

March 21, 1984

D(Rec'd)

Dockets Nos. 50-277
and 50-278

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Mr. Edward G. Bauer, Jr.
Vice President and General Counsel
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

The Commission has issued the enclosed Amendments Nos. 95 and 97 to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. These amendments consist of changes to the Technical Specifications (TSs) in response to your request dated April 4, 1983.

These amendments provide clarification of an isolation trip feature on the High Pressure Coolant Injection (HPCI) system. The change provides a statement in the Notes For Table 3.7.1 indicating that one of the HPCI steam line exhaust drain valves does not actuate upon receipt of one of the Group 4 isolation signals. Isolation of the HPCI steam exhaust drain line is, however, provided by existing isolation provisions that meet the provisions of Standard Review Plan 6.2.4 and TMI Action Plan II.E.4.2.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next monthly Federal Register notice.

Sincerely,

original signed by:
Gerald E. Gears, Project Manager
Operating Reactors Branch No. 4
Division of Licensing

Enclosures:

1. Amendment No. 95 to DPR-44
2. Amendment No. 97 to DPR-56
3. Safety Evaluation

ORB#4:DL
RIngram
03/9/84

ORB#4:DL
GGears;ef
03/12/84

ORB#4:DL
JStoltz
03/14/84

AD:OR
GLatinas
03/19/84

Conditional on staff checking with SEC prior to issuance for comments or that there are no such comments or petitions received.
J. G. Gray
03/15/84
done re 3/21/84

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Philadelphia Electric Company

cc w/enclosure(s):

Eugene J. Bradley
Philadelphia Electric Company
Assistant General Counsel
2301 Market Street
Philadelphia, Pennsylvania 19101

Troy B. Conner, Jr.
1747 Pennsylvania Avenue, N.W.
Washington, D. C. 20006

Thomas A. Deming, Esq.
Assistant Attorney General
Department of Natural Resources
Annapolis, Maryland 21401

Philadelphia Electric Company
ATTN: Mr. R. Fleishmann
Peach Bottom Atomic
Power Station
Delta, Pennsylvania 17314

Albert R. Steel, Chairman
Board of Supervisors
Peach Bottom Township
R. D. #1
Delta, Pennsylvania 17314

Allen R. Blough
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Peach Bottom Atomic Power Station
P. O. Box 399
Delta, Pennsylvania 17314

Mr. Thomas E. Murley, Regional Administrator
U. S. Nuclear Regulatory Commission, Region I
Office of Inspection and Enforcement
631 Park Avenue
King of Prussia, Pennsylvania 19406

Regional Radiation Representative
EPA Region III
Curtis Building (Sixth Floor)
6th and Walnut Streets
Philadelphia, Pennsylvania 19106

M. J. Cooney, Superintendent
Generation Division - Nuclear
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Mr. R. A. Heiss, Coordinator
Pennsylvania State Clearinghouse
Governor's Office of State Planning
and Development
P. O. Box 1323
Harrisburg, Pennsylvania 17120

Thomas M. Gerusky, Director
Bureau of Radiation Protection
Pennsylvania Department of
Environmental Resources
P. O. Box 2063
Harrisburg, Pennsylvania 17120



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 95
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated April 4, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

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Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 95, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch No. 4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 21, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 95

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

Remove

183

Insert

183

1. Reactor vessel low water level.
2. High drywell pressure.
3. Reactor building ventilation exhaust high radiation.
4. Refuel floor ventilation exhaust high radiation.

GROUP 4: The valves in Group 4 are actuated by any one of the following conditions:

1. HPCI steam line high flow.
2. HPCI steam line space high temperature.
3. HPCI steam line low pressure. (except for HPCI steam line exhaust drain valve AO-4247)

GROUP 4A: The valves in Group 4A are actuated by either of the following conditions:

1. Reactor vessel low-low water level.
2. High drywell pressure.

GROUP 4B: The valve in Group 4B is actuated when both of the following conditions are present:

1. High drywell pressure.
2. HPCI steam line low pressure.

GROUP 5: The valves in Group 5 are actuated by any one of the following conditions:

1. RCIC steam line high flow.
2. RCIC steam line space high temperature.
3. RCIC steam line low pressure.

GROUP 5A: The valves in Group 5A are actuated by the following condition:

1. Reactor vessel low-low water level.

GROUP 5B: The valve in Group 5B is actuated when both of the following conditions are present:

1. High drywell pressure.
2. RCIC steam line low pressure.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 97
License No. DPR-56

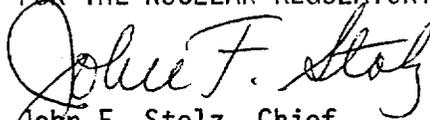
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated April 4, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 97, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch No. 4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 21, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 97

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

Remove

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Insert

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1. Reactor vessel low water level.
2. High drywell pressure.
3. Reactor building ventilation exhaust high radiation.
4. Refuel floor ventilation exhaust high radiation.

GROUP 4 : The valves in Group 4 are actuated by any one of the following conditions:

1. HPCI steam line high flow.
2. HPCI steam line space high temperature.
3. HPCI steam line low pressure. (except for HPCI steam line exhaust drain valve AO-5247)

GROUP 4A: The valves in Group 4A are actuated by either of the following conditions:

1. Reactor vessel low-low water level.
2. High drywell pressure.

GROUP 4B: The valve in Group 4B is actuated when both of the following conditions are present:

1. High drywell pressure.
2. HPCI steam line low pressure.

GROUP 5 : The valves in Group 5 are actuated by any one of the following conditions:

1. RCIC steam line high flow.
2. RCIC steam line space high temperature.
3. RCIC steam line low pressure.

GROUP 5A: The valves in Group 5A are actuated by the following condition:

1. Reactor vessel low-low water level.

GROUP 5B: The valve in Group 5B is actuated when both of the following conditions are present:

1. High drywell pressure.
2. RCIC steam line low pressure.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENTS NOS. 95 AND 97 TO FACILITY OPERATING LICENSES

NOS. DPR-44 AND DPR-56

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNITS NOS. 2 AND 3

DOCKETS NOS. 50-277 AND 50-278

Introduction

By letter dated April 4, 1983, the Philadelphia Electric Company (the licensee) proposed certain changes to the Technical Specifications for the Peach Bottom Atomic Power Station, Units 2 and 3.

The two inch High Pressure Coolant Injection (HPCI) turbine exhaust drain line is provided with redundant isolation valves (4247 and 4248 for Peach Bottom, Unit 2, and 5247 and 5248 for Peach Bottom, Unit 3). The licensee stated that the current Technical Specifications (Table 3.7.1, page 181) regarding the isolation signals to isolate the HPCI turbine exhaust drain line include: 1) high HPCI steam flow; 2) high temperature along the HPCI steam line or in the HPCI equipment room; and 3) HPCI low steam line pressure.

The licensee states that a review of the as-built configuration shows that valve 4248 (5248) receives all of the three isolation signals listed above; however, valve 4247 (5247) receives only signals 1 and 2, above. The licensee has requested that Table 3.7.1 be modified to reflect the actual isolation signals provided to each valve (See attached Figure 1).

Evaluation

The licensee states in their April 4, 1983 submittal that the absence of the low steam line pressure signal from the set of isolation signals for one of the two valves does not compromise the diversity requirement. The diversity requirement is satisfied by the steam line high flow and HPCI steam line space high temperature, which would be generated as a result of HPCI steam line break. The licensee further states that shutdown of the system after it has performed its safety-related function is not affected by the absence of the low steam line pressure signal from the set of isolation signals for one of the two valves. This is due to the redundant isolation valve on the line connecting the turbine exhaust line with the gas side of the suppression chamber, which does receive a HPCI steam line low pressure signal. On the parallel line, the suppression pool water provides a redundant isolation barrier. In this regard, we agree with the licensee's analysis that the HPCI steam line low pressure signal is a process signal and need not be provided

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for valve 4247 (5247). The basis for this conclusion is that reasonable isolation provisions that meet the provisions of Standard Review Plan 6.2.4 and TMI Action Plan II.E.4.2 are provided by the existing isolation signals and by the redundant barriers as explained above.

We conclude, based on the above considerations, that the proposed change to the Technical Specifications is acceptable.

Environmental Considerations

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 21, 1984

The following NRC personnel have contributed to this Safety Evaluation:
Farouk Eltawila.

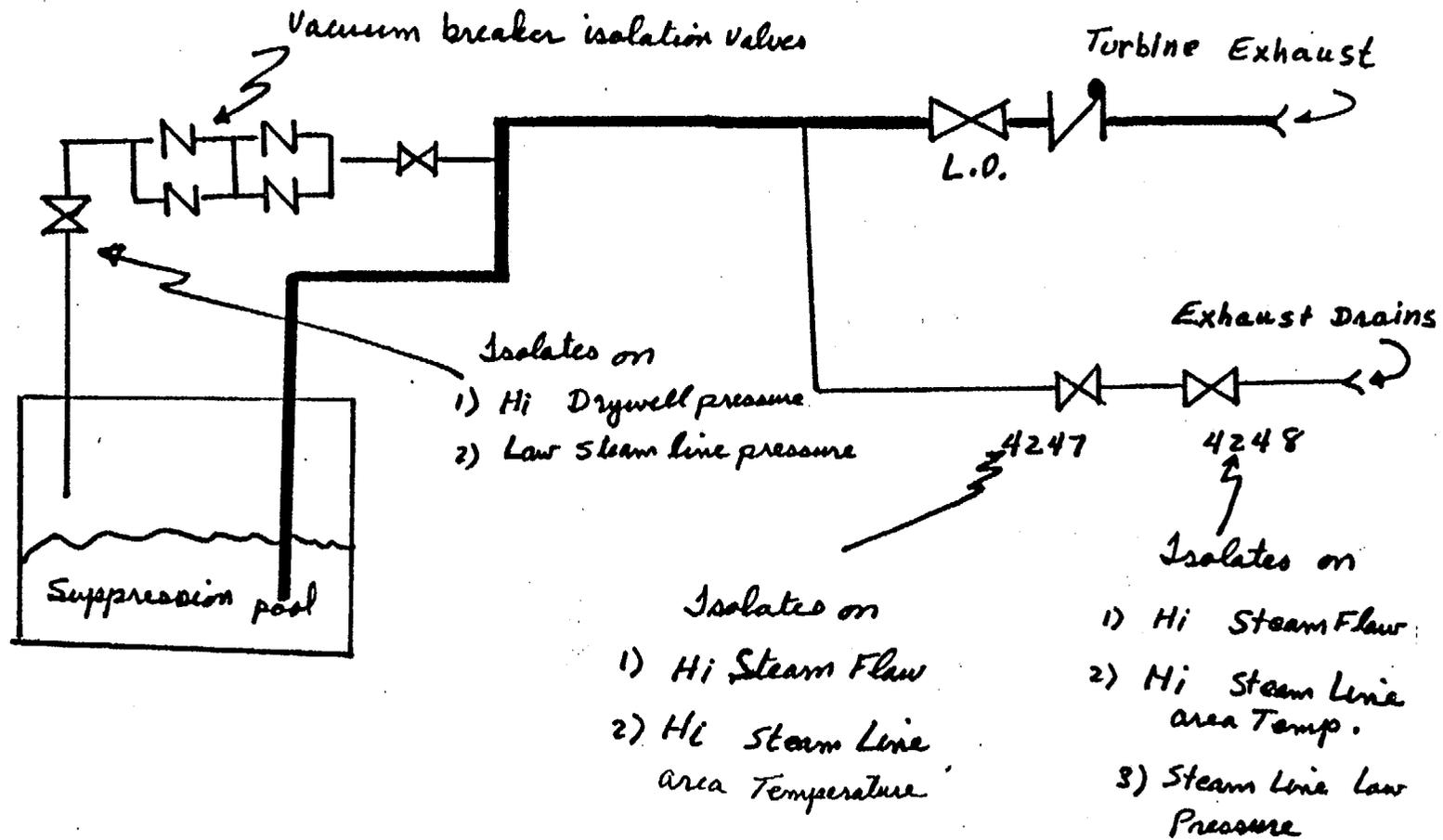


Figure 1. Valve Arrangement For HPCI Turbine Exhaust System