

REGULATORY DOCKET FILE COPY

MAY 13 1980

Dockets Nos. 50-277
and 50-278

Mr. Edward G. Bauer, Jr.
Vice President and General Counsel
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Distribution:
Docket files
NRC PDR
L PDR
TERA
NSIC
NRR Rdg.
ORB#4 Rdg.
DEisenhut
RPurple
TNovak
RTedesco
GLainas
RReid
DVerrelli
RIngram
Attorney, OELD
OI&E (5)

BJones (8)
BScharf (10)
DBrinkman
ACRS (16)
CMiles
RDiggs
HRDenton
JHeltemes, AEOD
Gray files

Dear Mr. Bauer:

The Commission has issued the enclosed Amendments Nos. 69 and 68 to Facility Operating License Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station (PBAPS), Units Nos. 2 and 3. The amendments are in response to your application dated April 15, 1980, and consist of administrative changes to (1) correct typographical errors, (2) delete obsolete notes regarding inerting makeup systems, (3) revise the specification regarding inoperable power supply equipment by deleting specific reporting requirements which are inherently specified by Administrative Controls and (4) clarify the staff's requirements for reactor mode switch refuel position interlocks, station fire hose hydrostatic test pressure, and minimum shift crew composition requirements. Each of these are discussed below:

1. Typographical errors - Table 3.2.B incorrectly specifies the low-low water level setpoint as -49" indicated level vs the correct setting of -48". This correction is supported by the bases portion of the specification (page 89) and Figure 7.8.2 of the PBAPS Final Safety Analysis Report (FSAR) on page 249. The correct abbreviation for the Operations and Safety Review Committee is OSR instead of ONSR. Based on our review, we find that the corrections are appropriate.
2. Inerting Makeup System - The Technical Specifications as originally issued for both Units 2 and 3 provided for the installation of a Containment Atmospheric Dilution (CAD) System before the end of the first refueling outage for each unit. During the interim period of time, the Limiting Conditions for Operation and Surveillance Requirements for the CAD were applicable to the Inerting Makeup System. Both units now have operable CAD systems with appropriate specifications. Therefore, deletion of the obsolete annotations relative to the inerting makeup system is appropriate.

ORB#4:DL
RIngram
05/12/80

ORB#4:DL
DVerrelli:kb
05/11/80

ORB#4:DL
RReid
05/12/80

AD-
TNovak
05/13/80

OELD
CUTCHIN
05/14/80

*No legal objection
+ pending assistance of
my amendment*

8005270 502

3. Inoperable Power Supply Equipment - The specification as originally issued included specific wording for notification of the AEC of degraded conditions. Since Section 6.9.2 as subsequently revised for Units 2 and 3, establishes a consistent set of reporting requirements for inoperable equipment, the proposed revision requires the same report; therefore the change is acceptable.
4. Clarification of Refueling Interlocks - The current wording of surveillance requirements is clarified to clearly identify the applicability to core alterations within or over the reactor core. The change eliminates the potential for misinterpretation that the specification also applies to fuel movements in the fuel pool storage area. Specifications related to the Spent Fuel Pool are included in Section 5.5. Based on our review, we find that the clarification is acceptable.
5. Clarification of Station Fire Hose Test Pressure - The current specification states that the hydrostatic test pressure is 250 psig. The proposed change would conform this specification to the staff's generic fire protection requirements of at least 50 psig greater than the maximum pressure available at that hose station but not less than 150 psig. Based on our review, we find the change acceptable.
6. Clarification of Minimum Shift Crew Composition - The current wording of the specifications provides no flexibility in minimum shift manning availability which might occur by unexpected absences or sudden illness or injury. The proposed change would conform this specification to the staff's Standard Technical Specifications and would rectify this deficiency. We find this change acceptable.

We note that the staff is currently considering an increase to the minimum shift staffing at operating reactors. The change approved by these amendments is not associated with this planned action.

We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendments. The amendments apply to administrative details. Therefore, we have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level, and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Since the amendments apply only to administrative details, they do not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. They do not involve a significant increase in the probability or consequences of an accident, do not involve a significant decrease in a safety margin, and therefore do not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

A copy of the Notice of Issuance is also enclosed.

Sincerely,

Original signed by
Robert W. Reid

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 59 to DPR-44
2. Amendment No. 68 to DPR-56
3. Notice of Issuance

cc w/enclosures: See next page



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

May 16, 1980

Distribution:
Docket files
ORB#4 Rdg.
R. Ingram

Docket No. **50-277**
50-278

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: **PEACH BOTTOM ATOMIC POWER STATION, UNITS NOS. 2 AND 3**

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Availability of Applicant's Environmental Report.
- Notice of Proposed Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).
- Other: Amendments No. 69 and 68. Referenced document: has been provided PDR.

Division of Licensing, ORB#4
Office of Nuclear Reactor Regulation

Enclosure:
As Stated

OFFICE →	ORB#4:DL					
SURNAME →	R. Ingram					
DATE →	05/16/80					



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 16, 1980

Dockets Nos. 50-277
and 50-278

Mr. Edward G. Bauer, Jr.
Vice President and General Counsel
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

The Commission has issued the enclosed Amendments Nos. 69 and 68 to Facility Operating License Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station (PBAPS), Units Nos. 2 and 3. The amendments are in response to your application dated April 15, 1980, and consist of administrative changes to (1) correct typographical errors, (2) delete obsolete notes regarding inerting makeup systems, (3) revise the specification regarding inoperable power supply equipment by deleting specific reporting requirements which are inherently specified by Administrative Controls and (4) clarify the staff's requirements for reactor mode switch refuel position interlocks, station fire hose hydrostatic test pressure, and minimum shift crew composition requirements. Each of these are discussed below:

1. Typographical errors - Table 3.2.B incorrectly specifies the low-low water level setpoint as -49" indicated level vs the correct setting of -48". This correction is supported by the bases portion of the specification (page 89) and Figure 7.8.2 of the PBAPS Final Safety Analysis Report (FSAR) on page 249. The correct abbreviation for the Operations and Safety Review Committee is OSR instead of ONSR. Based on our review, we find that the corrections are appropriate.
2. Inerting Makeup System - The Technical Specifications as originally issued for both Units 2 and 3 provided for the installation of a Containment Atmospheric Dilution (CAD) System before the end of the first refueling outage for each unit. During the interim period of time, the Limiting Conditions for Operation and Surveillance Requirements for the CAD were applicable to the Inerting Makeup System. Both units now have operable CAD systems with appropriate specifications. Therefore, deletion of the obsolete annotations relative to the inerting makeup system is appropriate.

8005270 508

3. Inoperable Power Supply Equipment - The specification as originally issued included specific wording for notification of the AEC of degraded conditions. Since Section 6.9.2 as subsequently revised for Units 2 and 3, establishes a consistent set of reporting requirements for inoperable equipment, the proposed revision requires the same report; therefore the change is acceptable.
4. Clarification of Refueling Interlocks - The current wording of surveillance requirements is clarified to clearly identify the applicability to core alterations within or over the reactor core. The change eliminates the potential for misinterpretation that the specification also applies to fuel movements in the fuel pool storage area. Specifications related to the Spent Fuel Pool are included in Section 5.5. Based on our review, we find that the clarification is acceptable.
5. Clarification of Station Fire Hose Test Pressure - The current specification states that the hydrostatic test pressure is 250 psig. The proposed change would conform this specification to the staff's generic fire protection requirements of at least 50 psig greater than the maximum pressure available at that hose station but not less than 150 psig. Based on our review, we find the change acceptable.
6. Clarification of Minimum Shift Crew Composition - The current wording of the specifications provides no flexibility in minimum shift manning availability which might occur by unexpected absences or sudden illness or injury. The proposed change would conform this specification to the staff's Standard Technical Specifications and would rectify this deficiency. We find this change acceptable.

We note that the staff is currently considering an increase to the minimum shift staffing at operating reactors. The change approved by these amendments is not associated with this planned action.

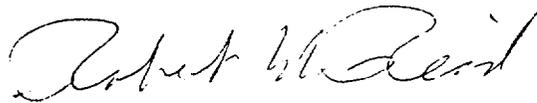
We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendments. The amendments apply to administrative details. Therefore, we have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level, and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

May 16, 1980

Since the amendments apply only to administrative details, they do not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. They do not involve a significant increase in the probability or consequences of an accident, do not involve a significant decrease in a safety margin, and therefore do not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

A copy of the Notice of Issuance is also enclosed.

Sincerely,



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 69 to DPR-44
2. Amendment No. 68 to DPR-56
3. Notice of Issuance

cc w/enclosures: See next page

Mr. Edward G. Bauer, Jr.
Philadelphia Electric Company

cc:

Eugene J. Bradley
Philadelphia Electric Company
Assistant General Counsel
2301 Market Street
Philadelphia, Pennsylvania 19101

Troy B. Conner, Jr.
1747 Pennsylvania Avenue, N. W.
Washington, D. C. 20006

Raymond L. Hovis, Esquire
35 South Duke Street
York, Pennsylvania 17401

Warren K. Rich, Esquire
Assistant Attorney General
Department of Natural Resources
Annapolis, Maryland 21401

Philadelphia Electric Company
ATTN: Mr. W. T. Ullrich
Peach Bottom Atomic
Power Station
Delta, Pennsylvania 17314

Albert R. Steel, Chairman
Board of Supervisors
Peach Bottom Township
R. D. #1
Delta, Pennsylvania 17314

Curt Cowgill
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Peach Bottom Atomic Power Station
P. O. Box 399
Delta, Pennsylvania 17314

Director, Technical Assessment
Division
Office of Radiation Programs
(AW-459)
US EPA
Crystal Mall #2
Arlington, Virginia 20460

Region III Office
ATTN: EIS COORDINATOR
Curtis Building (Sixth Floor)
6th and Walnut Streets
Philadelphia, Pennsylvania 19106

M. J. Cooney, Superintendent
Generation Division - Nuclear
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Government Publications Section
State Library of Pennsylvania
Education Building
Commonwealth and Walnut Streets
Harrisburg, Pennsylvania 17126

cc w/enclosure(s) & incoming dtd.:
04/15/80
Mr. R. A. Heiss, Coordinator
Pennsylvania State Clearinghouse
Governor's Office of State Planning
and Development
P. O. Box 1323
Harrisburg, Pennsylvania 17120



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 69
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated April 15, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 69, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 16, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 69

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

<u>Remove Pages</u>	<u>Insert Pages</u>
63	63
64	64
173	173
174	174
219	219
220	220
225	225
226	226
240f	240f
243	243
249	249

Page 63 is unchanged and is included for your convenience only.

PBAPS

NOTES FOR TABLE 3.2.A

1. Whenever Primary Containment integrity is required by Section 3.7, there shall be two operable or tripped trip systems for each function.
2. If the first column cannot be met for one of the trip systems, that trip system shall be tripped or the appropriate action listed below shall be taken.
 - A. Initiate an orderly shutdown and have the reactor in Cold Shutdown Condition in 24 hours.
 - B. Initiate an orderly load reduction and have Main Steam Lines isolated within eight hours.
 - C. Isolate Reactor Water Cleanup System.
 - D. Isolate Shutdown Cooling.
3. Instrument set point corresponds to 177.7" above top of active fuel.
4. Instrument set point corresponds to 129.7" above top of active fuel.
5. Two required for each steam line.
6. These signals also start SBGTS and initiate secondary containment isolation.
7. Only required in Run Mode (interlocked with Mode Switch).
8. At a radiation level of 1.5 times the normal rated power background an alarm will be tripped in the control room to alert the control room operators to an increase in the main steam line tunnel radiation level.

T.S. Change #2

TABLE 3.2.B

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT COOLING SYSTEMS

Minimum No. of Operable Instrument Channels Per Trip System(1)	Trip Function	Trip Level Setting	Number of Instrument Channels Provided by Design	Remarks
2	Reactor Low-Low Water Level	≥-48 in. indicated level	4 HPCI & RCIC Inst. Channels	Initiates HPCI & RCIC
2	Reactor Low-Low-Low-Water Level	≥-160 in. indicated level (4)	4 Core Spray & RHR Instrument Channels 4 ADS Instrument Channels	<ol style="list-style-type: none"> 1. In conjunction with Low Reactor Pressure initiates Core Spray and LPCI 2. In conjunction with confirmatory low level High Drywell Pressure, 120 second time delay and LPCI or Core Spray pump interlock initiates Auto Blowdown (ADS) 3. Initiates starting of Diesel Generator

LIMITING CONDITIONS FOR OPERATION

3.7.A.6.c (Cont'd)

the affected reactor must be taken out of power operation.

- a. A 30 psig limit is the maximum containment repressurization allowable using the CAD system. Venting via the SBT system to this stack must be initiated at 30 psig following the initial peak pressure of 49.1 psig.

SURVEILLANCE REQUIREMENTS

4.7.A.6.c. (Cont'd)

- ** The CAD system H2 and O2 analyzers shall be tested for operability using standard bottled H2 and O2 once per month and shall be calibrated once per 6 months. The atmosphere analyzing system shall be functionally tested once per operating cycle in conjunction with specification 4.7.A.6.a. Should one of the two H2 or O2 analyzers serving the drywell or suppression pool be found inoperable the remaining analyzer of the same type serving the same compartment shall be tested for operability once per week until the defective analyzer is made operable.

PBAPS

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.7.A.7 If the specifications of 3.7.A.1 through 3.7.A.5 cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition within 24 hours..

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.9.B Operation with Inoperable
Equipment

4.9.B

Whenever the reactor is in Run Mode or Startup Mode with the reactor not in a Cold Condition, the availability of electric power shall be as specified in 3.9.A, except as follows:

1. From and after the date incoming power is not available from one startup or emergency transformer, continued reactor operation is permissible for seven days. During this period, the four diesel generators and associated emergency buses must be demonstrated to be operable.
2. From and after the date that incoming power is not available from both start-up or emergency transformers, continued operation is permissible, provided the four diesel generators and associated emergency buses are operable, all core and containment cooling systems are operable and reactor power level is reduced to 25% of the design.

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS

3.9.B (CONT'D)

4.9.B

3. From and after the date that one of the diesel generators or associated emergency bus is made or found to be inoperable for any reasons, continued reactor operation is permissible in accordance with Specification 3.5.F if Specification 3.9.A.1 is satisfied.
4. From and after the date that one of the diesel generators or associated emergency buses and either the emergency or startup transformer power source are made or found to be inoperable for any reason, continued reactor operation is permissible in accordance with Specification 3.5.F, provided the other off-site source, startup transformer and emergency transformer are available and capable of automatically supplying power to the 4KV emergency buses.
5. From and after the date that one of the 125 volt battery systems is made or found to be inoperable for any reason, continued reactor operation is permissible during the succeeding three days within electrical safety considerations, provided repair work is initiated in the most expeditious manner to return the failed component to an operable state, and Specification 3.5.F is satisfied.

LIMITING CONDITIONS FOR OPERATION3.10 CORE ALTERATIONSApplicability

Applies to the fuel handling and core reactivity limitations.

Objective

To ensure that core reactivity is within the capability of the control rods and to prevent criticality during refueling.

SpecificationA. Refueling Interlocks

1. The reactor mode switch shall be locked in the "Refuel" position during core alterations and the refueling interlocks shall be operable except as specified in 3.10.A.5 and 3.10.A.6 below.
2. Fuel shall not be loaded into the reactor core unless all control rods are fully inserted.

SURVEILLANCE REQUIREMENTS4.10 CORE ALTERATIONSApplicability

Applies to the periodic testing of those interlocks and instrumentation used during refueling and core alterations.

Objective

To verify the operability of instrumentation and interlocks used in refueling and core alterations.

SpecificationA. Refueling Interlocks

1. Prior to any core alterations within or over the reactor core, the reactor switch "Refuel" position interlocks shall be functionally tested. They shall be tested at weekly intervals thereafter until no longer required. They shall also be tested following any repair work associated with the interlocks.
2. Prior to performing control rod or control rod drive maintenance on control cells without removing fuel assemblies, it shall be demonstrated that the core can be made subcritical by a margin of 0.25 percent Δk at any time during the maintenance with the strongest operable control rod

LIMITING CONDITIONS FOR OPERATION

3.10.A Refueling Interlocks

3. The fuel grapple hoist load switch shall be set at ≤ 1000 lbs.
4. If the frame-mounted auxiliary hoist, the monorail-mounted auxiliary hoist, or the service platform hoist is to be used for handling fuel with the head off the reactor vessel, the load limit switch on the hoist to be used shall be set at ≤ 400 lbs.
5. A maximum of two nonadjacent control rods may be withdrawn from the core for the purpose of performing control rod and/or control rod drive maintenance, provided the following conditions are satisfied:
 - a. The reactor mode switch shall be locked in the "refuel" position. The refueling interlock which prevents more than one control rod from being withdrawn may be bypassed for one of the control rods on which maintenance is being performed. All other refueling interlocks shall be operable.
 - b. A sufficient number of control rods shall be operable so that the core can be made subcritical with the strongest operable control rod fully withdrawn and all other operable control rods fully inserted, or all

SURVEILLANCE REQUIREMENTS

4.10.A.2 (Cont'd)

fully withdrawn and all other operable rods fully inserted. Alternatively if the remaining control rods are fully inserted and have their directional control valves electrically disarmed, it is sufficient to demonstrate that the core is subcritical with a margin of at least $0.25\% \Delta k$ at any time during the maintenance. A control rod on which maintenance is being performed shall be considered inoperable.

PBAPS

LIMITING CONDITIONS FOR OPERATION

- 3.14.A (Cont'd)
- c. Turbine Building
 - d. Circulating Water Pump Structure
6. When a hose station serving an area which contains equipment which is required to be operable becomes inoperable, establish a continuous fire watch equipped with portable fire suppression equipment within 1 hour and provide equivalent protection to the area served by the inoperable station from the operable hose station within 6 hours.
7. Except as specified in 3.14.A.8 below, the fire suppression spray system serving a Standby Gas Treatment System charcoal filter train shall be operable when a train is required to be operable.
8. If the requirements of 3.14.A.7 cannot be met,
- a. establish a fire water patrol to inspect the area with inoperable fire suppression equipment at least once per shift.
 - b. restore the system to an operable status within 14 days, or in lieu of any other report required by Specification 6.9.2 submit a Special Report to the Commission pursuant to Specification 6.9.3 within 31 days outlining the cause of the malfunction and the plans for restoring the system to an operable status. The SGTS may be considered operable for the purposes of Specification 3.7.3.

SURVEILLANCE REQUIREMENTS

- c. Hose station valve operability and blockage check - once every 3 years.
 - d. Hose hydrostatic test at a pressure at least 50 psig greater than the maximum pressure available at that hose station but not less than 150 psig, or replace with an appropriately tested hose - every 3 years
6. None
7. The SGTS fire suppression spray system testing shall be performed as follows:
- a. Simulated automatic actuation test - once every 18 months
 - b. Inspection of nozzles and spray header - once every 18 months
 - c. Header and nozzle air flow test - once every 3 years

PEAPS

6.0 ADMINISTRATIVE CONTROLS

6.1 Responsibility

6.1.1 The Station Superintendent shall be responsible for overall facility operation. In the absence of the Station Superintendent, the Assistant Superintendent or the Engineer-Technical (or any other person that the Station superintendent may designate in writing) shall, in that order, assume the Superintendent's responsibility for overall facility operation.

6.2 Organization

Offsite

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

Facility Staff

6.2.2 The facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Figure 6.2-2, except that the shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
- b. At least one licensed operator shall be in the control room and assigned to each reactor that contains fuel.
- c. At least two licensed operators, excluding the operator on the second unit, shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibility during this operation.
- f. A Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of the unit(s) (3 members) or any personnel required for other essential functions during a fire emergency.

6.5.2 Operation and Safety Review CommitteeFunction

- 6.5.2.1 The Operation and Safety Review Committee shall function to provide independent review and audit of designated activities in the area of:
- a. nuclear power plant operations
 - b. nuclear engineering
 - c. chemistry and radiochemistry
 - d. metallurgy
 - e. instrumentation and control
 - f. radiological safety
 - g. mechanical and electrical engineering
 - h. quality assurance practices

The members of the OSR Committee will be competent in the area of quality assurance practice and cognizant of the Quality Assurance requirements of 10 CFR 50, Appendix B. Additionally, they will be cognizant of the corporate Quality Assurance Program and will have the corporate Quality Assurance Organization available to them.

Organization

- 6.5.2.2 The Chairman, Members and alternate Members of the OSR Committee shall be appointed in writing by the Vice President, Electric Production, and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of five years technical experience, of which a minimum of three years shall be in one or more areas given in 6.5.2.1.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-287

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 68
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated April 15, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 68, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 16, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 68

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

<u>Remove Pages</u>	<u>Insert Pages</u>
63	63
64	64
173	173
174	174
219	219
220	220
225	225
226	226
240f	240f
243	243
249	249

Page 63 is unchanged and is included for your convenience only.

PBAPS

NOTES FOR TABLE 3.2.A

1. Whenever Primary Containment integrity is required by Section 3.7, there shall be two operable or tripped trip systems for each function.
2. If the first column cannot be met for one of the trip systems, that trip system shall be tripped or the appropriate action listed below shall be taken.
 - A. Initiate an orderly shutdown and have the reactor in Cold Shutdown Condition in 24 hours.
 - B. Initiate an orderly load reduction and have Main Steam Lines isolated within eight hours.
 - C. Isolate Reactor Water Cleanup System.
 - D. Isolate Shutdown Cooling.
3. Instrument set point corresponds to 177.7" above top of active fuel.
4. Instrument set point corresponds to 129.7" above top of active fuel.
5. Two required for each steam line.
6. These signals also start SBGTS and initiate secondary containment isolation.
7. Only required in Run Mode (interlocked with Mode Switch).
8. At a radiation level of 1.5 times the normal rated power background an alarm will be tripped in the control room to alert the control room operators to an increase in the main steam line tunnel radiation level.

T.S. Change #2

TABLE 3.2.B

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT
COOLING SYSTEMS

Minimum No. of Operable Instrument Channels Per Trip System (1)	Trip Function	Trip Level Setting	Number of Instrument Channels Provided by Design	Remarks
2	Reactor Low-Low Water Level	≥-48 in. indicated level	4 HPCI & RCIC Inst. Channels	Initiates HPCI & RCIC
2	Reactor Low-Low-Low-Water Level	≥-160 in. indicated level (4)	4 Core Spray & RHR Instrument Channels	1. In conjunction with Low Reactor Pressure initiates Core Spray and LPCI
			4 ADS Instrument Channels	2. In conjunction with confirmatory low level High Drywell Pressure, 120 second time delay and LPCI or Core Spray pump interlock initiates Auto Blowdown (ADS)
				3. Initiates starting of Diesel Generators

LIMITING CONDITIONS FOR OPERATION

3.7.A.6.c (Cont'd)

the affected reactor must be taken out of power operation.

- d. A 30 psig limit is the maximum containment repressurization allowable using the CAD system. Venting via the SBT system to this stack must be initiated at 30 psig following the initial peak pressure of 49.1 psig.

SURVEILLANCE REQUIREMENTS

4.7.A.6.c. (Cont'd)

** The CAD system H₂ and O₂ analyzers shall be tested for operability using standard bottled H₂ and O₂ once per month and shall be calibrated once per 6 months. The atmosphere analyzing system shall be functionally tested once per operating cycle in conjunction with specification 4.7.A.6.a. Should one of the two H₂ or O₂ analyzers serving the drywell or suppression pool be found inoperable the remaining analyzer of the same type serving the same compartment shall be tested for operability once per week until the defective analyzer is made operable.

PBAPS

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.7.A.7 If the specifications of 3.7.A.1 through 3.7.A.5 cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition within 24 hours.

LIMITING CONDITIONS FOR OPERATION3.9.B Operation with Inoperable
Equipment

Whenever the reactor is in Run Mode or Startup Mode with the reactor not in a Cold Condition, the availability of electric power shall be as specified in 3.9.A, except as follows:

1. From and after the date incoming power is not available from one startup or emergency transformer, continued reactor operation is permissible for seven days. During this period, the four diesel generators and associated emergency buses must be demonstrated to be operable.
2. From and after the date that incoming power is not available from both start-up or emergency transformers, continued operation is permissible, provided the four diesel generators and associated emergency buses are operable, all core and containment cooling systems are operable and reactor power level is reduced to 25% of the design.

SURVEILLANCE REQUIREMENTS

4.9.B

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS

3.9.B (Cont'd)

4.9.B

3. From and after the date that one of the diesel generators or associated emergency bus is made or found to be inoperable for any reasons, continued reactor operation is permissible in accordance with Specification 3.5.F if Specification 3.9.A.1 is satisfied.
4. From and after the date that one of the diesel generators or associated emergency buses and either the emergency or startup transformer power source are made or found to be inoperable for any reason, continued reactor operation is permissible in accordance with Specification 3.5.F, provided the other off-site source, startup transformer and emergency transformer are available and capable of automatically supplying power to the 4KV emergency buses.
5. From and after the date that one of the 125 volt battery systems is made or found to be inoperable for any reason, continued reactor operation is permissible during the succeeding three days within electrical safety considerations, provided repair work is initiated in the most expeditious manner to return the failed component to an operable state, and Specification 3.5.F is satisfied.

LIMITING CONDITIONS FOR OPERATION3.10 COKE ALTERATIONSApplicability

Applies to the fuel handling and core reactivity limitations.

Objective

To ensure that core reactivity is within the capability of the control rods and to prevent criticality during refueling.

SpecificationA. Refueling Interlocks

1. The reactor mode switch shall be locked in the "Refuel" position during core alterations and the refueling interlocks shall be operable except as specified in 3.10.A.5 and 3.10.A.6 below.
2. Fuel shall not be loaded into the reactor core unless all control rods are fully inserted.

SURVEILLANCE REQUIREMENTS4.10 COKE ALTERATIONSApplicability

Applies to the periodic testing of those interlocks and instrumentation used during refueling and core alterations.

Objective

To verify the operability of instrumentation and interlocks used in refueling and core alterations.

SpecificationA. Refueling Interlocks

1. Prior to any core alterations within or over the reactor core, the reactor switch "Refuel" position interlocks shall be functionally tested. They shall be tested at weekly intervals thereafter until no longer required. They shall also be tested following any repair work associated with the interlocks.
2. Prior to performing control rod or control rod drive maintenance on control cells without removing fuel assemblies, it shall be demonstrated that the core can be made subcritical by a margin of 0.25 percent Δk at any time during the maintenance with the strongest operable control rod

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS

3.10.A Refueling Interlocks

4.10.A.2 (Cont'd)

3. The fuel grapple hoist load switch shall be set at ≤ 1000 lbs.
4. If the frame-mounted auxiliary hoist, the monorail-mounted auxiliary hoist, or the service platform hoist is to be used for handling fuel with the head off the reactor vessel, the load limit switch on the hoist to be used shall be set at ≤ 400 lbs.
5. A maximum of two nonadjacent control rods may be withdrawn from the core for the purpose of performing control rod and/or control rod drive maintenance, provided the following conditions are satisfied:
 - a. The reactor mode switch shall be locked in the "refuel" position. The refueling interlock which prevents more than one control rod from being withdrawn may be bypassed for one of the control rods on which maintenance is being performed. All other refueling interlocks shall be operable.
 - b. A sufficient number of control rods shall be operable so that the core can be made subcritical with the strongest operable control rod fully withdrawn and all other operable controls rods fully inserted, or all

fully withdrawn and all other operable rods fully inserted. Alternatively if the remaining control rods are fully inserted and have their directional control valves electrically disarmed, it is sufficient to demonstrate that the core is subcritical with a margin of at least $0.25\% \Delta k$ at any time during the maintenance. A control rod on which maintenance is being performed shall be considered inoperable.

PBAPS

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

- 3.14.A (Cont'd)
- c. Turbine Building
 - d. Circulating Water Pump Structure
6. When a hose station serving an area which contains equipment which is required to be operable becomes inoperable, establish a continuous fire watch equipped with portable fire suppression equipment within 1 hour and provide equivalent protection to the area served by the inoperable station from the operable hose station within 6 hours.
7. Except as specified in 3.14.A.8 below, the fire suppression spray system serving a Standby Gas Treatment System charcoal filter train shall be operable when a train is required to be operable.
8. If the requirements of 3.14.A.7 cannot be met,
- a. establish a fire water patrol to inspect the area with inoperable fire suppression equipment at least once per shift.
 - b. restore the system to an operable status within 14 days, or in lieu of any other report required by Specification 6.9.2 submit a Special Report to the Commission pursuant to Specification 6.9.3 within 31 days outlining the cause of the malfunction and the plans for restoring the system to an operable status. The SGTS may be considered operable for the purposes of Specification 3.7.3.
- c. Hose station valve operability and blockage check - once every 3 years.
- d. Hose hydrostatic test at a pressure at least 50 psig greater than the maximum pressure available at that hose station but not less than 150 psig, or replace with an appropriately tested hose-every 3 years
6. None
7. The SGTS fire suppression spray system testing shall be performed as follows:
- a. Simulated automatic actuation test - once every 18 months
 - b. Inspection of nozzles and spray header- once every 18 months
 - c. Header and nozzle air flow test - once every 3 years

6.0 ADMINISTRATIVE CONTROLS6.1 Responsibility

- 6.1.1 The Station Superintendent shall be responsible for overall facility operation. In the absence of the Station Superintendent, the Assistant Superintendent or the Engineer-Technical (or any other person that the Station superintendent may designate in writing) shall, in that order, assume the Superintendent's responsibility for overall facility operation.

6.2 OrganizationOffsite

- 6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

Facility Staff

- 6.2.2 The facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Figure 6.2-2, except that the shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
- b. At least one licensed operator shall be in the control room and assigned to each reactor that contains fuel.
- c. At least two licensed operators, excluding the operator on the second unit, shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibility during this operation.
- f. A Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of the unit(s) (3 members) or any personnel required for other essential functions during a fire emergency.

6.5.2 Operation and Safety Review Committee

Function

- 6.5.2.1 The Operation and Safety Review Committee shall function to provide independent review and audit of designated activities in the area of:
- a. nuclear power plant operations
 - b. nuclear engineering
 - c. chemistry and radiochemistry
 - d. metallurgy
 - e. instrumentation and control
 - f. radiological safety
 - g. mechanical and electrical engineering
 - h. quality assurance practices

The members of the OSR Committee will be competent in the area of quality assurance practice and cognizant of the Quality Assurance requirements of 10 CFR 50, Appendix B. Additionally, they will be cognizant of the corporate Quality Assurance Program and will have the corporate Quality Assurance Organization available to them.

Organization

- 6.5.2.2 The Chairman, Members and alternate Members of the OSR Committee shall be appointed in writing by the Vice President, Electric Production, and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of five years technical experience, of which a minimum of three years shall be in one or more areas given in 6.5.2.1.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKETS NOS. 50-277 AND 50-278PHILADELPHIA ELECTRIC COMPANY, ET AL.NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 69 and 68 to Facility Operating License Nos. DPR-44 and DPR-56, issued to Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company, which revised Technical Specifications for operation of the Peach Bottom Atomic Power Station, Units Nos. 2 and 3 (the facility) located in York County, Pennsylvania. The amendments are effective as of the date of issuance.

The amendments revise the Technical Specifications to (1) correct typographical errors, (2) delete obsolete notes regarding inerting makeup systems, (3) revise the specification regarding inoperable power supply equipment by deleting specific reporting requirements which are inherently specified by Administrative Controls and (4) clarify the NRC staff's requirements for reactor mode switch refuel position interlocks, station fire hose hydrostatic test pressure, and minimum shift crew composition requirements.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

8005270 509

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated April 15, 1980, (2) Amendments Nos. 69 and 68 to License Nos. DPR-44 and DPR-56, and (3) the Commission's letter to the licensee dated May 16, 1980. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 16th day of May, 1980.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing