October 1, 19

Mr. Michael B. Roche
Vice President and Director
GPU Nuclear, Inc.
Oyster Creek Nuclear Generating Station
P.O. Box 388
Forked River, NJ 08731

SUBJECT: OYSTER CREEK - ISSUANCE OF AMENDMENT NO. 198 , RE: CHANGE TO SCRAM TIME TESTING REQUIREMENTS (TAC NO. MA2335)

The Commission has issued the enclosed Amendment No. 198 to Facility Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station, in response to your application dated July 21, 1998.

The amendment deletes Table 3.5.2 which lists automatic primary containment isolation valves. In addition, this amendment clarifies the applicability of an action statement which applies to several limiting conditions for operation in Section 3.5 and deletes closure time requirements for several automatic isolation valves in Section 4.5.F.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Ronald B. Eaton, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-219

Enclosures: 1. Amendment No. 198 to DPR-16
2. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION: See attached page

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M. Roche
GPU Nuclear, Inc.

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DATED: October 1, 1998

AMENDMENT NO. 198 TO FACILITY OPERATING LICENSE NO. DPR-16-OYSTER CREEK

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

GPU NUCLEAR, INC.

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 198
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment filed by GPU Nuclear, Inc., et al., (the licensee), dated July 21, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-16 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 198 , are hereby incorporated in the license. GPU Nuclear, Inc. shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Cecil O. Thomas, Director
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: October 1, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 198

FACILITY OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace the following page of the Appendix A, Technical Specifications, with the attached page as indicated. The revised page is identified by amendment number and contains vertical lines indicating the areas of change.

Remove	Insert
4.2-1	4.2-1

4.2 REACTIVITY CONTROL

Applicability: Applies to the surveillance requirements for reactivity control.

Objective: To verify the capability for controlling reactivity.

Specification:

A. SDM shall be verified:
   1. Prior to each CORE ALTERATION, and
   2. Once within 4 hours following the first criticality following any CORE ALTERATION.

B. The control rod drive housing support system shall be inspected after reassembly.

C. 1. After each major refueling outage and prior to resuming POWER OPERATION, all operable control rods shall be scram time tested from the fully withdrawn position with reactor pressure above 800 psig;

OR

2. a. Prior to resuming POWER OPERATION, scram time testing shall be conducted with the reactor depressurized. The 90% scram insertion time shall not exceed 2.2 seconds for each control rod tested; AND

   b. Prior to exceeding 40% reactor power operation, scram time testing shall be conducted with the reactor pressurized above 800 psig. The acceptance criteria of Section 3.2.B.3 shall be met.

3. Following each reactor scram from rated pressure, the mean 90% insertion time shall be determined for eight selected rods. If the mean 90% insertion time of the selected control rod drives does not fall within the range of 2.4 to 3.1 seconds or the measured scram time of any one drive for 90% insertion does not fall within the range of 1.9 to 3.6 seconds, an evaluation shall be made to provide reasonable assurance that proper control rod drive performance is maintained.

4. Following any outage not initiated by a reactor scram, eight rods shall be scram tested with reactor pressure above 800 psig provided these have not been measured in six months. The same criteria of 4.2.C(2) shall apply.

D. Each partially or fully withdrawn control rod shall be exercised at least once each week. This test shall be performed within 24 hours in the event power operation is continuing with two or more inoperable control rods or in the event power operation is continuing with one fully or partially withdrawn rod which cannot be moved and for which control rod drive mechanism damage has not been ruled out. The surveillance need not be completed within 24 hours if the number of inoperable rods has been reduced to less than two and if it has been demonstrated that control rod drive mechanism collet housing failure is not the cause of an immovable control rod.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 198

TO FACILITY OPERATING LICENSE NO. DPR-16

GPU NUCLEAR, INC. AND

JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated July 21, 1998, GPU Nuclear, Inc., operator of Oyster Creek Nuclear Generating Station (OCNGS) requested changes to the Technical Specification (TS) for OCNGS. The requested change would permit an alternative to the requirement to perform Control Rod Drive (CRD) scram time testing with the reactor pressurized before resuming power operation. Instead, the change would also permit: (1) scram time testing with the reactor depressurized before resuming power operation, and (2) a second scram time test with reactor pressure above 800 psig, before exceeding 40% reactor power.

2.0 EVALUATION

The Cold (Depressurized) Scram Time Test TS of 2.2 seconds at zero psig corresponds to about 3.2 seconds at 800 psig and 3.8 seconds at 600 psig. The current TS requirement is for the 90% insertion time average for all rods to be less than 5.0 seconds, with the reactor pressure above 800 psig. Therefore, the cold (depressurized) 90% insertion time acceptance criterion of TS Section 3.2.B.3 is met. Additionally, below 40% power, sufficient inherent margin exists to core operating limits (due to lower heat flux), to offset the potential impact of any slow scram time. Other considerations associated with cold scram tests are as follows. The cold scram tests will subject the CRD mechanisms to higher mechanical loads than a hot scram test and may cause the CRD buffer deal to wear or fail. This would result in difficulty in the ability to move the rod. However, the safety functions of the rod to insert on the receipt of a scram signal will be unaffected by this degradation. There may also be an increased risk of stub tube leakage with the cold scram test but, the integrity of the stub tube is verified by a 1000 psi hydrostatic test at every reactor start up.

The Hot Scram Time Test is the same as currently required by the OCNGS TS. The requested change allows the hot scram test to be done before reaching 40% power, instead of being required before startup.

The requested change will not increase the probability of occurrence of an accident previously evaluated in the Safety Analysis Report (SAR) or increase the consequences. The change will not create the possibility of a new or different kind of accident. This change will not decrease the margin of safety as defined in the bases of any TS. There are no safety concerns

associated with the increased wear or possibility of leaks on the CRDs with the cold scram time tests. The safety function of the reactor, the ability to scram, will not be affected by these changes to the TS.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (63 FR 43204). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Frahm, Sr.

Date: October 1, 1998