Distribution
Docket
ORB #3
Local PDR
NRC PDR
VStello
KGoller
GLear
CParrish
DVerrelli
Attorney, OELD
OI&E (5)
BJones (8)
BScharf (10)
JMcGough
DEisenhut
ACRS (16)
OPA (CMiles)
DRoss
TBAbernathy
JRBuchanan

Dockets Nos. 50-277
and 50-278

FEBRUARY 28 1978

Philadelphia Electric Company
ATTN: Mr. Edward G. Bauer, Jr., Esquire
Vice President and General Counsel
2301 Market Street
Philadelphia, Pennsylvania 19101

Gentlemen:

The Commission has issued the enclosed Amendments Nos. 39 and 39 to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. The amendments revise the Technical Specifications and consist of: (1) addition of interim specifications on fire protection and (2) deletion of the respiratory protection program in accordance with the revocation provision of the current Technical Specifications. The implementation of fire protection specifications is in response to your applications dated February 2, 1977 and July 18, 1977.

By our letter dated November 23, 1977, we forwarded proposed interim Fire Protection Technical Specifications and a related Safety Evaluation. Your letter dated December 12, 1977 raised certain objections to the specifications as proposed by the staff. In order to achieve expeditious implementation of fire protection Technical Specifications, the enclosed amendment includes consideration of certain objections raised in your December 12, 1977 letter and have been issued pending completion of our review of your overall Fire Protection Program. These changes to the Peach Bottom Technical Specifications are supported by the Safety Evaluation issued with our letter of November 23, 1977 except for those changes which have resulted from your letter dated December 12, 1977. These issued specifications were discussed with members of your staff and they do not object. Resolution of each of your comments is discussed below:

1. Section 3.14A.4, A.9, B.5 and C.3. Editorial changes have been made to our original submittal to eliminate misinterpretation that could lead to duplication of reports. The revised verbage is consistent with the original intent of the staff requirements.

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DATE ➤						

2. Section 3.14A.6. You considered the one hour action time for an inoperable hose station as being inappropriate considering the time required to repair a hose station and the allowable 24 hour period for inoperability of the entire fire water system (Specification 3.14A.3.a). We have revised the specification as you requested but have added the requirement to establish a continuous fire watch equipped with portable fire suppression equipment within one hour. This revised specification provides a comparable level of safety as originally proposed, yet provides flexibility for repair of inoperable hose stations.

3. Section 4.14A.1.e.(1)(b). As you indicated, this specification was redundant to section 4.14A.1.a. Accordingly, these two specifications have been combined and the revised specification 4.14A.1.a is consistent with the staff requirement for demonstrating fire pump operability and for providing special surveillance of diesel engine pumps.

4. Section 4.14.A.1.b. On an interim basis we have revised our original specification for verifying that valves are in their correct position. Since certain valves may be located in areas of difficult accessibility or in high radiation areas, the revised specification includes in the 31-day verification of line up, only those unmonitored valves that are not locked, sealed or otherwise secured in position. The revised specification provides an incentive for either electrically monitoring valve position or for physically securing valves in their correct position, and maintains licensee flexibility to reduce occupational exposures to as low as is reasonably achievable. This revised specification is being reviewed as part of our evaluation of your overall fire protection program.

5. Section 3.14D.1. PECO prpposed a variable inspection inverval for fire barrier penetrations based on the percentage of barriers found non-functional during the previous interval. We have previously approved for other components the concept of variable inspection intervals (e.g. Snubber inspections); however, in those cases, technical judgment of providing a constant level of protection was based on experience at a large number of operating plants. In the case of fire barrier penetration, we have no related experience upon which to base a variable inspection schedule. Therefore, the enclosed amendment maintains an inspection interval of at least once per 18 months. The specification does not preclude PECo from submitting a request to change this specification at some future time when relevant experience is acquired.

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6. Figure 6.2-2. The footnote to this figure has been revised to reflect the Station Superintendent's responsibility for implementation of "on-site fire protection activities." A revised Figure 6.2-1 has been included to reflect the responsibility for implementation of the Over-all Fire Protection Program at the Corporate level.

7. Section 6.4.2. The requirement for fire brigade training has been revised to indicate that members complete an instruction program with a two year period and that regularly scheduled training sessions will be held at least once every three months. This revised requirement is consistent with NRC guidance contained in Attachment No. 2, "Fire Brigade Training" to Nuclear Plant Fire Protection Functional Responsibilities, Administrative Controls and Quality Assurance.

By our letter dated July 29, 1977, we advised you of an amended Section 20.103 of 10 CFR 20 which became effective on December 29, 1976. You were further advised that one effect of the revision is that in order for you to receive credit for use of respiratory protection equipment at your facility after December 28, 1977, such use must be as stipulated in Regulatory Guide 8.15 rather than as specified in your current Technical Specifications. In view of the provisions of Section 6.11 of your Technical Specifications which require conformance with 10 CFR 20, and in the absence of written objection from you, the amendment executes the revocation provision of your current specifications on respiratory protection by deleting Section 6.12.

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5 (d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities

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will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

A copy of the related FEDERAL REGISTER Notice is also enclosed.

Sincerely,

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:
1. Amendment No. to DPR-44
2. Amendment No. to DPR-56
3. FEDERAL REGISTER Notice

cc w/enclosures: See next page

*SEE PREVIOUS YELLOW FOR CONCURRENCES

OFFICE➤	ORB #3	ORB #3	ORB #1	OELD	ORB #3	
SURNAME➤	*CParrish	*DVerrelli:mjf	TWambach	*	GLear	
DATE➤	2/14/78	2/14/78	2/22/78	2/22/78	2/ /78	

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Sincerely,

Original signed by

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:
1. Amendment No. 39 to DPR-44
2. Amendment No. 39 to DPR-56
3. ~~FEDERAL REGISTER Notice~~

OFFICE➤		WRMPart	ORB #3	ORB #3	OELD	ORB #3
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DATE➤		2/22/78	2/14/78	2/14/78	2/22/78	2/28/78

Subject to Rule

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 28, 1978

Dockets Nos. 50-277
and 50-278

Philadelphia Electric Company
ATTN: Mr. Edward G. Bauer, Jr., Esquire
Vice President and General Counsel
2301 Market Street
Philadelphia, Pennsylvania 19101

Gentlemen:

The Commission has issued the enclosed Amendments Nos. 39 and 39 to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units Nos. 2 and 3. The amendments revise the Technical Specifications and consist of: (1) addition of interim specifications on fire protection and (2) deletion of the respiratory protection program in accordance with the revocation provision of the current Technical Specifications. The implementation of fire protection specifications is in response to your applications dated February 2, 1977 and July 18, 1977.

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5. Section 3.14D.1. PECO proposed a variable inspection interval for fire barrier penetrations based on the percentage of barriers found non-functional during the previous interval. We have previously approved for other components the concept of variable inspection intervals (e.g. Snubber inspections); however, in those cases, technical judgment of providing a constant level of protection was based on experience at a large number of operating plants. In the case of fire barrier penetration, we have no related experience upon which to base a variable inspection schedule. Therefore, the enclosed amendment maintains an inspection interval of at least once per 18 months. The specification does not preclude PECo from submitting a request to change this specification at some future time when relevant experience is acquired.

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We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities

will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

A copy of the related FEDERAL REGISTER Notice is also enclosed.

Sincerely,

George Lear

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:
1. Amendment No. 39 to DPR-44
2. Amendment No. 39 to DPR-56
3. FEDERAL REGISTER Notice

cc w/enclosures: See next page

cc:

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Board of Supervisors
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Delta, Pennsylvania 17314

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State Library of Pennsylvania
Education Building
Commonwealth and Walnut Streets
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 39
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Philadelphia Electric Company (the licensee) dated February 2, 1977, as revised by letter dated July 18, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 39, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment becomes effective 30 days after the date of its issuance.

George Lear

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 28, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 39

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment numbers and contain vertical lines indicating the area of change. Pages 265 through 267 have been deleted. Pages 240c through 240n are new pages.

Remove	Replace
iii	iii
vi	vi
-	240c (new)
-	240d (new)
-	240e (new)
-	240f (new)
-	240g (new)
-	240h (new)
-	240i (new)
-	240j (new)
-	240k (new)
-	240l (new)
-	240m (new)
243	243
244	244
245	245
246	246
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262	262
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266 (deleted)	-
267 (deleted)	-

PBAPS

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C. Fire Detection	C	240i
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Amendment No. 39

PBAPS

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LIMITING CONDITIONS FOR OPERATION

3.14 FIRE PROTECTION

Applicability:

Applies to the operational status of the Fire Protection Systems.

Objective:

To assure the availability of the Fire Protection Systems.

Specifications:

A. Water Fire Protection System

1. Two fire pumps with their discharge aligned to the fire main and the automatic initiation logic for each pump shall be operable except as specified in 3.14.A.2 and 3.14.A.3 below. This shall include an operable flow path of transferring water through distribution piping with OPERABLE sectionalizing control or isolation valves to the yard hydrant curb valves and the valve ahead of the water flow alarm device on each sprinkler, hose, standpipe or spray system riser.

SURVEILLANCE REQUIREMENTS

4.14 FIRE PROTECTION

Applicability:

Applies to the surveillance requirements of the Fire Protection Systems.

Objective:

To verify the operability of the Fire Protection Systems.

Specification:

A. Water Fire Protection System

1. The Water Fire Protection System testing shall be performed as follows:

a. Pump operability - Each type of fire pump shall be started from ambient conditions and operated for at least 20 minutes on recirculation flow at least every 31 days such that a pump is tested approximately every 15 days.

b. Verification that each unmonitored valve (manual, power operated or automatic) in the flow path that is not locked sealed or otherwise secured in position is in its correct position - once each 31 days.

c. Pump flow rate test and automatic initiation logic check - once every 18 months. Each pump shall deliver at least 2700 gpm at a pressure of 105 psig.

d. Underground fire main flow test - once every 3 years.

e. Special surveillance of diesel engine.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
	(1) At least once per 31 days by verifying that the underground storage tank contains at least 300 gallons of fuel. (2) At least once per 92 days by verifying that a sample of diesel fuel from the fuel storage tank, obtained in accordance with ASTM-D270-65, is within the acceptable limits specified in Table 1 of ASTM-D975-74 with respect to viscosity, water content and sediment. (3) At least once per 18 months, by subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for the class of service. (4) At least once per 7 days by verifying that: (a) The electrolyte level of each battery is above the plates, and (b) The overall battery voltage is $\geq$24 volts. (5) At least once per 92 days by verifying that the specific gravity is appropriate for continued service of the battery. (6) At least once per 18 months by verifying that: (a) The batteries, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration, and (b) The battery-to-battery and terminal connections are clean, tight, free of corrosion and coated with anti-corrosion material.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
2. With one fire pump or logic inoperable, restore the equipment to an operable status within 7 days, or in lieu of any other report required by Specification 6.9.2, submit a Special Report to the Commission pursuant to Specification 6.9.3 within 31 days outlining the cause of the malfunction and the plans for restoring the equipment to an operable status. Reactor startup and/or continued reactor operation is permissible.	2. With one fire pump inoperable, the remaining fire pump shall be demonstrated to be operable immediately and at least every 72 hours thereafter until the inoperable pump is restored to an operable status.
3. With two fire pumps inoperable, a. establish a back-up water supply within 24 hours, b. notify the Commission pursuant to Specification 6.9.2.a within 24 hours, by telephone and in writing no later than the first working day following the event. Submit a report within 14 days outlining the actions taken and the plans and schedule for restoring the equipment to an operable status, and c. restore the equipment to an operable status within 14 days. d. If a. above cannot be fulfilled, place the reactor in Hot Standby within the next six hours and in Cold Shutdown within the following thirty hours.	3. None
4. Except as specified in 3.14.A.6 below, the fire hose stations serving the following structures shall be operable: a. Reactor Buildings b. Radwaste Building	4. The fire most station inspections shall be performed as follows: a. Visual inspection of hose station equipment availability - once every 31 days. b. Hose and gasket inspection - once every 18 months.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.14.A (Cont'd) c. Turbine Building d. Circulating Water Pump Structure	c. Hose station valve operability and blockage check - once every 3 years. d. Hose hydrostatic test at a pressure of 250 psig or replace with an appropriately tested hose every 3 years.
6. When a hose station serving an area which contains equipment which is required to be operable becomes inoperable, establish a continuous fire watch equipped with portable fire suppression equipment within 1 hour and provide equivalent protection to the area served by the inoperable station from the operable hose station within 6 hours.	6. None
7. Except as specified in 3.14.A.8 below, the fire suppression spray system serving a Standby Gas Treatment System charcoal filter train shall be operable when a train is required to be operable.	7. The SGTS fire suppression spray system testing shall be performed as follows: a. Simulated automatic actuation test- once every 18 months. b. Inspection of nozzles and spray header - once every 18 months. c. Header and nozzle air flow test - once every 3 years.
8. If the requirements of 3.14.A.7 cannot be met, a. establish a fire water partol to inspect the area with inoperable fire suppresssion equipment at least once per shift, b. restore the system to an operable status within 14 days, or in lieu of any other report required by Specification 6.9.2 submit a Special Report to the Commission pursuant to Specification 6.9.3 within 31 days outlining the cause of the malfunction and the plans for restoring the system to an operable status. The SGTS may be considered operable the purposes of Specification 3.7.B.	

LIMITING CONDITIONS FOR OPERATION

3.14.B CO2 Fire Protection Systems

1. The HPCI room CO2 Fire Protection System shall be operable when the HPCI system is required to be operable with the system comprised of:

   a. a minimum inventory of 2400 pounds of CO2 and a minimum pressure of 280 psig in the CO2 storage tank,

   b. an operable flow path to the HPCI room and

   c. three heat detectors, except that one detector may be inoperable for a period not to exceed 7 days.

2. The CO2 Fire Protection System serving the Control Room, Cable Spreading Room, and Computer Room shall be operable with the system comprised of:

   a. a minimum inventory of 11,000 pounds of CO2 and a minimum pressure of 280 psig in the CO2 storage tank(s) and

   b. an operable flow path to each room.

3. The Diesel Generator CO2 Fire Protection System shall be operable when the Diesel Generators are required to be operable with the system comprised of:

   a. a minimum inventory of 2200 pounds of CO2 and a minimum pressure of 280 psig in the CO2 storage tank,

SURVEILLANCE REQUIREMENTS

4.14.B. CO2 Fire Protection Systems

1. The CO2 Fire Protection Systems testing shall be performed as follows:

   a. CO2 storage tank level and pressure-checked once every 7 days.

   b. Simulated actuation test of valves, dampers, fans - once every 18 months.

   c. Header and nozzle air flow test - once every 18 months.

   d. Heat detector functional test - once every 6 months.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.14.B.3 (Cont'd) b. an operable flow path to each room and c. four heat detectors, except that one detector may be inoperable for a period not to exceed 7 days. 4. If the requirements of 3.14.B.1, 2, or 3 cannot be met, a. establish a continuous fire watch with back-up fire suppression equipment for the unprotected area (HPCI, Cable Spreading, Computer, Diesel Generator) within 1 hour b. restore the system to an operable status within 14 days, or in lieu of any other report required by Specification 6.8.2, submit a Special Report to the Commission pursuant to Specification 6.9.3 within 31 days outlining the cause of the malfunction and the plans for restoring the system to an operable systus. Reactor startup and/or continued reactor operation is permissible.	

LIMITING CONDITIONS FOR OPERATION

3.14.C Fire Detection

1. The fire detection instrumentation for each plant area listed in Table 3.14.C.1 shall be operable when the equipment in that area is required to be operable.

2. If the requirements of 3.14.C.1 cannot be met,

   a. establish a fire watch patrol to inspect each accessible area at intervals of at least:

      (1) Once per shift for areas with less than the minimum number of operable instruments required by Table 3.14.C.1 but with at least one instrument operable

      (2) Once every hour for areas without an operable instrument,

   b. restore accessible system components to an operable status within 14 days, or in lieu of any other report required by Specification

   c. 6.9.2, submit a Special Report to the Commisison pursuant to Specification 6.9.3 within 31 days outlining the cause of the malfunction and the plans for restoring the instruments to an operable status. Reactor startup and/or continued reactor operation is permissible.

SURVEILLANCE REQUIREMENTS

4.14.C Fire Detection

1. a. The smoke detectors listed in Table 3.14.C.1 shall be functionally tested semi-annually in accordance with the manufacturer's instructions.

   b. The heat detectors listed in Table 3.14.C.1 shall be functionally tested semi-annually with a heat source.

   c. The NFPA Code 72D Class A supervised circuits between the local panel and control room of each of the above required fire detection instruments shall be demonstrated OPERABLE at least once per 6 months.

2. The testing interval for smoke and heat detectors which are inaccessible due to high radiation or inerting may be extended until such time as the detectors become accessible for a minimum of 36 hours. Such detectors shall be functionally tested at a maximum interval of once per refueling cycle.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.14.D Fire Barrier Penetrations	4.14.d Fire Barrier Penetrations
1. Penetration fire barriers protecting the following areas shall be functional: a. Cable Spreading Room b. Emergency Switchgear Rooms c. Diesel Generator Rooms d. Battery Room e. Control Room	1. Visual inspection of penetration fire barriers shall be performed following repairs or maintenance and at least once per 18 months*.
2. If the requirements of 3.14.D.1 cannot be met, establish a continuous fire watch on at least one side of the affected penetration within 1 hour. Reactor startup and/or continued reactor operation is permissible.	
	*The initial inspection shall be performed within 18 months of the date of issuance of this amendment.

Table 3.14.C.1

FIRE DETECTORS

Location	Detector Type/ Designation (1)	Minimum Detectors Operable
Unit 2:		
Primary Containment (2) (3)	S1, S2, S8	3
Recirculation Pump MG Set Room	S15, S16, S17 S18, S19, S20	5
Emergency Switchgear Rooms	S11, S12, S13 S14	1 per room
Unit 3:		
Primary Containment (2)(3)	S103, S104, S106	3
Recirculation Pump MG Set Room	S111, S112, S113 S114, S116, S117	5
Emergency Switchgear Rooms	S107, S108, S109 S110	1 per room
Common:		
Control Room	S21, S22, S23, S23	4
Cable Spreading Room	S4, S7, S9, S10	4
Computer Room	S5, S6	2
Laboratory Area	H1, H2, H3, H4	4
Fan Area	S3, S105	2
Emergency Cooling Tower Switchgear Rooms	H562, H563, H564 H565	1 per room
HPSW Pump Structure	H397, H398	2
Recombiner Building	H566, H567, H568, H569	3
Start-up Switchgear Building	H558, H559 H560, H561	2

(1) S = Smoke Detector H = Heat Detector

(2) Detector(s) inaccessible during normal operation due to inerting.

(3) May be disabled during ILRT.

3.14 BASES

The Water and CO2 Fire Protection Systems, although not classified as safety related systems, provide fire suppression capabilities in those areas of the plant where protection of plant equipment is deemed necessary.

A. Water Fire Protection System

Two fire pumps supply water to sprinklers, manual hose stations, and hydrants in or surrounding the plant. One electrically driven pump is powered from an emergency power bus; the other pump is diesel driven. The capacity of each pump is in excess of the system design load.

In the event that both fire pumps become inoperable, immediate corrective measures are taken since this system is a major portion of the fire suppression capability of the plant. The requirement for a twenty-four hour report to the Commission provides for prompt evaluation of the acceptability of the corrective measures to provide adequate fire suppression capability for the continued protection of the plant.

B. CO2 Fire Protection Systems

The CO2 Fire Protection Systems provide fire suppression capability for the Cable Spreading Room and Computer Room (manual actuation), Control Room (manual hose reels), HPCI Rooms (automatic actuation), and the Diesel Generator Rooms (automatic actuation). The specified minimum quantities of CO2 provide the capability to flood the Cable Spreading Room and Computer Room simultaneously, a HPCI Room, or a Diesel Generator Room with sufficient CO2 to meet concentration objectives.

In the event that portions of the CO2 Fire Protection System are inoperable, alternate backup fire fighting equipment is required to be made available in the affected areas until the affected fire suppression equipment can be returned to service.

C. Fire Detection

Operability of the fire detectors ensures that adequate warning is available for the prompt detection of fires. This capability is required in order to detect and locate fires in their early stages. Prompt detection of fires will reduce the potential for damage to plant equipment and is an integral element in the overall plant fire protection program.

In the event that a portion of the fire detection instrumentation is inoperable, the establishment of fire patrols in the accessible affected areas is required to provide detection capability until the inoperable instrumentation is returned to service.

D. Fire Barrier Penetrations

The functional integrity of the fire barrier penetration seal ensures that fires will be confined or adequately retarded from spreading to adjacent portions of the facility. This design feature minimizes the possibility of a single fire rapidly involving several areas of the facility prior to detection and extinguishment. The fire barrier penetration seals are a passive element in the facility fire protection program and are subject to periodic inspections.

During periods of time when the seals are not functional, a continuous fire watch is required to be maintained in the vicinity of the affected seal until the seal is restored to functional status.

3.14 BASES

A. Water Fire Protection System

The monthly test of the fire pumps is conducted to check for equipment failures and deterioration. The fire pump minimum capacity is based on a design load of 2400 gpm for the largest sprinkler plus 300 gpm for manual hose lines.

When it is determined that a fire pump is inoperable, the increased surveillance required by 4.14.A.2 provides adequate assurance that the remaining pump will be operable when required.

B. CO2 Fire Protection Systems

Weekly checking of the storage tank level and pressure is deemed adequate to provide assurance that sufficient CO2 will be available in the event of a fire occurrence.

Semi-annual testing of the heat detectors in the automatic discharge systems is in accordance with NFPA-72B-1974.

Testing of the discharge initiation logic, injection valve, damper closings, and fan trippings without actual discharge of CO2 into a room demonstrates operability of the active components of the systems. System operability is demonstrated by both manual and automatic initiation for automatic discharge systems (HPCI and diesel generators). Testing of the headers and nozzles by an air flow test will detect buildups of material which may affect continued availability.

C. Fire Detection

Semi-annual testing of fire detectors is in accordance with NPFA-72B-1974.

D. Fire Barrier Penetrations

Penetration fire barrier seals are visually inspected to verify that they are functional.

6.0 ADMINISTRATIVE CONTROLS

6.1 Responsibility

6.1.1 The Station Superintendent shall be responsible for overall facility operation. In the absence of the Station Superintendent, the Assistant Superintendent or the Engineer-Technical (or any other person that the Station Superintendent may designate in writing) shall, in that order, assume the Superintendent's responsibility for overall facility operation.

6.2 Organization

Offsite

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

Facility Staff

6.2.2 The facility organization shall be as shown on Figure 6.2-2 and:

a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Figure 6.2-2.

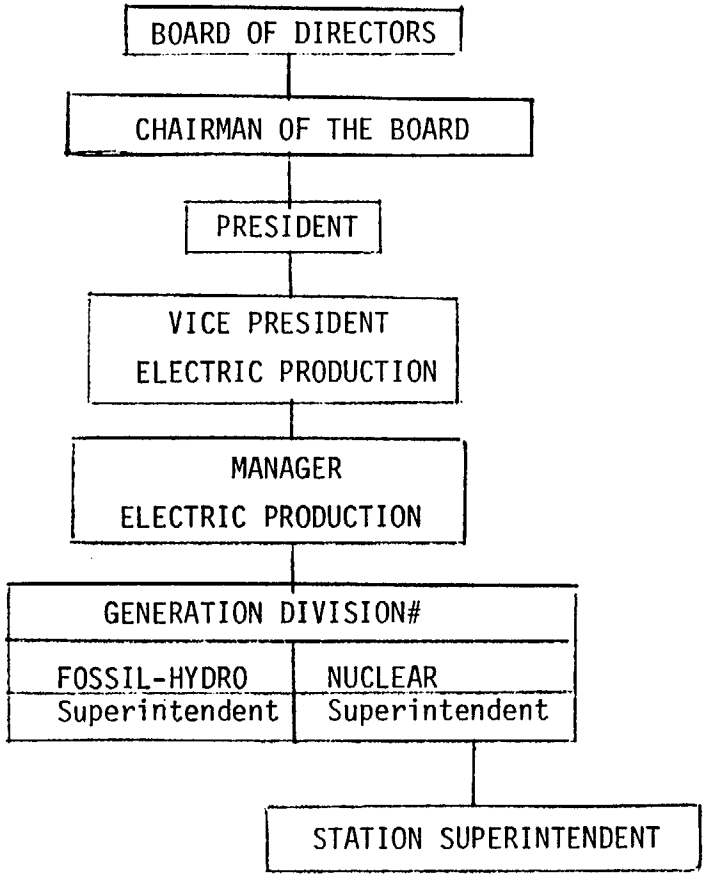
b. At least one licensed operator shall be in the control room and assigned to each reactor that contains fuel.

c. At least two licensed operators, excluding the operator on the second unit, shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.

d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.

e. ALL CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibility during this operation.

f. A Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of the unit(s) (3 members) or any personnel required for other essential functions during a fire emergency.



- Responsible for overall Fire Protection Program

PHILADELPHIA ELECTRIC COMPANY
PEACH BOTTOM ATOMIC POWER STATION
UNITS 2 and 3

MANAGEMENT ORGANIZATION
CHART

FIGURE 6.2-1

Superintendent SLO*#

Asst. Superintendent SLO*

- 1/S Shift Supt (SLO)**
  - Clerical
  - Guards
- Engineering Technical (SLO)
  - Engineer Reactor
  - Engineer I & C (or Results)
  - Engineer Health Physics & Chemistry
  - Engineers
  - Technicians
- Engineer Maintenance
  - Maintenance Foreman
    - Mechanics
  - Custodial
  - Stores

BWRs

1/S Shift Supr. (SLO)**

1/S Control Op. Nuclear (LO)

2/S Asst. Cont. Op. Nuclear (LO)

1/S Plant Op.

1/S Asst. Plant Op.

1/S Aux. Op.

LEGEND:

* - At least one of these shall be licensed.

** - Only one of these need to be licensed during a training and qualification period, provided a second SLO is available on site.

x/S - Number of men per shift. All shift positions are marked.

LO - NRC Licensed Operator

SLO - NRC Licensed Senior Operator

\# - Responsible for on site fire protection activities

ORGANIZATION FOR CONDUCT OF PLANT OPERATIONS

FIGURE 6.2-2

6.3 Facility Staff Qualifications

6.3.1 Each member of the facility staff shall meet the minimum qualifications of ANSI N18.1-1971 for comparable positions.

6.4 Training

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Station Superintendent and shall meet the requirements of Section 5.5 of ANSI N18.1-1971 and 10 CFR 55, Appendix A.

6.4.2 A training program for the Fire Brigade shall be conducted such that Fire Brigade members complete an instruction program within a two year period. Regularly planned meetings will be held every 3 months.

6.5 Review and Audit

6.5.1 Plant Operation Review Committee (PORC)

Function

6.5.1.1 The Plant Operation Review Committee shall function to advise the Station Superintendent on all matters related to nuclear safety.

Composition

6.5.1.2 The Plant Operation Reivew Committee shall be composed of the:

Station Superintendent-Chairman
Station Assistant Superintendent
Engineer - Technical
Engineer - Maintenance
Engineer - Operations
Engineer - Results
Engineer - Reactor
Engineer - Instrument & Control
Engineer - Health Physics & Chemistry
Shift Superintendent

Alternates

6.5.1.3 Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in PORC activities at any one time.

6.10.2 Continued

d. Records of radiation exposure for all individuals entering radiation control areas.

e. Records of gaseous and liquid radioactive material released to the environs.

f. Records of transient or operational cycles for those facility components designed for a limited number of transients or cycles.

g. Records of training and qualification for current members of the plant staff.

h. Records of in-service inspections performed pursuant to these Technical Specifications.

i. Records of Quality Assurance activities required by the QA Manual, except as described in 6.10.1 above.

j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.

k. Records of meetings of the PORC and the OSR Committee.

6.11 Radiation Protection Program

Procedures for personnel radiation protection shall be prepared consistent with requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 Fire Protection Inspections

a. An independent fire protection and loss prevention program inspection shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.

b. An inspection of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.

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6.13 High Radiation Area

6.13.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c) (2) of 10 CFR 20:

a. Each High Radiation Area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 39
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Philadelphia Electric Company (the licensee) dated February 2, 1977, as revised by letter dated July 18, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 39 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment becomes effective 30 days after the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 28, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 39

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment numbers and contain vertical lines indicating the area of change. Pages 265 through 267 have been deleted. Pages 240c through 240n are new pages.

Remove	Replace
iii	iii
vi	vi
-	240c (new)
-	240d (new)
-	240e (new)
-	240f (new)
-	240g (new)
-	240h (new)
-	240i (new)
-	240j (new)
-	240k (new)
-	240l (new)
-	240m (new)
243	243
244	244
245	245
246	246
261	261
262	262
263	263
265 (deleted)	-
266 (deleted)	-
267 (deleted)	-

PBAPS

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	A. Water Fire Protection System	A	240c
	B. Co_2 Fire Protection Systems	B	240g
	C. Fire Detection	C	240i
	D. Fire Barrier Penetrations	D	240j
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PBAPS

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3.11.D.1	Safety Related Shock Suppressors (Snubbers)	234d
3.14.C.1	Fire Detectors	240k

LIMITING CONDITIONS FOR OPERATION

3.14 FIRE PROTECTION

Applicability:

Applies to the operational status of the Fire Protection Systems.

Objective:

To assure the availability of the Fire Protection Systems.

Specifications:

A. Water Fire Protection System

1. Two fire pumps with their discharge aligned to the fire main and the automatic initiation logic for each pump shall be operable except as specified in 3.14.A.2 and 3.14.A.3 below. This shall include an operable flow path of transferring water through distribution piping with OPERABLE sectionalizing control or isolation valves to the yard hydrant curb valves and the valve ahead of the water flow alarm device on each sprinkler, hose, standpipe or spray system riser.

SURVEILLANCE REQUIREMENTS

4.14 FIRE PROTECTION

Applicability:

Applies to the surveillance requirements of the Fire Protection Systems.

Objective:

To verify the operability of the Fire Protection Systems.

Specification:

A. Water Fire Protection System

1. The Water Fire Protection System testing shall be performed as follows:

a. Pump operability - Each type of fire pump shall be started from ambient conditions and operated for at least 20 minutes on recirculation flow at least every 31 days such that a pump is tested approximately every 15 days.

b. Verification that each unmonitored valve (manual, power operated or automatic) in the flow path that is not locked sealed or otherwise secured in position is in its correct position - once each 31 days.

c. Pump flow rate test and automatic initiation logic check - once every 18 months. Each pump shall deliver at least 2700 gpm at a pressure of 105 psig.

d. Underground fire main flow test - once every 3 years.

e. Special surveillance of diesel engine.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
	(1) At least once per 31 days by verifying that the underground storage tank contains at least 300 gallons of fuel. (2) At least once per 92 days by verifying that a sample of diesel fuel from the fuel storage tank, obtained in accordance with ASTM-D270-65, is within the acceptable limits specified in Table 1 of ASTM-D975-74 with respect to viscosity, water content and sediment. (3) At least once per 18 months, by subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for the class of service. (4) At least once per 7 days by verifying that: (a) The electrolyte level of each battery is above the plates, and (b) The overall battery voltage is $\geq$24 volts. (5) At least once per 92 days by verifying that the specific gravity is appropriate for continued service of the battery. (6) At least once per 18 months by verifying that: (a) The batteries, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration, and (b) The battery-to-battery and terminal connections are clean, tight, free of corrosion and coated with anti-corrosion material.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
2. With one fire pump or logic inoperable, restore the equipment to an operable status within 7 days, or in lieu of any other report required by Specification 6.9.2, submit a Special Report to the Commission pursuant to Specification 6.9.3 within 31 days outlining the cause of the malfunction and the plans for restoring the equipment to an operable status. Reactor startup and/or continued reactor operation is permissible.	2. With one fire pump inoperable, the remaining fire pump shall be demonstrated to be operable immediately and at least every 72 hours thereafter until the inoperable pump is restored to an operable status.
3. With two fire pumps inoperable, a. establish a back-up water supply within 24 hours, b. notify the Commission pursuant to Specification 6.9.2.a within 24 hours, by telephone and in writing no later than the first working day following the event. Submit a report within 14 days outlining the actions taken and the plans and schedule for restoring the equipment to an operable status, and c. restore the equipment to an operable status within 14 days. d. If a. above cannot be fulfilled, place the reactor in Hot Standby within the next six hours and in Cold Shutdown within the following thirty hours.	3. None
4. Except as specified in 3.14.A.6 below, the fire hose stations serving the following structures shall be operable: a. Reactor Buildings b. Radwaste Building	4. The fire most station inspections shall be performed as follows: a. Visual inspection of hose station equipment availability - once every 31 days. b. Hose and gasket inspection - once every 18 months.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.14.A (Cont'd) c. Turbine Building d. Circulating Water Pump Structure	c. Hose station valve operability and blockage check - once every 3 years. d. Hose hydrostatic test at a pressure of 250 psig or replace with an appropriately tested hose every 3 years.
6. When a hose station serving an area which contains equipment which is required to be operable becomes inoperable, establish a continuous fire watch equipped with portable fire suppression equipment within 1 hour and provide equivalent protection to the area served by the inoperable station from the operable hose station within 6 hours.	6. None
7. Except as specified in 3.14.A.8 below, the fire suppression spray system serving a Standby Gas Treatment System charcoal filter train shall be operable when a train is required to be operable.	7. The SGTS fire suppression spray system testing shall be performed as follows: a. Simulated automatic actuation test- once every 18 months. b. Inspection of nozzles and spray header - once every 18 months. c. Header and nozzle air flow test - once every 3 years.
8. If the requirements of 3.14.A.7 cannot be met, a. establish a fire water partol to inspect the area with inoperable fire suppresssion equipment at least once per shift, b. restore the system to an operable status within 14 days, or in lieu of any other report required by Specification 6.9.2 submit a Special Report to the Commission pursuant to Specification 6.9.3 within 31 days outlining the cause of the malfunction and the plans for restoring the system to an operable status. The SGTS may be considered operable the purposes of Specification 3.7.B.	

LIMITING CONDITIONS FOR OPERATION

3.14.B CO2 Fire Protection Systems

1. The HPCI room CO2 Fire Protection System shall be operable when the HPCI system is required to be operable with the system comprised of:

   a. a minimum inventory of 2400 pounds of CO2 and a minimum pressure of 280 psig in the CO2 storage tank,

   b. an operable flow path to the HPCI room and

   c. three heat detectors, except that one detector may be inoperable for a period not to exceed 7 days.

2. The CO2 Fire Protection System serving the Control Room, Cable Spreading Room, and Computer Room shall be operable with the system comprised of:

   a. a minimum inventory of 11,000 pounds of CO2 and a minimum pressure of 280 psig in the CO2 storage tank(s) and

   b. an operable flow path to each room.

3. The Diesel Generator CO2 Fire Protection System shall be operable when the Diesel Generators are required to be operable with the system comprised of:

   a. a minimum inventory of 2200 pounds of CO2 and a minimum pressure of 280 psig in the CO2 storage tank,

SURVEILLANCE REQUIREMENTS

4.14.B. CO2 Fire Protection Systems

1. The CO2 Fire Protection Systems testing shall be performed as follows:

   a. CO2 storage tank level and pressure-checked once every 7 days.

   b. Simulated actuation test of valves, dampers, fans - once every 18 months.

   c. Header and nozzle air flow test - once every 18 months.

   d. Heat detector functional test - once every 6 months.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.14.B.3 (Cont'd) b. an operable flow path to each room and c. four heat detectors, except that one detector may be inoperable for a period not to exceed 7 days. 4. If the requirements of 3.14.B.1, 2, or 3 cannot be met, a. establish a continuous fire watch with back-up fire suppression equipment for the unprotected area (HPCI, Cable Spreading, Computer, Diesel Generator) within 1 hour b. restore the system to an operable status within 14 days, or in lieu of any other report required by Specification 6.8.2, submit a Special Report to the Commission pursuant to Specification 6.9.3 within 31 days outlining the cause of the malfunction and the plans for restoring the system to an operable status. Reactor startup and/or continued reactor operation is permissible.	

LIMITING CONDITIONS FOR OPERATION

3.14.C Fire Detection

1. The fire detection instrumentation for each plant area listed in Table 3.14.C.1 shall be operable when the equipment in that area is required to be operable.

2. If the requirements of 3.14.C.1 cannot be met,

   a. establish a fire watch patrol to inspect each accessible area at intervals of at least:

      (1) Once per shift for areas with less than the minimum number of operable instruments required by Table 3.14.C.1 but with at least one instrument operable

      (2) Once every hour for areas without an operable instrument,

   b. restore accessible system components to an operable status within 14 days, or in lieu of any other report required by Specification 6.9.2, submit a Special Report to the Commission pursuant to Specification 6.9.3 within 31 days outlining the cause of the malfunction and the plans for restoring the instruments to an operable status. Reactor startup and/or continued reactor operation is permissible.

SURVEILLANCE REQUIREMENTS

4.14.C Fire Detection

1. a. The smoke detectors listed in Table 3.14.C.1 shall be functionally tested semi-annually in accordance with the manufacturer's instructions.

   b. The heat detectors listed in Table 3.14.C.1 shall be functionally tested semi-annually with a heat source.

   c. The NFPA Code 72D Class A supervised circuits between the local panel and control room of each of the above required fire detection instruments shall be demonstrated OPERABLE at least once per 6 months.

2. The testing interval for smoke and heat detectors which are inaccessible due to high radiation or inerting may be extended until such time as the detectors become accessible for a minimum of 36 hours. Such detectors shall be functionally tested at a maximum interval of once per refueling cycle.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.14.D Fire Barrier Penetrations	4.14.d Fire Barrier Penetrations
1. Penetration fire barriers protecting the following areas shall be functional: a. Cable Spreading Room b. Emergency Switchgear Rooms c. Diesel Generator Rooms d. Battery Room e. Control Room	1. Visual inspection of penetration fire barriers shall be performed following repairs or maintenance and at least once per 18 months*.
2. If the requirements of 3.14.D.1 cannot be met, establish a continuous fire watch on at least one side of the affected penetration within 1 hour. Reactor startup and/or continued reactor operation is permissible.	
	*The initial inspection shall be performed within 18 months of the date of issuance of this amendment.

Table 3.14.C.1

FIRE DETECTORS

Location	Detector Type/ Designation (1)	Minimum Detectors Operable
Unit 2:		
Primary Containment (2) (3)	S1, S2, S8	3
Recirculation Pump MG Set Room	S15, S16, S17 S18, S19, S20	5
Emergency Switchgear Rooms	S11, S12, S13 S14	1 per room
Unit 3:		
Primary Containment (2)(3)	S103, S104, S106	3
Recirculation Pump MG Set Room	S111, S112, S113 S114, S116, S117	5
Emergency Switchgear Rooms	S107, S108, S109 S110	1 per room
Common:		
Control Room	S21, S22, S23, S23	4
Cable Spreading Room	S4, S7, S9, S10	4
Computer Room	S5, S6	2
Laboratory Area	H1, H2, H3, H4	4
Fan Area	S3, S105	2
Emergency Cooling Tower Switchgear Rooms	H562, H563, H564 H565	1 per room
HPSW Pump Structure	H397, H398	2
Recombiner Building	H566, H567, H568, H569	3
Start-up Switchgear Building	H558, H559 H560, H561	2

(1) S = Smoke Detector H = Heat Detector

(2) Detector(s) inaccessible during normal operation due to inerting.

(3) May be disabled during ILRT.

3.14 BASES

The Water and CO2 Fire Protection Systems, although not classified as safety related systems, provide fire suppression capabilities in those areas of the plant where protection of plant equipment is deemed necessary.

A. Water Fire Protection System

Two fire pumps supply water to sprinklers, manual hose stations, and hydrants in or surrounding the plant. One electrically driven pump is powered from an emergency power bus; the other pump is diesel driven. The capacity of each pump is in excess of the system design load.

In the event that both fire pumps become inoperable, immediate corrective measures are taken since this system is a major portion of the fire suppression capability of the plant. The requirement for a twenty-four hour report to the Commission provides for prompt evaluation of the acceptability of the corrective measures to provide adequate fire suppression capability for the continued protection of the plant.

B. CO2 Fire Protection Systems

The CO2 Fire Protection Systems provide fire suppression capability for the Cable Spreading Room and Computer Room (manual actuation), Control Room (manual hose reels), HPCI Rooms (automatic actuation), and the Diesel Generator Rooms (automatic actuation). The specified minimum quantities of CO2 provide the capability to flood the Cable Spreading Room and Computer Room simultaneously, a HPCI Room, or a Diesel Generator Room with sufficient CO2 to meet concentration objectives.

In the event that portions of the CO2 Fire Protection System are inoperable, alternate backup fire fighting equipment is required to be made available in the affected areas until the affected fire suppression equipment can be returned to service.

C. Fire Detection

Operability of the fire detectors ensures that adequate warning is available for the prompt detection of fires. This capability is required in order to detect and locate fires in their early stages. Prompt detection of fires will reduce the potential for damage to plant equipment and is an integral element in the overall plant fire protection program.

In the event that a portion of the fire detection instrumentation is inoperable, the establishment of fire patrols in the accessible affected areas is required to provide detection capability until the inoperable instrumentation is returned to service.

D. Fire Barrier Penetrations

The functional integrity of the fire barrier penetration seal ensures that fires will be confined or adequately retarded from spreading to adjacent portions of the facility. This design feature minimizes the possibility of a single fire rapidly involving several areas of the facility prior to detection and extinguishment. The fire barrier penetration seals are a passive element in the facility fire protection program and are subject to periodic inspections.

During periods of time when the seals are not functional, a continuous fire watch is required to be maintained in the vicinity of the affected seal until the seal is restored to functional status.

3.14 BASES

A. Water Fire Protection System

The monthly test of the fire pumps is conducted to check for equipment failures and deterioration. The fire pump minimum capacity is based on a design load of 2400 gpm for the largest sprinkler plus 300 gpm for manual hose lines.

When it is determined that a fire pump is inoperable, the increased surveillance required by 4.14.A.2 provides adequate assurance that the remaining pump will be operable when required.

B. CO2 Fire Protection Systems

Weekly checking of the storage tank level and pressure is deemed adequate to provide assurance that sufficient CO2 will be available in the event of a fire occurrence.

Semi-annual testing of the heat detectors in the automatic discharge systems is in accordance with NFPA-72B-1974.

Testing of the discharge initiation logic, injection valve, damper closings, and fan trippings without actual discharge of CO2 into a room demonstrates operability of the active components of the systems. System operability is demonstrated by both manual and automatic initiation for automatic discharge systems (HPCI and diesel generators). Testing of the headers and nozzles by an air flow test will detect buildups of material which may affect continued availability.

C. Fire Detection

Semi-annual testing of fire detectors is in accordance with NPFA-72B-1974.

D. Fire Barrier Penetrations

Penetration fire barrier seals are visually inspected to verify that they are functional.

6.0 ADMINISTRATIVE CONTROLS

6.1 Responsibility

6.1.1 The Station Superintendent shall be responsible for overall facility operation. In the absence of the Station Superintendent, the Assistant Superintendent or the Engineer-Technical (or any other person that the Station Superintendent may designate in writing) shall, in that order, assume the Superintendent's responsibility for overall facility operation.

6.2 Organization

Offsite

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

Facility Staff

6.2.2 The facility organization shall be as shown on Figure 6.2-2 and:

a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Figure 6.2-2.

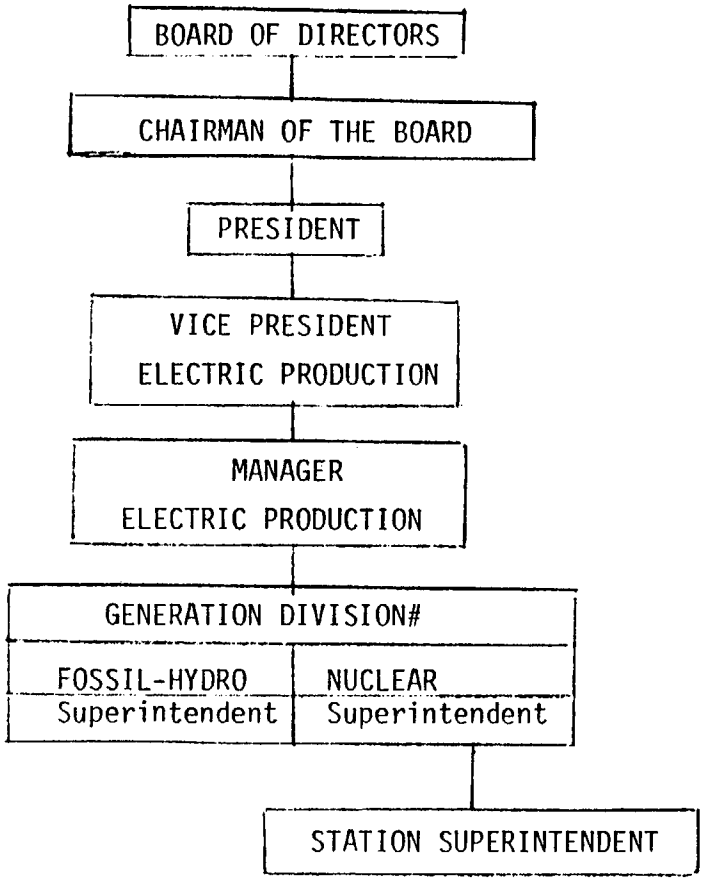
b. At least one licensed operator shall be in the control room and assigned to each reactor that contains fuel.

c. At least two licensed operators, excluding the operator on the second unit, shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.

d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.

e. ALL CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibility during this operation.

f. A Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of the unit(s) (3 members) or any personnel required for other essential functions during a fire emergency.



- Responsible for overall Fire Protection Program

PHILADELPHIA ELECTRIC COMPANY
PEACH BOTTOM ATOMIC POWER STATION
UNITS 2 and 3

MANAGEMENT ORGANIZATION
CHART

FIGURE 6.2-1

Superintendent SLO*#

Asst. Superintendent SLO*

- 1/S Shift Supt (SLO)**
  - Clerical
  - Guards
- Engineering Technical (SLO)
  - Engineer Reactor
  - Engineer I & C (or Results)
  - Engineer Health Physics & Chemistry
  - Engineers
  - Technicians
- Engineer Maintenance
  - Maintenance Foreman
    - Mechanics
  - Custodial
  - Stores

BWRs

1/S Shift Supr. (SLO)**

1/S Control Op. Nuclear (LO)

2/S Asst. Cont. Op. Nuclear (LO)

1/S Plant Op.

1/S Asst. Plant Op.

1/S Aux. Op.

LEGEND:

* - At least one of these shall be licensed.

** - Only one of these need to be licensed during a training and qualification period, provided a second SLO is available on site.

x/S - Number of men per shift. All shift positions are marked.

LO - NRC Licensed Operator

SLO - NRC Licensed Senior Operator

\# - Responsible for on site fire protection activities

ORGANIZATION FOR CONDUCT OF PLANT OPERATIONS

FIGURE 6.2-2

6.3 Facility Staff Qualifications

6.3.1 Each member of the facility staff shall meet the minimum qualifications of ANSI N18.1-1971 for comparable positions.

6.4 Training

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Station Superintendent and shall meet the requirements of Section 5.5 of ANSI N18.1-1971 and 10 CFR 55, Appendix A.

6.4.2 A training program for the Fire Brigade shall be conducted such that Fire Brigade members complete an instruction program within a two year period. Regularly planned meetings will be held every 3 months.

6.5 Review and Audit

6.5.1 Plant Operation Review Committee (PORC)

Function

6.5.1.1 The Plant Operation Review Committee shall function to advise the Station Superintendent on all matters related to nuclear safety.

Composition

6.5.1.2 The Plant Operation Reivew Committee shall be composed of the:

Station Superintendent-Chairman
Station Assistant Superintendent
Engineer - Technical
Engineer - Maintenance
Engineer - Operations
Engineer - Results
Engineer - Reactor
Engineer - Instrument & Control
Engineer - Health Physics & Chemistry
Shift Superintendent

Alternates

6.5.1.3 Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in PORC activities at any one time.

6.10.2 Continued

d. Records of radiation exposure for all individuals entering radiation control areas.

e. Records of gaseous and liquid radioactive material released to the environs.

f. Records of transient or operational cycles for those facility components designed for a limited number of transients or cycles.

g. Records of training and qualification for current members of the plant staff.

h. Records of in-service inspections performed pursuant to these Technical Specifications.

i. Records of Quality Assurance activities required by the QA Manual, except as described in 6.10.1 above.

j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.

k. Records of meetings of the PORC and the OSR Committee.

6.11 Radiation Protection Program

Procedures for personnel radiation protection shall be prepared consistent with requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 Fire Protection Inspections

a. An independent fire protection and loss prevention program inspection shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.

b. An inspection of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.

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6.13 High Radiation Area

6.13.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c) (2) of 10 CFR 20:

a. Each High Radiation Area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-277 AND 50-278

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 39 and 39 to Facility Operating Licenses Nos. DPR-44 and DPR-56, respectively, issued to Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company, which revised Technical Specifications for operation of the Peach Bottom Atomic Power Station, Units Nos. 2 and 3, located in Peach Bottom, York County, Pennsylvania. The amendments become effective 30 days after the date of issuance.

The amendments revise the Technical Specifications and consist of: (1) addition of interim specifications on the existing fire protection equipment and adds administrative controls related to fire protection of the facility pending completion of the Commission's overall fire protection review of the facility and (2) deletion of the respiratory protection program in accordance with the revocation provisions of the current Technical Specifications.

The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendment dated February 2, 1977 as revised by letter dated July 18, 1977, (2) the Commission's letters dated November 23, 1977 and July 29, 1977, (3) the licensee's letter dated December 12, 1977, (4) Amendments Nos. 39 and 39 to Licenses Nos. DPR-44 and DPR-56, and (5) the Commission's letter dated February 28, 1978. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania 17126. A copy of items (2), (4) and (5) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland this 28th day of February 1978.

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors