

Docket Nos. 50-277
and 50-278

SEPTEMBER 19 1979

REGULATORY DOCKET FILE COPY

Mr. Edward G. Bauer, Jr.,
Vice President and General Counsel
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

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- OPA (CMiles)
- RDiggs
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- JRBuchanan

Dear Mr. Bauer:

The Commission has issued Amendment Nos. 60 and 60 to Facility Operating License Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station Units 2 and 3. The amendments revise the table of Primary Containment Isolation Valves to reflect the re-routing of the Control Rod Drive Return Line (CRD RL) to the Reactor Water Cleanup System. The amendments are in response to your application dated April 14, 1978 as supplemented by information contained in your letters dated November 18, 1977, June 22 and August 27, 1979. This action is administrative in nature since it involves implementation of a previously reviewed action, i.e., the re-routing of the CRD RL as discussed in NUREG-0312, "Interim Technical Report on BWR Feedwater and Control Rod Drive Return Line Cracking," July 1977.

In your submittal dated August 27, 1979 you indicated your intent to cut and cap the CRD RL on Unit No. 3 during the Fall 1979 refueling outage and to expedite the re-routing as soon as practicable but not later than the end of the next Unit 3 refueling outage. The entire modification for Unit 2 is planned for the Spring 1980 refueling outage. Although the Control Rod Drive Hydraulic System is not an Engineered Safety Feature it does provide a high pressure capability for injecting water into the reactor vessel. With the return line isolated (or removed) General Electric Company estimates that the capability of a CRD pump to supply high-pressure water to an isolated reactor vessel (of the Peach Bottom design) is reduced from approximately 180 to 165 gpm. This reduced capability can be increased to approximately 205 gpm by optimizing the flow from two pump operation. Although two pump operation is not a design requirement, we understand from discussions with members of your staff that it is physically possible at Peach Bottom. Therefore, we believe that in the interim period before the return line is

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re-routed on Unit No. 3, special procedures should be developed and the operating staff appropriately instructed on the method for maximizing water flow to an isolated vessel. We have discussed this with members of your staff and they agree.

OFFICE
SURNAME
DATE

SEPTEMBER 19 1979

We have also completed our review of the information on your proposed feed-water nozzle and CRD RL nozzle inspection program for the 1979 refueling outage for Peach Bottom 3 as described in your June 22, 1979 letter and conclude that it is acceptable provided that certain of our assumptions are valid:

- (a) During the modifications of the CRD RL, flushing of all carbon steel portions of the system will be accomplished.
- (b) After the CRD RL is cut and the nozzle capped, non-destructive examination of the weld will be performed.

The acceptability of your inspection program is based on our finding that it is consistent with our criteria set forth in NUREG 0312.

We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendments. The amendments apply to administrative details. Therefore, we have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level, and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(d)(4) that an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Since the amendments apply only to administrative details, they do not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. They do not involve a significant increase in the probability or consequences of an accident, do not involve a significant decrease in a safety margin, and therefore do not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

A copy of a related Notice of Issuance is also enclosed.

Sincerely,

Original Signed by
T. A. Ippolito

Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures and ccs: See page 3

OFFICE >						
SURNAME >						
DATE >						

R. Schneider
SEP 8
9/12/79
R. SCHNAIDER

EB
9/5/79
V Noonan

RSB
9/6/79
P. CNECK.

PSB
9/11/79

OFFICE	ORB #3	ORB #3	AD: ORB	OELD	ORB #3
SURNAME	<i>PK</i> PKreutzer	<i>D</i> DVerrelli:mjf	<i>WG</i> WGamm 11	<i>CUTWIN</i> CUTWIN	<i>TIPOLITE</i> TIPOLITE
DATE	9/5/79	9/5/79	9/12/79	9/12/79	9/12/79

Enclosures:

1. Amendment No. 60 to DPR-44
2. Amendment No. 60 to DPR-56
3. Notice

cc w/enclosures:

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Governor's Office of State Planning
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Board of Supervisors
Peach Bottom Township
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Delta, Pennsylvania 17314

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Director, Technical Assessment
Division
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(AW-459)

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Arlington, Virginia 20460

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Philadelphia, Pennsylvania 19106

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Philadelphia Electric Company
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Philadelphia, Pennsylvania 19101

Government Publications Section
State Library of Pennsylvania
Education Building
Commonwealth and Walnut Streets
Harrisburg, Pennsylvania 17126



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 60
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al., (the licensee) dated April 14, 1978 with supplemental information dated November 18, 1977, June 22 and August 27, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:
 - (2) The Technical Specifications contained in Appendices A and B, as revised through Amendment No.60, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 19, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 60

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

Remove

179*
180

Insert

179*
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*Overleaf

TABLE 3.7.1

PRIMARY CONTAINMENT ISOLATION VALVES

Group	Valve Identification	Number of Power Operated Valves		Maximum Operating Time (sec.)	Normal Position	Action on Initiating Signal
		Inboard	Outboard			
1	Main steam line isolation valves	4	4	3 < T < 5	0	GC
1	Main steam line drain isolation valves	1	1	15	C	SC
1	Reactor water sample line isolation valves	1	1	5	C	SC
1	Main steam sample line	1	1	5	C	SC
3	Drywell purge inlet isolation valves		2	5	C	SC
3	Suppression chamber purge inlet isolation valves		2	5	C	SC
3	Nitrogen purge isolation valve		1	5	C	SC
3	Nitrogen makeup isolation valve		1	10	C	SC
3	Drywell main exhaust isolation valves		2	5	C	SC
3	Drywell exhaust valve bypass isolation valves		2	5	C	SC
3	Suppression chamber main exhaust isolation valves		2	5	C	SC
3	Suppression chamber exhaust valve bypass isolation valves		2	5	C	SC

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TABLE 3.7.1 (Cont'd.)

PRIMARY CONTAINMENT ISOLATION VALVES

Group	Valve Identification	Number of Power Operated Valves		Maximum Operating Time (sec.)	Normal Position	Action on Initiating Signal
		Inboard	Outboard			
11A	Feedwater check valves	2*	2*	NA	0	Process
NA	Control rod hydraulic return check valves	[1*]#	[1*]#	NA	C	Process
NA	Control rod hydraulic return valve		[1*]##	NA	C	Locked Closed
2D	Oxygen Analyzer System		14	NA	0	GC
NA	Standby liquid control system check valves	1*	1*	NA	C	Process
2B	RHRS shutdown cooling suction isolation valves	1	1	32	C	SC
2B	RHRS shutdown cooling injection isolation valves		1	24	C	SC
2B	RHRS Reactor Vessel Head Spray isolation valves	1	1	30	C	SC
2C	Feedwater Flush Valves		2	50	C	SC
4	HPCIS steam line isolation valves	1	1	20	0	GC
5	RCICS steam line isolation valves	1	1	15	0	GC
2A	Reactor water cleanup system isolation valves	1	1	30	0	GC
2A	Reactor water cleanup system return isolation valve		1	20	0	GC

*Valves not power operated.
 #Deleted after CRD RL is cut and capped
 ##Effective after CRD RL is rerouted to RWCU System

-180- Amendment No. 60



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 60
License No. DPR-56

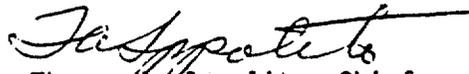
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al., (the licensee) dated April 14, 1978, with supplemental information dated November 18, 1977, June 22 and August 27, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 60, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 19, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 60

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

Remove

179*
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Insert

179*
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TABLE 3.7.1

PRIMARY CONTAINMENT ISOLATION VALVES

Group	Valve Identification	Number of Power-Operated Valves		Maximum Operating Time (sec.)	Normal Position	Action on Initiating Signal
		Inboard	Outboard			
1	Main steam line isolation valves	4	4	3 < T < 5	0	GC
1	Main steam line drain isolation valves	1	1	15	C	SC
1	Reactor water sample line isolation valves	1	1	5	C	SC
1	Main steam sample line	1	1	5	C	SC
3	Drywell purge inlet isolation valves		2	5	C	SC
3	Suppression chamber purge inlet isolation valves		2	5	C	SC
3	Nitrogen purge isolation valve		1	5	C	SC
3	Nitrogen makeup isolation valve		1	10	C	SC
3	Drywell main exhaust isolation valves		2	5	C	SC
3	Drywell exhaust valve bypass isolation valves		2	5	C	SC
3	Suppression chamber main exhaust isolation valves		2	5	C	SC
3	Suppression chamber exhaust valve bypass isolation valves		2	5	C	SC

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MAY 1973

TABLE 3.7.1 (Cont'd.)

PRIMARY CONTAINMENT ISOLATION VALVES

Group	Valve Identification	Number of Power Operated Valves		Maximum Operating Time (sec.)	Normal Position	Action on Initiating Signal
		Inboard	Outboard			
NA	Feedwater check valves	2*	2*	NA	0	Process
NA	Control rod hydraulic return check valves	[1*]#	[1*]#	NA	C	Process
NA	Control rod hydraulic return valve		[1*]##	NA	C	Locked Closed
2D	Oxygen Analyzer System		14	NA	0	GC
NA	Standby liquid control system check valves	1*	1*	NA	C	Process
2B	RHRS shutdown cooling suction isolation valves	1	1	32	C	SC
2B	RHRS shutdown cooling injection isolation valves		1	24	C	SC
2B	RHRS Reactor Vessel Head Spray isolation valves	1	1	30	C	SC
2C	Feedwater Flush Valves		2	50	C	SC
4	HPCIS steam line isolation valves	1	1	20	0	GC
5	RCICS steam line isolation valves	1	1	15	0	GC
2A	Reactor water cleanup system isolation valves	1	1	30	0	GC
2A	Reactor water cleanup system return isolation valve		1	20	0	GC
	*Valves not power operated.					
	#Deleted after CRD RL is cut and capped					
	##Effective after CRD RL is rerouted to RWCU System					

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-277 AND 50-278

PHILADELPHIA ELECTRIC COMPANY, ET AL.

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 60 and 60 to Facility Operating License Nos. DPR-44 and DPR-56, issued to Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company, which revised Technical Specifications for operation of the Peach Bottom Atomic Power Station, Units Nos. 2 and 3 (the facility) located in York County, Pennsylvania. The amendments are effective as of the date of issuance.

The amendments revise the table of Primary Containment Isolation Valves to reflect the re-routing of the Control Rod Drive Return Line to the Reactor Water Cleanup System.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

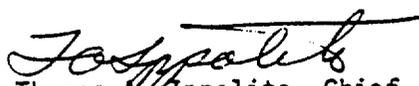
The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

- 2 -

For further details with respect to this action, see (1) the application for amendments dated April 14, 1978 with supplemental information dated November 18, 1977, June 22 and August 27, 1979, (2) Amendment Nos. 60 and 60 to License Nos. DPR-44 and DPR-56, and (3) the Commission's letter dated September 19, 1979. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 19th day of September 1979.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors