enclosed page. The revised area is indicated by marginal lines.

51	51
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LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENT				
<p><u>3.5.B Containment Cooling Subsystem (cont'd.)</u></p> <p>3. From and after the date that any 3 HPSW pumps are made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding fifteen days unless such pumps are sooner made operable provided all remaining components of the containment cooling system are operable.</p> <p>4. From and after the date that 3 containment cooling subsystem loops are made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding seven days unless such subsystem loop is sooner made operable, provided that all active components of the other containment cooling subsystem loop, including its associated diesel generators, are operable.</p> <p>5. If the requirements of 3.5.B cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a Cold Shutdown Condition within 24 hours.</p>	<p><u>4.5.B Containment Cooling Subsystem (cont'd.)</u></p> <p>3. When it is determined that any 3 HPSW pumps are inoperable, the remaining components of both containment cooling subsystems shall be demonstrated to be operable immediately and weekly thereafter.</p> <p>4. When 3 containment cooling subsystem loops become inoperable, the operable subsystem loop and its associated diesel-generator shall be demonstrated to be operable immediately and the operable containment cooling subsystem loop daily thereafter.</p>				
<p><u>C. HPCI Subsystem</u></p> <p>1. The HPCI Subsystem shall be operable whenever there is irradiated fuel in the reactor vessel, reactor pressure is greater than 105 psig, and prior to reactor startup from a Cold Condition, except as specified in 3.5.C.2 and 3.5.C.3 below.</p>	<p><u>C. HPCI Subsystem</u></p> <p>1. HPCI Subsystem testing shall be performed as follows:</p> <table border="1" data-bbox="868 1606 1388 1770"> <thead> <tr> <th data-bbox="966 1606 1039 1638"><u>Item</u></th> <th data-bbox="1193 1606 1339 1638"><u>Frequency</u></th> </tr> </thead> <tbody> <tr> <td data-bbox="868 1659 1112 1770">(a) Simulated Automatic Actuation Test</td> <td data-bbox="1193 1659 1388 1722">Once/operating cycle</td> </tr> </tbody> </table>	<u>Item</u>	<u>Frequency</u>	(a) Simulated Automatic Actuation Test	Once/operating cycle
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(a) Simulated Automatic Actuation Test	Once/operating cycle				

~~April 1973~~

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.7.A Primary Containment (Cont'd.)4.7.A Primary Containment (Cont'd.)3. Pressure Suppression Chamber -
Reactor Building Vacuum Breakers

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LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.7.A Primary Containment (Cont'd)

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c. Two drywell-suppression chamber vacuum breakers may be inoperable for opening.

d. If specifications 3.7.A.4.a, b, or c cannot be met, the situation shall be corrected within 24 hours or the unit shall be placed in a cold shutdown condition in an orderly manner.

5. Oxygen Concentration

- a. The primary containment atmosphere shall be reduced to less than 4% oxygen with nitrogen gas during reactor power operation with reactor coolant pressure above 100 psig, except as specified in 3.7.A.5.b.
- b. Within the 24-hour period, subsequent to placing the reactor in the RUN mode following a shutdown, the containment atmosphere oxygen concentration shall be reduced to less than 4% by volume and maintained in this condition. De-inerting may commence 24 hours prior to a shutdown.

4.7.A Primary Containment (Cont'd.)

to insure proper maintenance and operation.

- d. A leak test of the drywell to suppression chamber structure shall be conducted at each refueling outage to assure no bypass larger than or equivalent to a one-inch diameter hole exists between the drywell and suppression chamber.

5. Oxygen Concentration

The primary containment oxygen concentration shall be measured and recorded at least twice weekly.

~~October 1973~~

Amendment No. 130

7.4 Plant Reporting Requirements

7.4.1 Routine Reports

In addition to the environmental monitoring information required by Section 6.9.3.h of Appendix A to the Operating License, the following information shall be submitted in an annual report:

- A. Records of special study programs data and analysis thereof.
- B. Records of changes to the plant which affect the environmental impact of the facility.
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7.4.2 Non-Routine Reports

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A written report, including an evaluation of the environmental impact resulting from a change, shall be forwarded to the Director, Office of Nuclear Reactor Regulation (copy to the Director of the Regional Inspection and Enforcement Office) in the event of:

- 1. Changes to the plant that affect the environmental impact evaluation contained in the Environmental Report or the



UNITED STATES
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WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION SUPPORTING

AMENDMENT NOS. 127 AND 130 TO FACILITY OPERATING

LICENSE NOS. DPR-44 and DPR-56

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3

DOCKET NOS. 50-277 AND 50-278

1.0 INTRODUCTION

By letter dated August 6, 1981 as supplemented on April 2, 1984, December 2, 1985, October 29, 1986 and July 7, 1987, Philadelphia Electric Company requested an amendment to Facility Operating License Nos. DPR-44 and DPR-56 for Peach Bottom Atomic Power Station, Unit Nos. 2 and 3. The amendments in the August 6, 1981 submittal were in the areas of (1) Methods of verifying drywell-suppression chamber vacuum breaker closure, (2) the operability of radiation monitors, (3) the listing of safety related shock suppressors and (4) several changes of an administrative nature to correct errors, to establish consistency and for editorial clarity. By letter dated April 2, 1984 the licensee withdrew the changes in parts (2) and (3) above from the scope of this application. Based on further interactions with NRC staff the licensee further amended the application by letters dated December 2, 1985, October 29, 1986 and July 7, 1987.

The resulting agenda of changes proposed by the licensee in these documents which are acted upon in this amendment are as follows:

- (a) From the August 6, 1981 application; correction of the spelling of the word "greater" on page 128; and the vacuum breaker technical specifications, as amended by the later submittals, on page 171 of the Appendix A Technical Specifications. The licensee's August 6, 1981 submittal also proposed a clarification of the reporting period for submission of a written report on page 51 of the Environmental Technical Specifications in Appendix B to the facility licenses.
- (b) The December 2, 1985 submittal proposed revisions in the vacuum breaker specifications in 3.7.A.4.b on pages 170 and 171; corrected the spelling of "except" in 3.7.A.3.a on page 170; deleted the redundant word "valve" from the term "vacuum breaker valve" in 4.7.A.4.b and c on page 170; deleted the extraneous words "determined to be" from 3.7.A.4.c on page 171; changed the word "will" to "shall" in 3.7.A.4.d on page 171; clarified the event for conducting the

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subject test in 4.7.A.4.d as a "refueling outage" versus the prior term "refueling"; deleted an obsolete and extraneous reference to the initial startup test program and demonstration of electrical output in 3.7.A.5 and added page 171a to accommodate the expansion of the specification 3.7.A text.

- (c) The October 29, 1986 submittal revised the specification 3.7.A.4.b requirement to initiate testing within 8 hours to a requirement to perform testing within 24 hours; consolidated the specification 3.7.A.4.b requirement for periodic testing following initial detection of a "not fully seated" position to include the previous 15 day and 24 hour tests based on whether indication exists that the vacuum breaker is fully seated and eliminated the need for a page 171a.
- (d) The July 7, 1987 submittal again revised the specification 3.7.A.4.b requirement on page 170 back to the form it had in the December 2, 1985 submittal. This change was made in response to the staff's letter dated April 28, 1987.

The NRC staff has published notice of consideration of this amendment in the Federal Register on June 20, 1984 (49 FR 25369) and on January 29, 1986 (51 FR 3716). Those notices reflected the August 6, 1981 and December 2, 1985 submittals. The changes introduced by the October 29, 1986 and July 7, 1987 submittals are within the scope of these notices since the wording of the more significant item, the 3.7.A.4.b testing requirement on page 170, has reverted to the wording for this item in the December 2, 1985 submittal and the provision allowing testing to be extended from once per 15 days to a monthly interval under certain conditions has been deleted.

2.0 EVALUATION

By letter dated December 2, 1985, the licensee revised their earlier proposal (August 6, 1981) for a change in TS 3.7.A.4.b relating to Drywell-Pressure Suppression Chamber Vacuum Breakers (DSVB). The licensee stated that the revised proposal reflected staff's suggestions made during a meeting on February 26, 1985. Specifically, the revised proposal stated that the DSVBs would be considered fully closed even though the "not fully seated" position indication is shown, provided that a leak test is initiated within 8 hours of such detection. This test should confirm that the bypass area between the drywell and the suppression chamber is less than or equivalent to a one-inch diameter hole. Additionally, the revised proposal identified the frequency of followup confirmatory periodic leak tests should a "not fully seated" position indication exist for any DSVB.

Based on review of the revised proposal, the staff by telephone conversation on February 5, 1986, suggested some modifications to the proposed change in the area of follow-up periodic leak tests. In response, the licensee provided a submittal dated October 29, 1986 and

subsequently revised it by a submittal dated July 7, 1987. In summary, the licensee has replaced the TS 3.7.A provision permitting continuous operation with one DSVB in the position between "fully closed" and "3 degrees open" with (a) a requirement for initiation of a bypass area leakage test within 8 hours of detection of a "not fully seated" position indication (December 2, 1985 and July 7, 1987 submittals), and (b) additional follow-up periodic leak tests to ensure that (1) testing is performed within 24 hours following DSVB exercising required by surveillance requirement 4.7.A.4.a or b and (2) the time interval between any two consecutive tests, including the initial test, is not to exceed 15 days.

The staff has reviewed the licensee's proposed changes to Peach Bottom Units 2 and 3 TS 3.7.A.4.b as given in the above submittals and the associated justification. Based on the review, the staff finds the proposed approach for considering the DSVBs to be "fully seated," i.e., confirming that the bypass area is less than or equal to the area corresponding to a one-inch diameter hole by leak tests, to be acceptable. This is because, the proposed approach assures that the bypass leakage following an accident would be less than the maximum allowable bypass leakage. Additionally, the staff finds that the proposed change will allow the licensee greater operational flexibility than what is currently available in the sense that with the change, more than one DSVB can be "not fully seated" as shown by the corresponding position indications. However, acceptance is based on the initial and periodic leak tests confirming that the limiting value for the bypass area mentioned above is not exceeded. The staff also finds that the proposed initial and periodic confirmation leak tests provide reasonable assurance that the DSVBs will not be open in excess of the limiting bypass area, when they are required to remain closed.

Based on the above, the staff concludes that the licensee's proposed changes to TS 3.7.A.4.b, 3.7.A.4.c, 3.7.A.4.d, 4.7.A.4.b, c and d for Peach Bottom, Units 2 and 3, are acceptable. Acceptance is based, particularly, on the staff's finding that with the proposed change, the maximum bypass leakage following an accident should be less than the maximum allowable bypass leakage. The staff further finds the minor changes to TS 3.7.A, 4.7.A on TS pages 170 and 171, the change on page 128 and the change on page 51 of the Appendix B TS serve to correct errors, to establish consistency, to provide editorial clarity and do not change the intent of the TS. These changes are, therefore, acceptable.

3.0 ENVIRONMENTAL CONSIDERATIONS

These amendments involve a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or

cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of the amendments.

4.0 CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (49 FR 25369) on June 20, 1984 and (51 FR 3716) on January 29, 1986 and consulted with the State of Pennsylvania. No public comments were received and the State of Pennsylvania did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. Chandrasekaran

Dated: February 18, 1988