

MAR 01 1983

DISTRIBUTION:
Docket File
NRC PDR
L PDR
ORB#4 Rdg

Gray File=2 +4
Eisenhut
RIngram
GGears
OELD
AEOD
IE-Harmon (2)
EJordan
JTaylor

ACRS-10
LSchneider
DBrinkman
CMiles
RDiggs
TBarnhart-3
ASLAB
KEccleston

Dockets Nos. 50-277
and 50-278

Mr. Edward G. Bauer, Jr.
Vice President and General Counsel
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

SUBJECT: TECHNICAL SPECIFICATION AMENDMENT PERTAINING TO OPERABILITY
AND SURVEILLANCE REQUIREMENTS FOR THE SCRAM DISCHARGE
VOLUME VENT AND DRAIN ISOLATION VALVES

The Commission has issued the enclosed Amendments Nos. 90 and 90 to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated December 15, 1982.

The changes to the TSs provide operability and surveillance requirements for the scram discharge volume vent and drain isolation valves.

Copies of the Safety Evaluation and a related Notice of Issuance are also enclosed.

Sincerely,

"ORIGINAL SIGNED BY:"

Gerald E. Gears, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 90 to DPR-44
2. Amendment No. 90 to DPR-56
3. Safety Evaluation
4. Notice

cc w/enclosures:
See next page

8303110151 830301
PDR ADOCK 05000277
P PDR

*No copies to be made of this
of Amendment 104
R. G. Gears
2/27/83
Reviewed and
approved*

OFFICE	ORB#4:DL	ORB#4:DL	C-ORB#4:DL	AD-OR:DL	OELD	ORB#2:DL
SURNAME	RIngram	GGears	JSto	GLinas	CUTCHIN	KEccleston
DATE	2/27/83	2/28/83	2/28/83	2/28/83	2/27/83	2/27/83

MAR 01 1983
Philadelphia Electric Company

cc w/enclosure(s):

Eugene J. Bradley
Philadelphia Electric Company
Assistant General Counsel
2301 Market Street
Philadelphia, Pennsylvania 19101

Troy B. Conner, Jr.
1747 Pennsylvania Avenue, N.W.
Washington, D. C. 20006

Thomas A. Deming, Esq.
Assistant Attorney General
Department of Natural Resources
Annapolis, Maryland 21401

Philadelphia Electric Company
ATTN: Mr. W. T. Ullrich
Peach Bottom Atomic
Power Station
Delta, Pennsylvania 17314

Albert R. Steel, Chairman
Board of Supervisors
Peach Bottom Township
R. D. #1
Delta, Pennsylvania 17314

Allen R. Blough
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Peach Bottom Atomic Power Station
P. O. Box 399
Delta, Pennsylvania 17314

Mr. Ronald C. Haynes, Regional Administrator
U. S. Nuclear Regulatory Commission, Region I
Office of Inspection and Enforcement
631 Park Avenue
King of Prussia, Pennsylvania 19406

Regional Radiation Representative
EPA Region III
Curtis Building (Sixth Floor)
6th and Walnut Streets
Philadelphia, Pennsylvania 19106

M. J. Cooney, Superintendent
Generation Division - Nuclear
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Mr. R. A. Heiss, Coordinator
Pennsylvania State Clearinghouse
Governor's Office of State Planning
and Development
P. O. Box 1323
Harrisburg, Pennsylvania 17120



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 90
License No. DPR-44

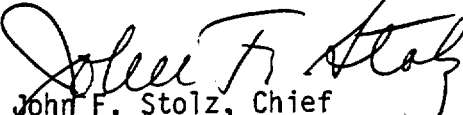
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated December 15, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 90, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: MAR 01 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 90

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

Insert

--

181a. (new page)

186

186

187

187

TABLE 3.7.1 (Cont'd)

PRIMARY CONTAINMENT ISOLATION VALVES

Group	Valve Identification	No. of Power Operated Valves		Maximum Operating Time (sec.)	Normal Position	Action on Initiating Signal
		Inboard	Outboard			
NA	Scram discharge volume vent valves	2	2	15	0	GC
NA	Scram discharge volume drain valves	1	1	15	0	GC

Amendment No. 90

181a

PBAPS
TABLE 3.7.4 (Cont'd)

UNIT 2

PRIMARY CONTAINMENT TESTABLE ISOLATION VALVES

<u>Pen No.</u>		<u>NOTES</u>
22	AO-2969A; Check Valve	(1) (2) (4) (5) (10)
25	AO-2520; AO-2505; AO-2519; AO-2521A; AO-2521B	(1) (2) (4) (5) (9)
25	AO-2523; Check Valves	(1) (2) (4) (5)
26	AO-2506; AO-2507	(1) (2) (4) (5) (9)
26	SV-2671G; SV-2978G	(1) (2) (4) (5)
26	AO-2509; AO-2510; AO-4235	(1) (2) (4) (5) (9)
26	SV-4960B; SV-4961B; SV-4966B	(1) (2) (4) (5)
26	SV-8100	(1) (2) (4) (5)
26, 51C 203, 219	SV-8101	(1) (2) (4) (5)
38	CV32A, CV35A	(1) (2) (4) (5)
38	CV32B, CV35B	(1) (2) (4) (5)
38	CV33, CV36	(1) (2) (4) (5)
39A	MO-10-31B; MO-10-26B	(1) (2) (4) (5) (9)
39A	SV-4949B; SV-4948B	(1) (2) (4) (5)
39B	MO-10-31A; MO-10-26A	(1) (2) (4) (5) (9)
39B	SV-4949A; SV-4948A	(1) (2) (4) (5)
41	AO-2-39; AO-2-40	(1) (2) (4) (5) (9)
42	Check Valve 11-16, XV-14A, XV-14B	(1) (2) (4) (5) (10)
51A	SV-2671E; SV-2978E	(1) (2) (4) (5)
51B	SV-2671D; SV-2978D	(1) (2) (4) (5)
51C	SV-2671C; SV-2978C	(1) (2) (4) (5)
51C	SV-4960C; SV-4961C; SV-4966C	(1) (2) (4) (5)
51D	SV-2980; Check Valve	(1) (2) (4) (5)
52F	AO-2969B; Check Valve	(1) (2) (4) (5) (10)
57	AO-2-316; AO-2-317	(1) (2) (4) (5) (9)

PRIMARY CONTAINMENT TESTABLE ISOLATION VALVES

<u>Pen No.</u>		<u>NOTES</u>
102B	Breathing Air System-2 Gate Valves	(1) (2) (4) (5) (9)
203	SV-2671B; SV-2978B	(1) (2) (4) (5)
203	SV-4960D; SV-4961D SV-4966D	(1) (2) (4) (5)
205A	AO-2502B; Check Valve 9-26B	(1) (2) (4) (5) (9)
205B	AO-2502A; Check Valve 9-26A	(1) (2) (4) (5) (9)
211A	MO-10-38B; MO-10-39B; MO-10-34B	(1) (2) (4) (5) (9)
211A	SV-4951B; SV-4950B	(1) (2) (4) (5)
211B	MO-10-38A; MO-10-39A; MO-10-34A	(1) (2) (4) (5) (9)
211B	SV-4951A; SV-4950A	(1) (2) (4) (5)
212	Check Valve 13-50; AO-4240; AO-4241	(1) (2) (4) (5) (9)
214	Check Valve 23-65; AO-4247; AO-4248	(1) (2) (4) (5) (9)
217B	MO-4244; MO-4244A	(1) (2) (4) (5) (9)
218A	AO-2968	(1) (2) (4) (5) (10)
218B	SV-2671A; SV-2978A	(1) (2) (4) (5)
219	AO-2511; AO-2512; AO-2513; AO-2514	(1) (2) (4) (5) (9)
219	SV-2671F; SV-2978F; SV-4960A SV-4961A; SV-4966A	(1) (2) (4) (5)
221	Check Valve 13-38	(1) (2) (4) (5) (9)
223	Check Valve 23-56	(1) (2) (4) (5) (9)
225	MO-13-41; MO-13-39	(1) (2) (4) (5) (9)
225	MO-14-70; MO-14-71	(1) (2) (4) (5) (9)
227	MO-23-58; MO-23-57	(1) (2) (4) (5) (9)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 90
License No. DPR-56

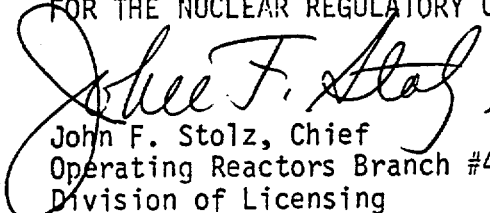
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated December 15, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 90, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: **MAR 01 1983**

ATTACHMENT TO LICENSE AMENDMENT NO. 90

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

--

186

187

Insert

181a (new page)

186

187

TABLE 3.7.1 (Cont'd)

PRIMARY CONTAINMENT ISOLATION VALVES

Group	Valve Identification	No. of Power Operated Valves		Maximum Operating Time (sec.)	Normal Position	Action on Initiating Signal
		Inboard	Outboard			
NA	Scram discharge volume vent valves	2	2	15	0	GC
NA	Scram discharge volume drain valves	1	1	15	0	GC

PRIMARY CONTAINMENT TESTABLE ISOLATION VALVES

<u>Pen No.</u>		<u>NOTES</u>
22	AO-3969A; Check Valve	(1)-(2) (4) (5) (10)
25	AO-3520; AO-3505; AO-3519; AO-3521A; AO-3521B	(1) (2) (4) (5) (9)
25	AO-3523; Check Valves	(1) (2) (4) (5)
26	AO-3506; AO-3507	(1) (2) (4) (5) (9)
26	SV-3671G; SV-3978G	(1) (2) (4) (5)
26	AO-3509; AO-3510; AO-5235	(1) (2) (4) (5) (9)
26	SV-5960B; SV-5961B; SV-5966B	(1) (2) (4) (5)
26	SV-9100	(1) (2) (4) (5)
26, 51C 203, 219	SV-9101	(1) (2) (4) (5)
38	CV32A, CV35A	(1) (2) (4) (5)
38	CV32B, CV35B	(1) (2) (4) (5)
38	CV33, CV36	(1) (2) (4) (5)
39A	MO-10-31B; MO-10-26B	(1) (2) (4) (5) (9)
39A	SV-5949B; SV-5948B	(1) (2) (4) (5)
39B	MO-10-31A; MO-10-26A	(1) (2) (4) (5) (9)
39B	SV-5949A; SV-5948A	(1) (2) (4) (5)
41	AO-2-39; AO-2-40	(1) (2) (4) (5) (9)
42	Check Valve 11-16, XV-14A, XV-14B	(1) (2) (4) (5) (10)
51A	SV-3671E; SV-3978E	(1) (2) (4) (5)
51B	SV-3671D; SV-3978D	(1) (2) (4) (5)
51C	SV-3671C; SV-3978C	(1) (2) (4) (5)
51C	SV-5960C; SV-5961C; SV-5966C	(1) (2) (4) (5)
51D	SV-3980; Check Valve	(1) (2) (4) (5)
52F	AO-3969B; Check Valve	(1) (2) (4) (5) (10)
57	AO-2-316; AO-2-317	(1) (2) (4) (5) (9)

PBAPS
TABLE 3.7.4 (Cont'd)

UNIT 3

PRIMARY CONTAINMENT TESTABLE ISOLATION VALVES

<u>Pen No.</u>		<u>NOTES</u>
102B	Breathing Air System-2 Gate Valves	(1) (2) (4) (5) (9)
203	SV-3671B; SV-3978B	(1) (2) (4) (5)
203	SV-5960D; SV-5961D SV-5966D	(1) (2) (4) (5)
205A	AO-3502B; Check Valve 9-26B	(1) (2) (4) (5) (9)
205B	AO-3502A; Check Valve 9-26A	(1) (2) (4) (5) (9)
211A	MO-10-38B; MO-10-39B; MO-10-34B	(1) (2) (4) (5) (9)
211A	SV-5951B; SV-5950B	(1) (2) (4) (5)
211B	MO-10-38A; MO-10-39A; MO-10-34A	(1) (2) (4) (5) (9)
211B	SV-5951A; SV-5950A	(1) (2) (4) (5)
212	Check Valve 13-50; AO-5240; AO-5241	(1) (2) (4) (5) (9)
214	Check Valve 23-65; AO-5247; AO-5248	(1) (2) (4) (5) (9)
217B	MO-5244; MO-5244A	(1) (2) (4) (5) (9)
218A	AO-3968	(1) (2) (4) (5) (10)
218B	SV-3671A; SV-3978A	(1) (2) (4) (5)
219	AO-3511; AO-3512; AO-3513; AO-3514	(1) (2) (4) (5) (9)
219	SV-3671F; SV-3978F; SV-5960A SV-5961A; SV-5966A	(1) (2) (4) (5)
221	Check Valve 13-38	(1) (2) (4) (5) (9)
223	Check Valve 23-56	(1) (2) (4) (5) (9)
225	MO-13-41; MO-13-39	(1) (2) (4) (5) (9)
225	MO-14-70; MO-14-71	(1) (2) (4) (5) (9)
227	MO-23-58; MO-23-57	(1) (2) (4) (5) (9)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 90 AND 90 TO FACILITY LICENSE NOS. DPR-44 AND DPR-56

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION UNITS NOS. 2 AND 3

DOCKETS NOS. 50-277 AND 50-278

Introduction

By letter dated December 15, 1982 (Reference 1), the Philadelphia Electric Company (PECo or the licensee) made application to amend the Technical Specifications (TSs) for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3, to provide operability and surveillance requirements for the scram discharge volume (SDV) vent and drain isolation valves. As indicated in their December 16, 1980 letter (Reference 2), the licensee, as a result of an NRC requested reevaluation of their SDV system, outlined modifications which would provide redundant isolation valves on the SDV system vent and drain lines in series with the existing valves. These new valves would protect against loss of reactor coolant from the discharge volume, given a single active failure of one of the valves.

Evaluation

The licensee committed by letter (Reference 2) to provide redundant isolation valves on the SDV system vent and drain lines in series with the existing valves. These valves were installed in conformance with criteria developed by the BWR Owners' Group as requested by the NRC staff.

The proposed revisions to the TSs are intended to provide operability and surveillance requirements for these redundant isolation valves in accordance with the existing TSs on Primary Containment Isolation Valves. We have reviewed the requested TS changes to determine whether they are in conformance with the licensee's existing surveillance TSs for Containment Isolation Valves. Our review indicates that the proposed TSs do conform with the licensee's existing TS surveillance requirements and are, therefore, acceptable. The licensee has also requested the deletion of obsolete footnotes pertaining to Amendment No. 73 for Peach Bottom Unit No. 2 and Amendment No. 72 for Peach Bottom Unit No. 3 since the modifications authorized by these amendments have been completed.

8303110157 830301
PDR ADOCK 05000277
PDR

We have reviewed the above deletion requests and concur that the modifications have been completed. Therefore, these TS changes are acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §1.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of an accident previously evaluated, do not create the possibility of an accident of a type different from any evaluated previously, and do not involve a significant reduction in a margin of safety, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: **MAR 01 1983**

The following NRC personnel have contributed to this Safety Evaluation:
Gerry Gears and Ken Eccleston.

References:

1. Letter to H. R. Denton (NRR) from E. J. Bradley (PECo), Application for Amendment of Facility Operating Licenses DPR-44 and DPR-56, December 15, 1982.
2. Letter to D. G. Eisenhut (NRR) from S. L. Daltroff (PECo), Responses to October 1, 1980 letter Requesting Results of SDV System Reevaluation, December 16, 1980.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKETS NOS. 50-277 AND 50-278PHILADELPHIA ELECTRIC COMPANY, ET ALNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 90 and 90 to Facility Operating Licenses Nos. DPR-44 and DPR-56, issued to Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company, which revised Technical Specifications (TSs) for operation of the Peach Bottom Atomic Power Station, Units Nos. 2 and 3 (the facility) located in York County, Pennsylvania. The amendments are effective as of the date of issuance.

The revised TSs provide operability and surveillance requirements for the scram discharge volume vent and drain isolation valves.

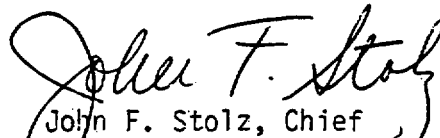
The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated December 15, 1982, (2) Amendment No. 90 to License No. DPR-44 and Amendment No. 90 to License No. DPR-56, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 1st day of March 1983.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing