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Dockets Nos. 50-277  
and 50-278

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DIVISION OF LICENSING SERVICES

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RECEIVED DISTRIBUTION SERVICES UNIT

Mr. Edward G. Bauer, Jr.  
Vice President and General Counsel  
Philadelphia Electric Company  
2301 Market Street  
Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

On September 22, 1980 we issued Amendments Nos. 73 and 72 to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. Included in those amendments was a revision to the Table of Containment Isolation Valves to reflect modifications made to satisfy Item 2.1.5.a of NUREG-0578 "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations". As stated in our letter transmitting those amendments we would advise you separately of our determination on the adequacy of your changes to meet our acceptance criteria.

We have completed our review of this issue and determined that the actions taken are acceptable. The basis for our conclusion is enclosed.

Sincerely,

Original signed by  
Robert W. Reid  
Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Enclosure:  
Safety Evaluation

cc w/enclosure: See next page

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OFFICE	ORB#4:DL	C-ORB#4:DL			
SURNAME	DVerrelli/cb	RReid			
DATE	10/2/80	10/29/80			



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

October 29, 1980

Dockets Nos. 50-277  
and 50-278

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Sincerely,

A handwritten signature in cursive script that reads "Robert W. Reid".

Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Enclosure:  
Safety Evaluation

cc w/enclosure: See next page

8011170421

Philadelphia Electric Company

cc w/enclosure(s):

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNITS NOS. 2 AND 3

DOCKETS NOS. 50-277 AND 50-278

I. INTRODUCTION

By letter dated July 16, 1980, Philadelphia Electric Company (licensee) requested amendments to Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station Units Nos. 2 and 3. The proposed amendments involve a revision to the Technical Specifications to incorporate into the Table of Containment Isolation Valves those modifications performed to satisfy our requirements of 2.1.5.a of NUREG-0578 "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations".

On September 22, 1980 we issued Amendments Nos. 73 and 72 to Licenses Nos. DPR-44 and DPR-56 which conformed the Technical Specifications with modifications we requested. The adequacy of the licensee's actions to meet our acceptance criteria is evaluated below.

II. EVALUATION

The licensee requested in his submittal that he install containment isolation valves in the radioactive gas sampler line, and in the instrument nitrogen compressor suction line to meet the requirements of Item 2.1.5.a of NUREG-0578. There are presently two isolation valves on both the radioactive gas sampler line and the instrument compressor suction line. However, during operation of the Containment Atmosphere Dilution (CAD) system, one of these valves will be opened and one will remain closed. The licensee proposed that an inboard isolation valve be added in series with the outboard valve in the instrument nitrogen compressor suction line, and an outboard isolation valve be added in the radioactive gas sampler line downstream of the header that connects all four sample lines. Accordingly, the licensee requested that Tables 3.7.1 and 3.7.4 be revised to include these valves as primary containment isolation valves and be designated Group 3 isolation valves and thus subject to the appropriate leak rate tests.

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Based on our review of the licensee's request for authorization to add containment isolation valves to the radioactive gas sampler lines and instrument nitrogen line, we conclude that the system design as proposed is in accordance with the requirements of General Design Criteria 54 and 56. The proposed design meets the criteria to provide redundancy of isolation barrier and of isolation signals to the valves. These valves will be subject to Type C leakage rate tests as outlined in Appendix J of 10 CFR Part 50. The isolation valves will be located within three feet of the present isolation valves. This meets the provision of SRP Section 6.2.4, Paragraph II.4 that the isolation valves be located as close as practical to the containment.

We conclude from our review that the proposed changes satisfy the provisions of SRP 6.2.4 and that the proposed change is acceptable.

Dated: October 29, 1980