March 28, 1984

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Dockets Nos. 50-277
and 50-278

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Mr. Edward G. Bauer, Jr.
Vice President and General Counsel
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

SUBJECT: TECHNICAL SPECIFICATION AMENDMENTS IN RESPONSE TO AN AMENDMENT REQUEST DATED MARCH 11, 1983

The Commission has issued the enclosed Amendments Nos. 96 and 98 , to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated March 11, 1983.

The changes to the TSs permit a revision of the definition of "Primary Containment Integrity", deletions of obsolete references, correction of an error in reference to two smoke detectors, and revisions of the requirements for administrative control of high radiation area access keys.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next monthly Federal Register notice.

Sincerely,

"ORIGINAL SIGNED BY:"
Gerald E. Gears, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:
1. Amendment No. 96 to DPR-44
2. Amendment No. 98 to DPR-56
3. Safety Evaluation

cc w/enclosures:
See next page

| ORB#4:DL | ORB#4:DL | ORB#4:DL | AD:OR:DL | OELD |
|---|---|---|---|---|
| RIngram | GGears;cf | JStolz | GLainas | J.GRAY |
| 3/ 6 /84 | 3/15 /84 | 3/16/84 | 3/ /84 | 3/20 /84 |

Philadelphia Electric Company

cc w/enclosure(s):

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 96
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated March 11, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 96, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 28, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 96

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

| Remove | Insert |
|---|---|
| 4 | 4 |
| 67 | 67 |
| 240m | 240m |
| 240n | 240n |
| 262 | 262 |

1.0 DEFINITIONS (cont'd)

component, or device to perform its function are also capable of performing their related support function.

Operating - Operating means that a system or component is performing its intended functions in its required manner.

Operating Cycle - Interval between the end of one refueling outage for a particular unit and the end of the next subsequent refueling outage for the same unit.

Primary Containment Integrity - Primary containment integrity means that the drywell and pressure suppression chamber are intact and all of the following conditions are satisfied:

1. All primary containment penetrations required to be closed during accident conditions are either:

   a) Capable of being closed by an OPERABLE containment automatic isolation valve system, or

   b) Closed by at least one manual valve, blind flange, or deactivated automatic valve secured in its closed position, except as may be provided in Specifications 3.7.D.2 and 4.7.D.2. Manual valves may be opened to perform necessary operational activities.

2. At least one door in each airlock is closed and sealed.

3. All blind flanges and manways are closed.

Protective Action - An action initiated by the protection system when a limit is reached. A protective action can be at a channel or system level.

Protective Function - A system protective action which results from the protective action of the channels monitoring a particular plant condition.

Rated Power - Rated power refers to operation at a reactor power of 3,293 MWt; this is also termed 100 percent power and is the maximum power level authorized by the operating license. Rated steam flow, rated coolant flow, rated neutron flux, and rated nuclear system pressure refer to the values of these parameters when the reactor is at rated power.

TABLE 3.2.B (CONTINUED)

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT COOLING SYSTEMS

| Minimum No. Of Operable Instrument Channels Per Trip System(1) | Trip Function | Trip Level Setting | Number of Instrument Channels Provided by Design | Remarks |
|---|---|---|---|---|
| 2 | Core Spray Pump Start Timer | 6 ± 1 sec<br>10 ± 1 sec | 4 timers<br>4 timers | In conjunction with loss of power initiates the starting of CSCS pumps. |
| 2 | LPCI Pump Start Timer (Two Pumps) | 5 ± 1 sec | 4 timers | |
| 1 | Auto Blowdown Timer | 90 ≤ t ≤ 120 | 2 timers | In conjunction with Low Reactor Water Level, High Drywell Pressure and LPCI or Core Spray Pump running interlock, initiates Auto Blowdown. |
| 2 | RHR (LPCI) Pump Discharge Pressure Interlock | 50 ± 10 psig | 4 channels | Defers ADS actuation pending confirmation of Low Pressure Core Cooling system operation (LPCI Pump running interlock.) |
| 2 | Core Spray Pump Discharge Pressure Interlock | 185 ± 10 psig | 4 channels | Defers ADS actuation pending confirmation of Low Pressure Core Cooling system operation (Core Spray Pump running interlock.) |

TABLE 3.14.C.1

FIRE DETECTORS

| Location | Detector Type/ Designation(1) | Minimum Detectors Operable |
|---|---|---|
| UNIT 2 | | |
| Primary Containment (2)(3) | S1, S2, S8 | 3 |
| CRD Area (135') Rms. 208, 209, 212 | S7A, S8A, S9A, S10A S11A, S12A, S13A, S14A S15A, S16A, S17A, S18A S19A, S20A | 13 |
| Isol. Valve Compt. (135') Rm. 204 | S21A | 1 |
| Operating Area (165') Rm. 402, 403 | S31A, S32A, S33A, S34A S35A, S36A, S37A, S38A S39A, S40A, S41A, S42A S43A | 12 |
| Laydown Area (195') Rm. 501, 502 508 | S45A, S46A, S47A, S48A S49A, S50A, S51A, S52A | 7 |
| Vent. Equip. Area (195') Rm. 506 | S53A, S54A | 2 |
| Vent Stack Rad. Mon.-Refuel floor (234') | S58A, S59A | 2 |
| HPCI Room | S78 H5, H6, H7 | 1 (See 3.14.B.1.c) |
| RCIC Room | S45, S46 | 2 |
| Reactor Bldg. Sump Area | S79 | 1 |
| Core Spray Pump Rooms | S41, S42, S43, S44 | 4 |
| Vac. Breaker Area-Rm. 107, 108 | S91, S92, S93 | 3 |
| RHR Rooms | | |
| Room 101 | S30, S31, S32 | 3 |
| Room 102 | S33, S34, S35 | 3 |
| Room 103 | S36, S37, S38 | 3 |
| Room 104 | S39, S40 | 2 |
| Torus Area | S83, S84, S85, S86 S87, S88, S89, S90 | 7 |
| M-G Set Lube Oil Rm (Rm 105) | S94, S95, S96, S97, S98 | 4 |



Amendment No. 39, 83, 85, 96

TABLE 3.14.C.1

FIRE DETECTORS

| Location | Detector Type/ Designation(1) | Minimum Detectors Operable |
|---|---|---|
| Recirc. Pump MG Set Room | S15, S16, S17<br>S18, S19, S20 | 5 |
| Emerg. Switchgear Rooms | S11, S12, S13, S14 | 4 |
| Battery Rooms<br>Room 219<br>Room 225 | <br>S70, S71<br>S68, S69 | <br>2<br>2 |
| 13KV Switchgear Area (116') | S72, S73, S74 | 3 |
| HPSW Pump Room | S390 | 1 |
| UNIT 3 | | |
| Primary Containment (2)(3) | S103, S104, S106 | 3 |
| CRD Area (135') Rms. 250 252, 257 | S166,S167,S168,S169<br>S170,S171,S172,S173<br>S174,S175,S176,S177<br>S178,S179 | 13 |
| Isol. Valve Compt. (135') Rm 249 | S181 | 1 |
| Operating Area (165')<br>Rm. 443, 444 | S182,S183,S184,S195<br>S186,S187,S188,S189<br>S190,S191,S192,S193<br>S194 | 12 |
| Laydown Area (195')<br>Rm. 517, 518, 523 | S196,S197,S198,S199<br>S103A,S104A,S105A,S106A | 7 |
| Vent. Equip Area (195')<br>Rm. 520 | S107A, S108A | 2 |
| Vent Stack Rad. Mon.-Refuel floor (234') | S109A, S110A | 2 |
| HPCI Room | S148<br>H115, H116, H117 | 1<br>(See 3.14.B.1.c) |
| RCIC Room | S131, S132 | 2 |
| Reactor Bldg. Sump Area | S149 | 1 |

6.13 High Radiation Area

6.13.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20:

a. Each High Radiation Area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

1. A radiation monitoring device which continuously indicates the radiation dose rate in the area.

2. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them.

3. An individual qualified in radiation protection procedures who is equipped with a radiation dose rate monitoring device. This individual shall be responsible for providing positive control over activities within the area and shall perform periodic radiation surveillance at the frequency specified by the plant Health Physicist or his designee on the Radiation Work Permit.

b. Each High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.13.1 (a) above. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Shift Superintendent, the Shift Supervisor or the Senior Health Physicist.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 98
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated March 11, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 98 , are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 28, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 98

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

| Remove | Insert |
|---|---|
| 4 | 4 |
| 67 | 67 |
| 240m | 240m |
| 240n | 240n |
| 262 | 262 |

1.0 DEFINITIONS (cont'd)

component, or device to perform its function are also capable of performing their related support function.

Operating - Operating means that a system or component is performing its intended functions in its required manner.

Operating Cycle - Interval between the end of one refueling outage for a particular unit and the end of the next subsequent refueling outage for the same unit.

Primary Containment Integrity - Primary containment integrity means that the drywell and pressure suppression chamber are intact and all of the following conditions are satisfied:

1. All primary containment penetrations required to be closed during accident conditions are either:

   a) Capable of being closed by an OPERABLE containment automatic isolation valve system, or

   b) Closed by at least one manual valve, blind flange, or deactivated automatic valve secured in its closed position, except as may be provided in Specifications 3.7.D.2 and 4.7.D.2. Manual valves may be opened to perform necessary operational activities.

2. At least one door in each airlock is closed and sealed.

3. All blind flanges and manways are closed.

Protective Action - An action initiated by the protection system when a limit is reached. A protective action can be at a channel or system level.

Protective Function - A system protective action which results from the protective action of the channels monitoring a particular plant condition.

Rated Power - Rated power refers to operation at a reactor power of 3,293 MWt; this is also termed 100 percent power and is the maximum power level authorized by the operating license. Rated steam flow, rated coolant flow, rated neutron flux, and rated nuclear system pressure refer to the values of these parameters when the reactor is at rated power.

TABLE 3.2.B (CONTINUED)

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT COOLING SYSTEMS

| Minimum No. Of Operable Instrument Channels Per Trip System(1) | Trip Function | Trip Level Setting | Number of Instrument Channels Provided by Design | Remarks |
|---|---|---|---|---|
| 2 | Core Spray Pump Start Timer | 6 ± 1 sec<br>10 ± 1 sec | 4 timers<br>4 timers | In conjunction with loss of power initiates the starting of CSCS pumps. |
| 2 | LPCI Pump Start Timer (Two Pumps) | 5 ± 1 sec | 4 timers | |
| 1 | Auto Blowdown Timer | 90 ≤ t ≤ 120 | 2 timers | In conjunction with Low Reactor Water Level, High Drywell Pressure and LPCI or Core Spray Pump running interlock, initiates Auto Blowdown. |
| 2 | RHR (LPCI) Pump Discharge Pressure Interlock | 50 ± 10 psig | 4 channels | Defers ADS actuation pending confirmation of Low Pressure Core Cooling system operation (LPCI Pump running interlock.) |
| 2 | Core Spray Pump Discharge Pressure Interlock | 185 ± 10 psig | 4 channels | Defers ADS actuation pending confirmation of Low Pressure Core Cooling system operation (Core Spray Pump running interlock.) |

TABLE 3.14.C.1

FIRE DETECTORS

| Location | Detector Type/ Designation(1) | Minimum Detectors Operable |
|---|---|---|
| UNIT 2 | | |
| Primary Containment (2)(3) | S1, S2, S8 | 3 |
| CRD Area (135') Rms. 208, 209, 212 | S7A, S8A, S9A, S10A S11A, S12A, S13A, S14A S15A, S16A, S17A, S18A S19A, S20A | 13 |
| Isol. Valve Compt.(135')Rm.204 | S21A | 1 |
| Operating Area(165')Rm.402,403 | S31A, S32A, S33A, S34A S35A, S36A, S37A, S38A S39A, S40A, S41A, S42A S43A | 12 |
| Laydown Area(195')Rm.501,502 508 | S45A, S46A, S47A, S48A S49A, S50A, S51A, S52A | 7 |
| Vent.Equip.Area(195')Rm.506 | S53A, S54A | 2 |
| Vent Stack Rad. Mon.-Refuel floor (234') | S58A, S59A | 2 |
| HPCI Room | S78<br>H5, H6, H7 | 1<br>(See 3.14.B.1.c) |
| RCIC Room | S45, S46 | 2 |
| Reactor Bldg. Sump Area | S79 | 1 |
| Core Spray Pump Rooms | S41, S42, S43, S44 | 4 |
| Vac. Breaker Area-Rm. 107,108 | S91, S92, S93 | 3 |
| RHR Rooms | | |
| Room 101 | S30, S31, S32 | 3 |
| Room 102 | S33, S34, S35 | 3 |
| Room 103 | S36, S37, S38 | 3 |
| Room 104 | S39, S40 | 2 |
| Torus Area | S83,S84,S85,S86 S87,S88,S89,S90 | 7 |
| M-G Set Lube Oil Rm (Rm 105) | S94,S95,S96,S97,S98 | 4 |

Table 3.T4.C.1

FIRE DETECTORS

| Location | Detector Type/ Designation(1) | Minimum Detectors Operable |
|---|---|---|
| Recirc. Pump MG Set Room | S15, S16, S17 S18, S19, S20 | 5 |
| Emerg. Switchgear Rooms | S11, S12, S13, S14 | 4 |
| Battery Rooms | | |
| Room 218 | S70, S71 | 2 |
| Room 225 | S68, S69 | 2 |
| 13KV Switchgear Area (116') | S72, S73, S74 | 3 |
| HPSW Pump Room | S390 | 1 |
| UNIT 3 | | |
| Primary Containment (2)(3) | S103, S104, S106 | 3 |
| CRD Area (135') Rms. 250 252, 257 | S166,S167,S168,S169 S170,S171,S172,S173 S174,S175,S176,S177 S178,S179 | 13 |
| Isol. Valve Compt. (135') Rm 249 | S181 | 1 |
| Operating Area (165') Rm. 443, 444 | S182,S183,S184,S185 S186,S187,S188,S189 S190,S191,S192,S193 S194 | 12 |
| Laydown Area (195') Rm. 517, 518, 523 | S196,S197,S198,S199 S103A,S104A,S105A,S106A | 7 |
| Vent. Equip Area (195') Rm. 520 | S107A, S108A | 2 |
| Vent Stack Rad. Mon.-Refuel floor (234') | S109A, S110A | 2 |
| HPCI Room | S148 H115, H116, H117 | 1 (See 3.14.B.1.c) |
| RCIC Room | S131, S132 | 2 |
| Reactor Bldg. Sump Area | S149 | 1 |

6.13 High Radiation Area

6.13.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20:

a. Each High Radiation Area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

1. A radiation monitoring device which continuously indicates the radiation dose rate in the area.

2. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them.

3. An individual qualified in radiation protection procedures who is equipped with a radiation dose rate monitoring device. This individual shall be responsible for providing positive control over activities within the area and shall perform periodic radiation surveillance at the frequency specified by the plant Health Physicist or his designee on the Radiation Work Permit.

b. Each High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.13.1 (a) above. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Shift Superintendent, the Shift Supervisor or the Senior Health Physicist.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION SUPPORTING

AMENDMENTS NOS. 96 AND 98 TO FACILITY OPERATING LICENSES NOS. DPR-44 AND DPR-56

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNITS NOS. 2 AND 3

DOCKETS NOS. 50-277 AND 50-278

## INTRODUCTION

By letter dated March 11, 1983, the Philadelphia Electric Company (PECO or the licensee), made application to amend the Technical Specifications (TSs) for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3, to permit a revision of the definition of "Primary Containment Integrity", deletions of obsolete references, correction of an error in reference to two smoke detectors, and revisions of the requirements for administrative control of high radiation area access keys.

## EVALUATION

The licensee requested in its March 11, 1983, submittal that their current Technical Specifications be amended to incorporate four changes.

The first change would modify the definition of "Primary Containment Integrity" to permit satisfying the condition of primary containment penetration closure by one of two conditions: closure by an operable automatic containment isolation valve system or closure by at least one manual valve, blind flange, or deactivated automatic valve secured in its closed position. The current TS definition requires that both conditions be satisfied. However, a Limiting Condition for Operation in Section 3.7.D.2 of the licensee's Peach Bottom Technical Specifications permits the use of manual isolation valves secured in the closed position when automatic isolation valves are inoperable or cannot be shown to be operable. The proposed revision would revise the definition to make it consistent with this current Limiting Condition for Operation. The proposed definition for "Primary Containment Integrity" closely follows the Standard Technical Specifications for Boiling Water Reactors (NUREG-0123, Rev. 3) by permitting two means to satisfy the closure requirement for primary containment penetrations required to be closed during accident conditions. Therefore, we conclude that the above revision to Section 1.0 (Definitions) of the Peach Bottom TSs is acceptable.

The second proposed change involves deletion of an entry in Table 3.2.B pertaining to modifications to the Low Pressure Coolant Injection (LPCI) Pump Start Timers which have been completed. Since these LPCI Pump Start Timer modifications have been completed, references in the "Remarks" column of Table 3.2.B to the replaced timers are now obsolete. Therefore, we conclude that deletion of this obsolete entry is acceptable.

The third requested change involves deletion of references to two smoke detectors from Table 3.14.C.1. The licensee indicated that as a result of a field survey and reevaluation conducted during its 10 CFR 50, Appendix R (Fire Protection Program) review, it was determined that the necessary Appendix R safe shutdown equipment was physically located in an area already protected by a smoke detection system. Therefore, these smoke detectors were deleted from the licensee's final design of its smoke detection system and subsequently removed from service. However, the final design was not reflected in the licensee's application for amendment dated February 18, 1982. This amendment request was subsequently approved by the Commission with these two detectors added to Table 3.14.C.1.

We conclude that the requested deletion of references in Table 3.14.C.1 to smoke detectors S22A and S180 is acceptable because these detectors, even if in service, do not protect areas containing safe shutdown components or safety-related systems or components as specified in 10 CFR 50, Appendix R, and therefore are not required.

However, we wish to point out that we have not completed our final review and inspection of the licensee's Appendix R program. Subsequent staff conclusions may require additional revisions to the licensee's Technical Specifications, including addition and/or deletion of smoke detectors.

Finally, the fourth proposed change extends the administrative control of all keys necessary to control access to high radiation areas from the Shift Superintendent and Shift Supervisor to the Senior Health Physicist. This transfer of control and authority over high radiation areas to the Senior Health Physicist provides better overall control of access to high radiation areas because this individual has a better overall knowledge and current status of all radiation levels throughout the plant. This change is in accordance with the Boiling Water Reactor Standard Technical Specifications (NUREG-0123, Rev. 3, Section 6.12.2) concerning high radiation areas. Therefore, based upon the above, we conclude that this change is acceptable.

ENVIRONMENTAL CONSIDERATIONS

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 28, 1984

The following NRC personnel have contributed to this Safety Evaluation: Gerry Gears.