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Dockets Nos. 50-277
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Mr. Edward G. Bauer, Jr.
Vice President and General Counsel
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

The Commission has issued the enclosed Amendments Nos. 75 and 74 to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated August 8, 1980, as supplemented by your letter dated October 3, 1980.

The amendments add operability and surveillance requirements for the new seismic monitoring instrumentation system you are planning to install.

Your application stated that the new system should be operational by the fall of 1980. In the event that you expect any delay in installation beyond December 1, 1980, we request that you advise us promptly.

Copies of our Safety Evaluation and a related Notice of Issuance are also enclosed.

Sincerely,
Original signed by
Robert W. Reid

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:
1. Amendment No. 75 to DPR-44
2. Amendment No. 74 to DPR-56
3. Safety Evaluation
4. Notice

cc w/enclosures: See next page

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DATE	11/12/80	11/17/80	11/12/80	11/14/80	11/17/80	

NRC FORM 318 (9-76) NRCM 0240 ☆U.S. GOVERNMENT PRINTING OFFICE: 1979-289-369

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

November 19, 1980

DISTRIBUTION:
Docket File
ORB#4 Rdg
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Docket No. 50-277/278

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: PEACH BOTTOM 2 & 3

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12) of the Notice are enclosed for your use.

☐ Notice of Receipt of Application for Construction Permit(s) and Operating License(s).

☐ Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.

☐ Notice of Availability of Applicant's Environmental Report.

☐ Notice of Proposed Issuance of Amendment to Facility Operating License.

☐ Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.

☐ Notice of Availability of NRC Draft/Final Environmental Statement.

☐ Notice of Limited Work Authorization.

☐ Notice of Availability of Safety Evaluation Report.

☐ Notice of Issuance of Construction Permit(s).

☒ Notice of Issuance of Facility Operating License(s) or Amendment(s).

☒ Other: Amendments 75 & 74
Referenced documents haveebeen provided PDR

Division of Licensing, ORB#4
Office of Nuclear Reactor Regulation

Enclosure:
As Stated

OFFICE →	ORB#4:DL				
SURNAME →	RIngram/cb				
DATE →	11/20/80				

NRC FORM 102 (1-76)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASH · GTON, D. C. 20555

November 19, 1980

Dockets Nos. 50-277
and 50-278

Mr. Edward G. Bauer, Jr.
Vice President and General Counsel
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

The Commission has issued the enclosed Amendments Nos. 75 and 74 to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated August 8, 1980, as supplemented by your letter dated October 3, 1980.

The amendments add operability and surveillance requirements for the new seismic monitoring instrumentation system you are planning to install.

Your application stated that the new system should be operational by the fall of 1980. In the event that you expect any delay in installation beyond December 1, 1980, we request that you advise us promptly.

Copies of our Safety Evaluation and a related Notice of Issuance are also enclosed.

Sincerely,

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:
1. Amendment No. 75 to DPR-44
2. Amendment No. 74 to DPR-56
3. Safety Evaluation
4. Notice

cc w/enclosures: See next page

Philadelphia Electric Company

cc w/enclosure(s):

Eugene J. Bradley
Philadelphia Electric Company
Assistant General Counsel
2301 Market Street
Philadelphia, Pennsylvania 19101

Troy B. Conner, Jr.
1747 Pennsylvania Avenue, N.W.
Washington, D. C. 20006

Raymond L. Hovis, Esq.
35 South Duke Street
York, Pennsylvania 17401

Warren K. Rich, Esq.
Assistant Attorney General
Department of Natural Resources
Annapolis, Maryland 21401

Philadelphia Electric Company
ATTN: Mr. W. T. Ullrich
Peach Bottom Atomic
Power Station
Delta, Pennsylvania 17314

Albert R. Steel, Chairman
Board of Supervisors
Peach Bottom Township
R. D. #1
Delta, Pennsylvania 17314

Curt Cowgill
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Peach Bottom Atomic Power Station
P. O. Box 399
Delta, Pennsylvania 17314

Director, Criteria and Standards
Division
Office of Radiation Programs (ANR-460)
U. S. Environmental Protection Agency
Washington, D. C. 20460

U. S. Environmental Protection Agency
Region III Office
ATTN: EIS COORDINATOR
Curtis Building (Sixth Floor)
6th and Walnut Streets
Philadelphia, Pennsylvania 19106

M. J. Cooney, Superintendent
Generation Division - Nuclear
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Government Publications Section
State Library of Pennsylvania
Education Building
Commonwealth and Walnut Streets
Harrisburg, Pennsylvania 17126

cc w/enclosure(s) & incoming dtd.:
8/8/80 & 10/3/80

Mr. R. A. Heiss, Coordinator
Pennsylvania State Clearinghouse
Governor's Office of State Planning
and Development
P. O. Box 1323
Harrisburg, Pennsylvania 17120

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated August 8, 1980, as supplemented October 3, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 75 , are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 19, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 75

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Pages	Insert Pages
iii	iii
--	vii (new page)
--	240n-240q (new pages)
257	257

TABLE OF CONTENTS (cont'd)

LIST OF TABLES

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.15 Seismic Monitoring Instrumentation	4.15 Seismic Monitoring Instrumentation
Applicability	Applicability
Applies to the operational status of the seismic monitoring instrumentation.	Applies to the surveillance requirements of the seismic monitoring instrumentation.
Specifications	Specifications
A. The seismic monitoring instrumentation shown in Table 3.15 shall be operable.	A. Each of the required seismic monitoring instruments shall be demonstrated operable by the performance of the Instrument Check, Instrument Functional Test, and Instrument Calibration operations at the frequencies shown in Table 4.15.
B. With one or more seismic monitoring instruments inoperable for more than 30 days, in lieu of any other report required by Specification 6.9.2, prepare and submit a Special Report to the Director of the appropriate Regional Office pursuant to Specification 6.9.3 within the next 10 working days outlining the cause of the malfunction and the plans for restoring the instrument(s) to operable status.	B. Each of the required seismic monitoring instruments actuated during a seismic event shall be restored to operable status within 24 hours and an Instrument Calibration performed within 5 days following the seismic event. Data shall be retrieved from actuated instruments and analyzed to determine the magnitude of the vibratory ground motion. In lieu of any other report required by Specification 6.9.2, a Special Report shall be prepared and submitted to the Director of the appropriate Regional Office pursuant to Specification 6.9.3 within 10 working days describing the magnitude, frequency spectrum and resultant effect upon facility features important to safety.
C. The provisions of Specification 3.0.c are not applicable.	

TABLE 3.15**

SEISMIC MONITORING INSTRUMENTATION

Instruments and Sensor Locations#	Measurement Range	Minimum Instruments Operable
1. Triaxial Time-History Accelerographs		
a. Containment Foundation (torus compartment)	0.1-10g	1
b. Refueling Floor	0.1-10g	1
c. RCIC Pump (Rm #7)	0.1-10g	1
d. "C" Diesel Generator	0.1-10g	1
2. Triaxial Peak Accelerographs		
a. Reactor Piping (Drywell)	0.01-2g	1
b. Refueling Floor	0.01-2g	1
c. "C" Diesel Generator	0.01-2g	1
3. Triaxial Response-Spectrum Recorders		
a. Cable Spreading Rm	0.1-10g	1*

* With reactor control room annunciation
** Effective upon completion of installation.
# Seismic instrumentation located in Unit 2

TABLE 4.15**

SEISMIC MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

Instruments and Sensor Locations#	Instrument* Check	Instrument* Functional Test	Instrument* Calibration
1. Triaxial Time-History Accelerographs			
a. Containment Foundation (torus compartment)	M	SA	R
b. Refueling Floor	M	SA	R
c. RCIC Pump (Rm #7)	M	SA	R
d. "C" Diesel Generator	M	SA	R
2. Triaxial Peak Accelerographs			
a. Reactor Piping (Drywell)	NA	NA	R
b. Refueling Floor	NA	NA	R
c. "C" Diesel Generator	NA	NA	R
3. Triaxial Response-Spectrum Recorders			
a. Cable Spreading Rm	M	SA	R

* Surveillance Frequencies

M: every month
SA: every 6 months
R: every 18 months

** Effective upon completion of installation.
# Seismic instrumentation located in Unit 2.

3.15/4.14 BASES

The operability of the seismic monitoring instrumentation ensures that sufficient capability is available to promptly determine the magnitude of a seismic event and evaluate the response of those features important to safety. This capability is required to permit comparison of the measured response to that used in the design basis for the plant.

The time-history recordings of the triaxial time-history accelerographs are done in the cable spreading room on a digital cassette accelerograph. In addition to being recorded, the containment foundation sensor is analyzed on line by a response spectrum analyzer. The spectrum of any sensor can be obtained by playing back its time-history cassette through the response spectrum analyzer.

6.9.2 Continued

(3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.

(4) Abnormal degradation of systems other than those specified in item 2.a(3) above designed to contain radioactive material resulting from the fission process.

Note: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

6.9.3 Unique Reporting Requirements

Special reports shall be submitted to the Director of the appropriate Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

a. Loss of shutdown margin, Specification 3.3.A and 4.3.A within 14 days of the event.

b. Reactor vessel inservice inspection, Specification 3.6.G and 4.6.G within 90 days of the completion of the reviews.

c. Report seismic monitoring instrumentation inoperable for more than 30 days (Specification 3.15B) within the next 10 working days. Submit a seismic event analysis (Specification 4.15B) within 10 working days of the event.

d. Primary containment leak rate testing approximately three months after the completion of the periodic integrated leak rate test (Type A) required by Specification 4.7.A.2.c.2. For each periodic test, leakage test results from Type A, B and C tests shall be reported. B and C tests are local leak rate tests required by Specification 4.7.A.2.f. The report shall contain an analysis and interpretation of the Type A test results and a summary analysis of periodic Type B and Type C tests that were performed since the last Type A test.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 74
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated August 8, 1980, as supplemented October 3, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 74, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 19, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 74

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Pages	Insert Pages
iii	iii
--	vii (new page)
--	240n-240q (new pages)
257	257

TABLE OF CONTENTS (cont'd)

LIST OF TABLES

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.15 Seismic Monitoring Instrumentation	4.15 Seismic Monitoring Instrumentation
Applicability	Applicability
Applies to the operational status of the seismic monitoring instrumentation.	Applies to the surveillance requirements of the seismic monitoring instrumentation.
Specifications	Specifications
A. The seismic monitoring instrumentation shown in Table 3.15 shall be operable.	A. Each of the required seismic monitoring instruments shall be demonstrated operable by the performance of the Instrument Check, Instrument Functional Test, and Instrument Calibration operations at the frequencies shown in Table 4.15.
B. With one or more seismic monitoring instruments inoperable for more than 30 days, in lieu of any other report required by Specification 6.9.2, prepare and submit a Special Report to the Director of the appropriate Regional Office pursuant to Specification 6.9.3 within the next 10 working days outlining the cause of the malfunction and the plans for restoring the instrument(s) to operable status.	B. Each of the required seismic monitoring instruments actuated during a seismic event shall be restored to operable status within 24 hours and an Instrument Calibration performed within 5 days following the seismic event. Data shall be retrieved from actuated instruments and analyzed to determine the magnitude of the vibratory ground motion. In lieu of any other report required by Specification 6.9.2, a Special Report shall be prepared and submitted to the Director of the appropriate Regional Office pursuant to Specification 6.9.3 within 10 working days describing the magnitude, frequency spectrum and resultant effect upon facility features important to safety.
C. The provisions of Specification 3.0.c are not applicable.	

TABLE 3.15**

SEISMIC MONITORING INSTRUMENTATION

Instruments and Sensor Locations#	Measurement Range	Minimum Instruments Operable
1. Triaxial Time-History Accelerographs		
a. Containment Foundation (torus compartment)	0.1-10g	1
b. Refueling Floor	0.1-10g	1
c. RCIC Pump (Rm #7)	0.1-10g	1
d. "C" Diesel Generator	0.1-10g	1
2. Triaxial Peak Accelerographs		
a. Reactor Piping (Drywell)	0.01-2g	1
b. Refueling Floor	0.01-2g	1
c. "C" Diesel Generator	0.01-2g	1
3. Triaxial Response-Spectrum Recorders		
a. Cable Spreading Rm	0.1-10g	1*

* With reactor control room annunciation
** Effective upon completion of installation.
# Seismic instrumentation located in Unit 2

TABLE 4.15**

SEISMIC MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

Instruments and Sensor Locations#	Instrument* Check	Instrument* Functional Test	Instrument* Calibration
1. Triaxial Time-History Accelerographs			
a. Containment Foundation (torus compartment)	M	SA	R
b. Refueling Floor	M	SA	R
c. RCIC Pump (Rm #7)	M	SA	R
d. "C" Diesel Generator	M	SA	R
2. Triaxial Peak Accelerographs			
a. Reactor Piping (Drywell)	NA	NA	R
b. Refueling Floor	NA	NA	R
c. "C" Diesel Generator	NA	NA	R
3. Triaxial Response-Spectrum Recorders			
a. Cable Spreading Rm	M	SA	R

* Surveillance Frequencies

M: every month
SA: every 6 months
R: every 18 months

** Effective upon completion of installation.
# Seismic instrumentation located in Unit 2.

3.15/4.14 BASES

The operability of the seismic monitoring instrumentation ensures that sufficient capability is available to promptly determine the magnitude of a seismic event and evaluate the response of those features important to safety. This capability is required to permit comparison of the measured response to that used in the design basis for the plant.

The time-history recordings of the triaxial time-history accelerographs are done in the cable spreading room on a digital cassette accelerograph. In addition to being recorded, the containment foundation sensor is analyzed on line by a response spectrum analyzer. The spectrum of any sensor can be obtained by playing back its time-history cassette through the response spectrum analyzer.

6.9.2 Continued

(3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.

(4) Abnormal degradation of systems other than those specified in item 2.a(3) above designed to contain radioactive material resulting from the fission process.

Note: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

6.9.3 Unique Reporting Requirements

Special reports shall be submitted to the Director of the appropriate Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

a. Loss of shutdown margin, Specification 3.3.A and 4.3.A within 14 days of the event.

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c. Report seismic monitoring instrumentation inoperable for more than 30 days (Specification 3.15B) within the next 10 working days. Submit a seismic event analysis (Specification 4.15B) within 10 working days of the event.

d. Primary containment leak rate testing approximately three months after the completion of the periodic integrated leak rate test (Type A) required by Specification 4.7.A.2.c.2. For each periodic test, leakage test results from Type A, B and C tests shall be reported. B and C tests are local leak rate tests required by Specification 4.7.A.2.f. The report shall contain an analysis and interpretation of the Type A test results and a summary analysis of periodic Type B and Type C tests that were performed since the last Type A test.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

AMENDMENT NO. 75 TO LICENSE NO. DPR-44
AND
AMENDMENT NO. 74 TO LICENSE NO. DPR-56

PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3

PHILADELPHIA ELECTRIC COMPANY

DOCKETS NOS. 50-277 AND 50-278

I. INTRODUCTION

By letter dated August 8, 1980, Philadelphia Electric Company (licensee) proposed amendments to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. The amendments would revise the Technical Specifications by adding operability and surveillance requirements for seismic monitoring instrumentation. The licensee's application is in response to our request dated July 1, 1980.

II. EVALUATION

By our letter dated July 1, 1980, we indicated to the licensee that a review of the Technical Specifications for Peach Bottom 2 and 3 reflected that they did not include Limiting Conditions for Operation or Surveillance Requirements for seismic instrumentation. We requested that the licensee propose appropriate Technical Specifications so that verification of his conformance with the requirements of Appendix A to 10 CFR Part 100 could be assured. In response to our request the licensee indicated that a new seismic monitoring system which would comply with the guidelines of Regulatory Guide 1.12 was initiated in the spring of 1980; delivery was expected in September 1980; and the equipment would be operational by the fall of 1980. The system to be installed consists of four (4) triaxial time-history accelerographs, three (3) triaxial peak accelerographs, and a triaxial response-spectrum recorder. On September 24, 1980, we discussed, via telephone, the proposed system. The licensee stated that the response-spectrum recorder located in the cable spreading room has two sets of sensors associated with it: one triaxial time-history and one response-spectrum monitor, both located on the containment foundation. This information was documented in the licensee's letter dated October 3, 1980. Based on this information, we have determined that the proposed system is in conformance with our position set forth in Regulatory Guide 1.12 and is therefore acceptable. We suggested to the licensee that the triaxial response-spectrum recorder description be included in the bases to clearly demonstrate consistency with Regulatory Guide 1.12 regulatory position. He agreed.

The licensee's submittal of Limiting Conditions for Operation for this equipment and associated Surveillance Requirements is consistent with current licensing practice as set forth in NUREG-0123, "Standard Technical Specifications for General Electric Boiling Water Reactors". Accordingly, the licensee's submittal, as amended by the staff, is acceptable.

III. ENVIRONMENTAL CONSIDERATIONS

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

IV. CONCLUSIONS

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 19, 1980

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-277 AND 50-278

PHILADELPHIA ELECTRIC COMPANY, ET AL

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 75 and 74 to Facility Operating Licenses Nos. DPR-44 and DPR-56, issued to Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company, which revised Technical Specifications for operation of the Peach Bottom Atomic Power Station, Units Nos. 2 and 3 (the facility) located in York County, Pennsylvania. The amendments are effective as of the date of issuance.

The amendments to the Technical Specifications add operability and surveillance requirements for a new seismic monitoring instrumentation system which is being installed at the facility.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated August 8, 1980, as supplemented October 3, 1980, (2) Amendments Nos. 75 and 74 to Licenses Nos. DPR-44 and DPR-56, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 19th day of November 1980.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing