

May 16, 2001

Mr. Oliver D. Kingsley, President
Exelon Nuclear
Exelon Generation Company, LLC
Quad Cities Nuclear Power Station
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: QUAD CITIES NUCLEAR POWER STATION
NRC INSPECTION REPORT 50-254/01-07(DRS); 50-265/01-07(DRS)

Dear Mr. Kingsley:

On April 27, 2001, the NRC completed an inspection at your Quad Cities Nuclear Power Station. The enclosed report documents the inspection findings which were discussed on April 27, 2001, with Mr. Barnes and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel. Specifically, this inspection focused on the baseline annual inspection for evaluations of changes, tests, or experiments (10 CFR 50.59) and the baseline biennial inspection for permanent plant modifications.

No findings of significance were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

Ronald N. Gardner, Chief
Electrical Engineering Branch
Division of Reactor Safety

Docket Nos. 50-254; 50-265
License Nos. DPR-29; DPR-30

Enclosure: Inspection Report 50-254/01-07(DRS);
50-265/01-07(DRS)

See Attached Distribution

cc w/encl: W. Bohlke, Senior Vice President, Nuclear Services
C. Crane, Senior Vice President - Mid-West Regional
J. Cotton, Senior Vice President - Operations Support
J. Benjamin, Vice President - Licensing and Regulatory Affairs
R. Krich, Director - Licensing
H. Stanley, Operations Vice President
J. Skolds, Chief Operating Officer
R. Helfrich, Senior Counsel, Nuclear
DCD - Licensing
T. J. Tulon, Site Vice President
G. Barnes, Quad Cities Station Manager
W. Beck, Regulatory Affairs Manager
W. Leach, Manager - Nuclear
Vice President - Law and Regulatory Affairs
Mid American Energy Company
M. Aguilar, Assistant Attorney General
Illinois Department of Nuclear Safety
State Liaison Officer, State of Illinois
State Liaison Officer, State of Iowa
Chairman, Illinois Commerce Commission

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U. S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket Nos: 50-254, 50-265
License Nos: DPR-29, DPR-30

Report No: 50-254/01-07(DRS), 50-265/01-07(DRS)

Licensee: Exelon Generation Co., LLC

Facility: Quad Cities Nuclear Power Station

Location: 22710 206th Avenue North
Cordova, IL 61242

Dates: April 23 through 27, 2001

Inspectors: George M. Hausman, Senior Reactor Inspector
Darrell L. Schrum, Reactor Inspector
Robert A. Winter, Reactor Inspector

Approved by: Ronald N. Gardner, Chief
Electrical Engineering Branch
Division of Reactor Safety

SUMMARY OF FINDINGS

IR 05000254-01-07(DRS), IR 05000265-01-07(DRS), on 04/23-04/27/2001, Exelon Generation Co., LLC, Quad Cities Nuclear Power Station. Evaluations of Changes, Tests, or Experiments and Permanent Plant Modifications Report.

The report covers a five day announced inspection. The inspection was conducted by three Region III based reactor inspectors. No findings of significance were identified. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using IMC 0609 "Significance Determination Process" (SDP). The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described at its Reactor Oversight Process website at <http://www.nrc.gov/NRR/OVERSIGHT/index.html>. Findings for which the SDP does not apply are indicated by "No Color" or by the severity level of the applicable violations.

A. Inspector Identified Findings

Cornerstones: Initiating Events, Mitigating Systems, Barrier Integrity

No findings of significance were identified.

Report Details

Summary of Plant Status

Unit 1 was at 100 percent power during the inspection period, until a shutdown was commenced on April 26, 2001. Unit 2 was at 100 percent power throughout the inspection period.

1. REACTOR SAFETY

Cornerstones: Initiating Events, Mitigating Systems, and Barrier Integrity

1R02 Evaluations of Changes, Tests, or Experiments (71111.02)

.1 Review of 10 CFR 50.59 Evaluations and Screenings for Changes, Tests, or Experiments

a. Inspection Scope

The inspectors reviewed ten evaluations performed pursuant to 10 CFR 50.59. The evaluations related to permanent plant modifications, setpoint changes, procedure changes, conditions adverse to quality, and changes to the updated final safety analysis report. The inspectors confirmed that the evaluations were thorough and that prior NRC approval was obtained as appropriate. The inspectors also reviewed 15 screenings where the licensee had determined that a 10 CFR 50.59 evaluation was not necessary. In regard to the changes reviewed where no 10 CFR 50.59 evaluation was performed, the inspectors verified that the changes did not meet the threshold to require a 10 CFR 50.59 evaluation. These evaluations and screenings were chosen based on risk significance of samples from the different cornerstones.

b. Findings

No findings of significance were identified.

1R17 Permanent Plant Modifications (71111.17)

.1 Review of Recent Permanent Plant Modifications

a. Inspection Scope

The inspectors reviewed ten permanent plant modifications that were installed in the last two years. The modifications were chosen based upon their affecting systems that had high probabilistic risk analysis (PRA) significance in the licensee's Individual Plant Evaluation (IPE) or high maintenance rule safety significance. The inspectors reviewed the modifications to verify that the completed design changes were in accordance with the specified design requirements and the licensing bases and to confirm that the changes did not affect any systems' safety function. Design and post-modification

testing aspects were verified to ensure the functionality of the modification, its associated system, and any support systems. The inspectors also verified that the modifications performed did not place the plant in an increased risk configuration.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES (OA)

4OA2 Identification and Resolution of Problems

a. Inspection Scope

The inspectors reviewed a selected sample of condition reports associated with Quad Cities' permanent plant modifications and concerning 10 CFR 50.59 evaluations and screenings to verify that the licensee had an appropriate threshold for identifying issues. The inspectors evaluated the effectiveness of the corrective actions for the identified issues.

b. Findings

No findings of significance were identified.

4OA6 Meetings

Exit Meeting

The inspectors presented the inspection results to Mr. Barnes and other members of licensee management at the conclusion of the inspection on April 27, 2001. The licensee acknowledged the findings presented. No proprietary information was identified.

KEY POINTS OF CONTACT

Licensee

D. Barker, Radiation Protection Manager
G. Barnes, Station Manager
W. Beck, Regulatory Assurance Manager
T. Bell, Deputy Engineering Director
G. Boerschig, Engineering Manager
N. Chrissotimos, Regulatory Assurance
R. Chrzanowski, Nuclear Oversight Manager
T. Fuhs, Regulatory Assurance
R. Green, Work Control
C. Iben, Operations
M. Perito, Maintenance Manager
B. Porter, Design Engineering Manager
T. Schiffley, Design Engineering - Downers Grove
S. Short, Electrical Engineering Supervisor
J. Sirovy, Nuclear Oversight Assessor
T. Tulon, Site Vice-President
M. Wagner, Regulatory Assurance

Illinois Department of Nuclear Safety (IDNS)

R. Ganser, Resident Inspector

LIST OF ACRONYMS USED

ADAMS	Agencywide Documents Access and Management System
CFR	Code of Federal Regulations
CR	Condition Report
DBA	Design Basis Accident
dc	Direct Current
DCP	Design Change Package
DPR	Demonstration Power Reactor
DRS	Division of Reactor Safety
EDG	Emergency Diesel Generator
EOP	Emergency Operating Procedure
EQ	Environmental Qualification
HPCI	High Pressure Coolant Injection
IDNS	Illinois Department of Nuclear Safety
IMC	Inspection Manual Chapter
IPE	Individual Plant Evaluation
IR	Inspection Report
LLC	Limited Liability Company
LT	Level Transmitter
MCC	Motor Control Center
MOD	Modification
MOV	Motor Operated Valve
MSIV	Main Steam Isolation Valve
NO	Nuclear Oversight
NRC	United States Nuclear Regulatory Commission
NRR	Office of Nuclear Reactor Regulation
NWR	Nuclear Work Request
OA	Other Activities
OAD	Operational Analysis Division
OPRM	Oscillation Power Range Monitor
PARS	Publicly Available Records
PRA	Probabilistic Risk Analysis
QA	Quality Assurance
QC	Quality Control
RPS	Reactor Protection System
RWCU	Reactor Water Cleanup
RHR	Residual Heat Removal
SDP	Significance Determination Process
SQUG	Seismic Qualification Utility Group
SSC	Structure, System, or Component
TS	Technical Specification
UFSAR	Updated Final Safety Analysis Report
V	Volt

LIST OF DOCUMENTS REVIEWED

The following is a list of licensee documents reviewed during the inspection, including documents prepared by others for the licensee. Inclusion on this list does not imply that NRC inspectors reviewed the documents in their entirety, but rather that selected sections or portions of the documents were evaluated as part of the overall inspection effort. Inclusion of a document in this list does not imply NRC acceptance of the document, unless specifically stated in the inspection report.

CONDITION REPORTS GENERATED PRIOR TO INSPECTION

<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
Q2000-02335	Evaluation of 125 Vdc Load Shedding	06/26/2000
Q2000-02363	Bus 14-1, Cub 1 Wiring Discrepancy	06/28/2000
Q2000-02506	Untimely Corrective Action for HPCI Turning Gear Solenoid	07/10/2000
Q2000-02524	Nuclear Oversight Identified Outdated Information in Multiple Safety Evaluations	07/07/2000
Q2000-02876	HPCI EOP Modification DCP 9900079	08/11/2000
Q2000-03242	Engineer Performed 10 CFR 50.59 Review Without Proper Qualification	09/08/2000
Q2000-03301	QC/NO Identified Scaffolding Rework	09/18/2000
Q2000-03528	HPCI Relay 2-2330-116B, Contacts 7&8 Did Not Close with Relay	10/06/2000
Q2000-03568	10 CFR 50.59 Focus Self-Assessment Identifies Issues with Certain Station Procedures	10/10/2000
Q2000-03641	OPRM Jumper Installation ½ Scram	10/14/2000
Q2000-03663	10 CFR 50.59 Inadequate for Use of RPS Test Box as Instructed in OPRM NWR's	10/14/2000
Q2000-03924	Post-Restoration Testing for DCP9900101/T-Mod Not Complete	10/26/2000
Q2000-03982	Different Tolerance Band in Work Package vs Mod Package Specified Tolerance	10/28/2000
Q2000-04104	1-3506-A/B/C Valve Capacities Undersized Due to Vendor Design/Manufacturing Error	11/11/2000
Q2001-00469	No Identified Inadequate 10 CFR 50.59 Evaluation	02/12/2001
Q2001-00954	No Identified Concerns with Calculation QDC-6600-S-0722	05/05/2000
Q2001-00955	Work Request Coded as Mod Without Going Through Mod Process	03/26/2001

CONDITION REPORTS GENERATED PRIOR TO INSPECTION

<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
Q2001-01034	Incorrect Parts Specified for SQUG Electrical Project on Bills of Material	12/13/2000

CORRECTIVE ACTION/TRACKING DOCUMENTS GENERATED DURING INSPECTION

<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
AT50673	Action Tracking Item - Provide Startup Rod Block Procedure Changes to NRC Residents for Review	04/27/2001
Q2001-01260	Condition Report - DCP 9900238 and 9900239 Modification Testing Changed but Not Documented on CC-AA-107	04/27/2001

DRAWINGS

<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
4E-1359B	Electro-Hyd Control System Schematic Diagram Alarm & Trip Part 4	L
4E-1420B	Schematic Diagram Recirculation Pump 1B Motor Operated Valves, Sheet 1	AA & AB
4E-1575AX	Schematic Diagram Main Control Room Annunciator Panel 901-6 Part 1 of 3	F
4E-1681J	Wiring & Schematic Diagram Reactor Building Essential Service 480V MCC 18/19-5 Part 3	AM & AN
M-373	Diagram of Drywell Ventilation System	H
M-399	Drywell Cooling System Plans and Section	B

MODIFICATIONS:

<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
9300330	Add 2-out-of-2 Logic Trip for Reactor Feed Pump Suction Pressure	11/06/2000
9800239	Modify HPCI Logic So That it Restarts at -59 after Hi Lvl	10/10/2000
9800267	Install Emergency Lighting in Cable Spreading Room	01/10/2001
9800284	Replace Level Transmitter for LT-2 646A/B Narrow Range Level	02/07/2000
9800299	Replace/Install EQ Cable for the 1-0202-5B	11/06/2000

MODIFICATIONS:

<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
9900032	Install Permanent Locking Device on Dampers 2A, 2C, 2F, and 2G	10/30/2000
9900275	Revise the Unit 1 Diesel Auto-Start Logic to Time Delay	09/27/2000
9900335	Modify Floor Drains in Condensate Pump Rooms, Unit 1	05/09/2000
9900419	Modify Rod Block Circuit to Facilitate Concurrent Testing	10/23/2000
ER9900343	Evaluate New Strainer for Use	08/12/1999

PROCEDURES

<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
CC-AA-10	Configuration Control Process Description	0
CC-AA-102	Design Impact Screening	1
CC-AA-103	Design Change Package	0
CC-AA-104	Configuration Change Closeout	1
CC-AA-106	Performance of Walkdowns and Control of Walkdown Information	0
CC-AA-107	Design Change Acceptance Testing Criteria	0
CC-AA-110	Field Change Requests	1
CC-AA-112	Temporary Modifications	2
CC-AA-309	Control of Design Analyses	0
LS-AA-999	10 CFR 50.59 Safety Evaluation Process	0
LS-AA-104	Exelon 10 CFR 50.59 Review Process	2
NES-G-14	Calculations	1
NES-G-16	Guidance for Determining Design Change Package (DCP) Applicability, Development and Implementation	1
QCOA 0010-16	Flood Emergency Procedure	12/28/1999
QCAP 1100-9	Conduct of 10 CFR 50.59 Safety Screening and Evaluations	0
RS-AA-107	UFSAR and Fire Protection Report Update Procedure	0

REFERENCES

<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
A/R 00048167	Work Request Coded as Mod Without Going Through Mod Process	03/27/2001
LS-AA-104-1000	Exelon 10 CFR 50.59 Resource Manual	03/08/2001
UFSAR-99-R6-044	Revise Method to Cope With External Flood	01/21/2000
UFSAR-99-R6-164	Revise UFSAR Section 3.4.1.1, External Flood Protection Measures	03/22/2001

SAFETY EVALUATIONS

<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
SE-99-001	A and B RHR Logic Test During Non-Outage	01/11/1999
SE-99-004	Replace RHR Pressure Switches 2-1001-74A	01/21/1999
SE-99-029	RWCU High Area Temperature Automatic Isolation	04/26/1999
SE-99-102	Revise Method to Cope With External Flood	11/12/1999
SE-00-010	Diesel Generator Endurance and Margin/Full Load Reject/Hot Restart Test	01/15/2000
SE-00-018	Unit 1 Diesel Auto-Start Logic Time Delay Change	06/05/2000
SE-00-023	Modify Closing Logic for MOV 2-2301-8	02/03/2000
SE-00-043	Use of Sparge Air to Purge the Offgas System	05/26/2000
SE-00-070	Loss of Safety Related 250 Vdc Battery Chargers Concurrent with DBA	08/29/2000
SE-00-105	UFSAR Change to Implement Changes to Address Issues in DBI Review	12/14/2000

SAFETY SCREENINGS

<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
QC-S-2001-0045	Alternate Decay Heat Removal	04/15/2001
QC-V-2001-0002	Current Limit Check for Safety Related 125 Vdc and 250 Vdc Chargers	04/03/2001
SS-F-99-0061	Revise Level Switch Setpoints for EDG Fuel Oil Tanks Unit 1 and 0	03/26/1999

SAFETY SCREENINGS

<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
SS-F-00-0012	Installation of OPRM Instrumentation - Phase 1 RPS Bypass Jumpers	01/14/2000
SS-F-00-0021	Main Generator Output Breaker Logic Contact	01/22/2000
SS-F-00-0051	Mod. Flood Drains Cond. Pump Area	03/06/2000
SS-F-00-0093	Installation of Em. Lighting Cable Spreading Room	04/14/2000
SS-F-00-0140	HPCI Area Temperature Switch	06/06/2000
SS-F-00-0150	Temperature Vent and Scaffolding Unit 2 Battery Room	06/24/2000
SS-F-00-0223	Setpoint Change for Loss of Voltage Relay Setpoint for Buses 13-1, 14-1, 23-1 and 24-1	09/22/2000
SS-F-00-0230	MSIV Closure and Turbine Stop Valve Closure Limit Switch Setpoints	11/10/2000
SS-P-99-0308	Unit 2 Division II Core Spray Injection Valve Interlock Logic Test	02/24/1999
SS-P-00-0175	Unique Test for DCP 9900120	02/05/2000
SS-P-01-0170	RHR Service Water Operation Using Loop A Cross-Tie Header RHR Service Operation Using Loop B Cross-tie Header	03/05/2001
SS-U-00-0016	Incorporate Clarifying Information on 125 and 250 Vdc Battery Chargers	12/15/2000

TEMPORARY ALTERATIONS

<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
97-2-105	Wire Open the 2A, 2C, 2F, and 2G Drywell Discharge Dampers	10/06/1997
97-2-111	Wire Open Drywell Discharge Damper No. 2-C-5772-23	12/10/1997

WORK REQUESTS

<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
990137184-01	OAD to Test U-2 Main Generator Output Breakers Logic Contact Verification	02/07/2000

LIST OF INFORMATION REQUESTED

INITIAL DOCUMENT REQUEST

(Sent to Resident/Licensee on 4/5/01)

I. Information Needed for in Office Preparation Week

The following information is needed by Friday, April 13, 2001, or sooner, to facilitate the selection of items to be reviewed during the onsite inspection week (April 23-27, 2001). The team will select specific items from the information requested below and submit the selected items to your staff during the week before the onsite inspection.

We request that the specific items selected from the lists be available and ready for review on the first day of inspection April 23, 2001.

a. Permanent Plant Modifications

- (1) List of permanent plant modifications to risk significant SSCs involving: (a) permanent plant changes; (b) design changes; (c) set point changes; (d) equivalency evaluations; (e) suitability analyses; (f) calculations; (g) commercial grade dedications.¹
- (2) List of condition reports (open and closed) issued to address plant permanent modification issues/concerns.¹
- (3) Copy of modification procedure(s) and post modification testing procedure.

b. Changes, Tests, or Experiments

- (1) List of all 10 CFR 50.59 completed evaluations involving: (a) changes to facility (modifications); (b) procedure revisions; (c) tests or non-routine operating configurations; (d) changes to the UFSAR; (e) calculation.¹
- (2) List of all 10 CFR 50.59 screenings that have been screened out as not requiring a full evaluation involving: (a) changes to facility (modifications); (b) procedure revisions; (c) tests or non-routine operating configurations; (d) changes to the UFSAR; (e) calculations.¹
- (3) List of condition reports generated because of problems associated with 10 CFR 50.59 evaluations.¹
- (4) Copies of procedures that specify how 10 CFR 50.59 evaluations and screenings are performed.

¹ Provide information requested going back two years.

LIST OF INFORMATION REQUESTED

- (5) Copies of procedures that delineate how 10 CFR 50.59 UFSAR updates are prepared by engineers or staff and how the licensee submits 10 CFR 50.59 UFSAR updates.
- (6) List of special tests or experiments and non-routine operating configurations in the last two years (if any).

C. General Information

- (1) List of procedure changes. ¹
- (2) List of calculation revisions. ¹
- (3) List of setpoint changes. ¹
- (4) List of equivalency evaluations. ¹
- (5) List of suitability analyses. ¹
- (6) List of commercial grade dedications. ¹
- (7) List of Temporary Modifications
- (8) Latest Engineering Organization Chart

¹ Provide information requested going back two years.

LIST OF INFORMATION REQUESTED

II. Information Request to be Available on First Day of Inspection (April 23, 2000)

a. We request that the following information be available to the team once they arrive onsite. Copies of these documents do not need to be solely available to the team as long as the inspectors have ready access to them.

- Updated Final Safety Analysis Report
- Technical Specifications (TS)
- Latest IPE/PRA report
- Vendor manuals
- Equipment qualification binders
- The latest 10 CFR 50.59 UFSAR update submittal

b. Please provide copies of the following documents:

- Copies of inspector selected sample of permanent plant modifications and design changes, set point changes, procedure changes, equivalency evaluations, suitability analyses, calculations, and commercial grade dedications (list of selected items will be provided to licensee by Wednesday April 18, 2000).
- Provide copies of QA audits, self-assessments, and outside organization audits conducted in the areas of permanent plant modifications and 10 CFR Part 50.59 evaluations and screenings. Also include corrective action documentation/status of identified findings (last 2 years).
- Copies of any self-assessments and associated Condition Reports generated in preparation for the inspection.
- Copies of any CRs generated as a result of the team's findings during this inspection.
- Copies of the list of questions submitted by the team members and the status/resolution of the information requested (provide daily during the inspection to each team member).

NOTE: If you have any questions regarding the requested information please contact George Hausman at NRC Region III, (630) 829-9743.