

RETURN TO REACTOR DOCKET FILES

JUNE 15 1979

Docket Nos. 50-277 ✓
and 50-278

Mr. Edward G. Bauer, Jr., Esquire
Vice President and General Counsel
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

The Commission has issued the enclosed Amendment Nos. 55 and 56 to Facility Operating Licenses No. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station Units 2 and 3. The amendment consists of changes to the Technical Specifications in response to your application dated January 12, 1979.

This amendment revises the Technical Specifications to (1) permit a decrease in the discharge pressure of the High Pressure Service Water pumps from 280 psig to 233 psig; (2) add certain snubbers to the table of Safety Related Shock Suppressors, and (3) revise the wording of the Administrative Controls specifications to clarify staff requirements for audits of facility activities.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Richard J. Clark for.

Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

*Crush
cup*

Enclosures:

1. Amendment Nos. 55 and 56
2. Safety Evaluation
3. Notice

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*Noted objection to piece of
notice and amendment
(SER & TS not reviewed)*

cc w/enclosures:
See next page

OFFICE →	ORB #3	AD: E&P	PSB	RSE	OELD	ORB #3
SURNAME →	Verrelli/Kreutzer	BGrimes	Blainas	PCheck	CUTCHIN	Ippolito
DATE →	5/2/79	5/11/79	1/22/79	5/12/79	6/13/79	5/13/79

Mr. Edward G. Bauer, Jr.

- 2 -

June 15, 1979

cc:

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Commonwealth and Walnut Streets
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 55
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company et al., (the licensee) dated January 12, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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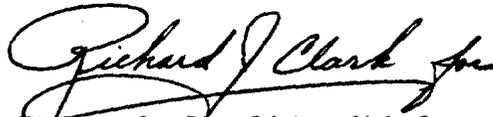
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 55, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 15, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 55

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

127
234k
234l
234m
234n
251

Insert

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234k
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251

LIMITING CONDITIONS FOR OPERATION3.5.A Core Spray and LPCI Subsystem (cont'd)

6. All recirculation pump discharge valves shall be operable prior to reactor startup (or closed if permitted elsewhere in these specifications).
7. If the requirements of 3.5.A cannot be met, an orderly shutdown of the reactor shall be initiated and the reactor shall be in the Cold Shutdown Condition within 48 hours.

B. Containment Cooling Subsystem (HPSW)

1. Except as specified in 3.5.B.2, 3.5.B.3, 3.5.B.4, and 3.5.F.3 below, all containment cooling subsystem loops shall be operable whenever irradiated fuel is in the reactor vessel and reactor coolant temperature is greater than 212°F, and prior to reactor startup from a Cold Shutdown Condition.
2. From and after the date that any two HPSW pumps are made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding thirty days, unless such pump is sooner made operable, provided that during such thirty days all active components of the containment cooling subsystem are operable.

SURVEILLANCE REQUIREMENTS4.5.A Core Spray and LPCI Subsystem (cont'd)

6. All recirculation pump discharge valves shall be tested for operability during any period of reactor cold shutdown exceeding 48 hours, if operability tests have not been performed during the preceding 31 days.

B. Containment Cooling Subsystem (HPSW)

1. Containment Cooling Subsystem Testing shall be as follows:

<u>Item</u>	<u>Frequency</u>
(a) Pump Operability	Once/month
(b) Motor operated valve operability	Once/month
(c) Pump Capacity Test. Each HPSW pump shall deliver 4500 gpm at 233 psig.	After pump maintenance and every 3 months.
(d) Air test on drywell and torus headers and nozzles.	Once/5 years

2. When it is determined that any two HPSW pumps are inoperable, the remaining components of the containment cooling subsystems shall be demonstrated to be operable immediately and weekly thereafter.

PBAPS
TABLE 3.11.D.1 (Cont'd)

Unit 2

Safety Related Shock Suppressors (Snubbers)

SNUBBER NUMBER	LOCATION	ELEVATION	SNUBBER IN HIGH(1) RADIATION AREA DURING SHUTDOWN	SNUBBERS ESPECIALLY DIFFICULT TO REMOVE	SNUBBERS INACCESSIBLE DURING NORMAL OPERATION	SNUBBERS ACCESSIBLE DURING NORMAL OPERATION
10-GB-S-12	RHR	98	See 4.11.D.4.b			'C'RHR RM.
10-GB-S-43-1	RHR	130	"	X		TORUS RM.
10-GB-S-43-2	RHR	130	"	X		TORUS RM.
10-GB-S-44	RHR	128	"	X		TORUS RM.
10-GB-S-48	RHR	124	"			'B'RHR RM.
10-GB-S-49	RHR	124	"			"
10-GB-S-50	RHR	98	"			"
10-GB-S-51	RHR	98	"			'C'RHR RM.
10-GB-S-52	RHR	124	"			"
10-GB-S-53	RHR	124	"			"
10-GB-S-54	RHR	130	"	X		TORUS RM.
10-GB-S-55	RHR	130	"	X		"
10-GB-S-58	RHR	98	"			'B'RHR RM.
10-DCN-S-73	RHR	180	"	X	Drywell	
10-DCN-S-74	RHR	180	"	X	Drywell	

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PBAPS
TABLE 3.11.D.1 (Cont'd)

Unit 2

Safety Related Shock Suppressors (Snubbers)

SNUBBER NUMBER	LOCATION	ELEVATION	SNUBBER IN HIGH(1) RADIATION AREA DURING SHUTDOWN	SNUBBERS ESPECIALLY DIFFICULT TO REMOVE	SNUBBERS INACCESSIBLE DURING NORMAL OPERATION	SNUBBERS ACCESSIBLE DURING NORMAL OPERATION
10-GB-S-77	RHR	102	See 4.11.D,4.b			'A'RHR RM.
10-GB-S-76	RHR	102	"			"
10-GB-S-75	RHR	93	"			"
10-GB-S-80	RHR	102	"			'D'RHR RM.
10-GB-S-79	RHR	102	"			"
10-GB-S-78	RHR	93	"			"
12-DCN-S-2	RWCU	173.5	"	X		RWCU ISOLATION VALVE RM. 165
12-DCN-S-5	RWCU	165	"	X	Drywell	
12-DCN-S-7	RWCU	165	"	X	Drywell	
12-DCN-S-8A	RWCU	162	"		Drywell	
14-DCN-S-23	CORE SPRAY	168	"	X	Drywell	
14-DCN-S-24	CORE SPRAY	168	"	X	Drywell	
14-DCN-S-26	CORE SPRAY	168	"	X	Drywell	
14-DCN-S-27	CORE SPRAY	168	"	X	Drywell	

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PBAPS
TABLE 3.11.D.1 (Cont'd)

Unit 2

Safety Related Shock Suppressors (Snubbers)

SNUBBER NUMBER	LOCATION	ELEVATION	SNUBBER IN HIGH(1) RADIATION AREA DURING SHUTDOWN	SNUBBERS ESPECIALLY DIFFICULT TO REMOVE	SNUBBERS INACCESSIBLE DURING NORMAL OPERATION	SNUBBERS ACCESSIBLE DURING NORMAL OPERATION
13-HC-S-1	RCIC	107	See 4.11.D.4.b	X		RCIC ROOM
13-DDN-S-13	RCIC	96	"			"
13-HB-S-14	RCIC	102	"	X		"
13-DBN-S-15	RCIC	107	"	X		"
13-DBN-S-16	RCIC	140	"	X		TORUS ROOM
23-DBN-S-1	HPCI	112	"	X		HPCI ROOM
23-DBN-S-2	HPCI	112	"	X		"
23-DBN-S-3	HPCI	97	"			"
23-DBN-S-4	HPCI	97	"			"
23-DDN-S-9	HPCI	105	"	X		"
23-HB-S-16	HPCI	103	"			"
23-HB-S-19	HPCI	103	"	X		"
23-DBN-S-22	HPCI	155	"	X	Drywell	
23-DBN-S-23	HPCI	155	"	X	Drywell	
23-DDN-S-25	HPCI	105	"	X		HPCI ROOM
23-DBN-S-27	HPCI	112	"	X		"

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PBAPS
TABLE 3 11.D.1 (Cont'd)

Unit 2

Safety Related Shock Suppressors (Snubbers)

SNUBBER NUMBER	LOCATION	ELEVATION	SNUBBER IN HIGH(1) RADIATION AREA DURING SHUTDOWN	SNUBBERS ESPECIALLY DIFFICULT TO REMOVE	SNUBBERS INACCESSIBLE DURING NORMAL OPERATION	SNUBBERS ACCESSIBLE DURING NORMAL OPERATION
23-DBN-S-28	HPCI	117	"	X		TORUS ROOM
23-DBN-S-29	HPCI	117	"	X		TORUS ROOM
23-HB-S-30	HPCI	93	"			HPCI ROOM
23-HB-S-36	HPCI	103	"			"
23-HB-S-37	HPCI	103	"			"
23-HB-S-38	HPCI	126	"	X		TORUS ROOM

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Notes for Table 3.11.D.1

- (1) Modifications to this Table due to changes in high radiation areas should be submitted to the NRC as part of the next license amendment.

PBAPS

6.5.2.7 Continued

- d. Proposed changes in Technical Specifications or Licenses.
- e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. All events which are required by regulations or Technical Specifications to be reported to the NRC in writing within 24 hours.
- h. Any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meeting minutes of the Plant Operation Review Committee.

Audits

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the OSR Committee. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- b. The performance, training and qualifications of the entire facility staff at least once per year.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.
- d. The performance of activities required by the Quality Assurance Program to meet the criteria of 10 CFR 50, Appendix B, at least once per two years.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 55
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al., (the licensee) dated January 12, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 55, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 15, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 55

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

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Insert

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LIMITING CONDITIONS FOR OPERATION3.5.A Core Spray and LPCI Subsystem (cont'd)

6. All recirculation pump discharge valves and bypass valve(s)[*] shall be operable prior to reactor startup (or closed if permitted elsewhere in these specifications).
7. If the requirements of 3.5.A cannot be met, an orderly shutdown of the reactor shall be initiated and the reactor shall be in the Cold Shutdown Condition within 48 hours.

B. Containment Cooling Subsystem (HPSW)

1. Except as specified in 3.5.B.2, 3.5.B.3, 3.5.B.4, and 3.5.F.3 below, all containment cooling subsystem loops shall be operable whenever irradiated fuel is in the reactor vessel and reactor coolant temperature is greater than 212°F, and prior to reactor startup from a Cold Shutdown Condition.
2. From and after the date that any two HPSW pumps are made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding thirty days, unless such pump is sooner made operable, provided that during such thirty days all active components of the containment cooling subsystem are operable.

SURVEILLANCE REQUIREMENTS4.5.A Core Spray and LPCI Subsystem (cont'd)

6. All recirculation pump discharge valves and bypass valve(s)[*] shall be tested for operability during any period of reactor cold shutdown exceeding 48 hours, if operability tests have not been performed during the preceding 31 days.

B. Containment Cooling Subsystem (HPSW)

1. Containment Cooling Subsystem Testing shall be as follows:

	<u>Item</u>	<u>Frequency</u>
(a)	Pump Operability	Once/month
(b)	Motor operated valve operability	Once/month
(c)	Pump Capacity Test. Each HPSW pump shall deliver 4500 gpm at 233 psig.	After pump maintenance and every 3 months.
(d)	Air test on drywell and torus headers and nozzles.	Once/5 years

2. When it is determined that any two HPSW pumps are inoperable, the remaining components of the containment cooling subsystems shall be demonstrated to be operable immediately and weekly thereafter.

* Upon the removal of both recirculation pump discharge valve bypass valves, operability and surveillance of only the recirculation pump discharge valves is required.

6.5.2.7 Continued

- d. Proposed changes in Technical Specifications or Licenses.
- e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. All events which are required by regulations or Technical Specifications to be reported to the NRC in writing within 24 hours.
- h. Any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meeting minutes of the Plant Operation Review Committee.

Audits

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the OSR Committee. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- b. The performance, training and qualifications of the entire facility staff at least once per year.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.
- d. The performance of activities required by the Quality Assurance Program to meet the criteria of 10 CFR 50, Appendix B, at least once per two years.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 55 AND 56 TO FACILITY LICENSE NOS. DPR-44 AND DPR-56

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION UNITS NOS. 2 AND 3

DOCKETS NOS. 50-277 AND 50-278

I. INTRODUCTION

By letter dated January 12, 1979 Philadelphia Electric Company (licensee) requested an amendment to Operating License Nos. DPR-44 and 56 for the Peach Bottom Atomic Power Station Unit Nos. 2 and 3. The proposed amendment would revise the Technical Specifications to: (1) permit a decrease in the discharge pressure of the High Pressure Service Water (HPSW) pumps from 280 psig to 233 psig; (2) add certain snubbers to the Table of Safety Related Shock Suppressors, and (3) revise the wording of the Administrative Controls specifications to clarify staff requirements for audits of facility activities.

II. EVALUATION

A. HPSW Pump Discharge Pressure

Technical Specification 4.5.B.1(b) for Peach Bottom Units 2 and 3 requires that, after pump maintenance and every three months, each HPSW pump be tested to confirm that it delivers 4500 gpm of water at 280 psig. The licensee requested that the HPSW pump pressure be reduced to 233 psig to account for a reduced pressure on the Residual Heat Removal (RHR) side of the RHR heat exchangers by installation of flow restricting orifices and drag valves upstream of the heat exchangers. The modification on the RHR side of the heat exchanger was performed to provide (1) throttling capability in the shutdown cooling mode of operation and (2) additional piping system resistance to prevent pump runout which could occur with one RHR pump injecting into a broken recirculation loop.¹ Prior to the modification additional resistance was accomplished by throttling the manual heat exchanger outlet valves. With the additional resistance, the manual valves can open fully. Because the operating pressure on the shell side of the heat exchanger had been reduced, the licensee proposed to reduce the pressure limit on the tube side in order to maintain the same pressure differential. The reduced pressure on the HPSW system would have the advantage of reducing pump wear from river water silt. This reduction in wear would accordingly decrease pump outage for corrective maintenance and increase the overall availability of this safety related component.

¹NRC Evaluation of this modification was transmitted by letter NRC (Lear) to PECO (Bauer) dated January 4, 1977.

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The 4 HPSW pumps, arranged in two trains of 2 pumps each, supply cooling water to the tube side of the RHR heat exchangers. Since the shell side of the RHR exchangers contains radioactively contaminated water, any flow resulting from a tube leak should be from the tube to the shell side of the heat exchanger. This design feature is a requirement of the RHR heat exchanger to prevent contamination of the HPSW resulting from tube leaks thereby minimizing the potential for significant unmonitored radioactive releases.

The licensee performed an analysis of RHR and HPSW pressures similar to that previously reviewed and approved by the staff.² The maximum pressures at the inlet of the process side of the RHR heat exchanger considered all modes of operation of the RHR/HPSW system except the LPCI mode of operation.

In the LPCI mode of operation, HPSW is not required for the first ten minutes of operation to mitigate the consequences of an accident.³ Thus in this mode, the pressure on the RHR side of the heat exchanger is higher than on the HPSW side. Should a tube leak develop during this period of time, potentially contaminated water could leak from the primary system to the service water side of the heat exchanger. However, water would not be discharged in the river because the discharge valve from the heat exchanger is closed.⁴ Therefore, we conclude that the LPCI mode of operation need not be considered in the analysis of maximum pressures on the RHR side in order to determine the minimum HPSW discharge pressure.

In RHR mode of operation other than the LPCI mode, if the pressure at the tube outlet is greater than the pressure at the shell inlet of an RHR heat exchanger, no leakage from the shell to the tube side will occur through a tube leak. The shell design provides for the inlet near the bottom of the heat exchanger and its outlet near the top; the tubes both enter and exit near the top of the RHR heat exchanger. The maximum pressure on the shell (contaminated water) side inlet was calculated to be 198 psig. To assure that the HPSW pressure at the heat exchanger outlet is greater than 198 psig, the HPSW pump discharge pressure must be equal to 198 psig plus a pressure differential to account for heat exchanger pressure drop, pipe losses, and elevation differences. The licensee's analysis accounted for accuracy of the instrument measuring the pump discharge pressure and the variation of river water level. We have determined that the calculations support a HPSW pump discharge pressure of 233 psig.

The licensee has previously stated that his procedures establish that the HPSW pumps are started before the respective RHR loop is pressurized by the RHR pump, and the HPSW pump is stopped after the RHR pump is shutdown. Further, the differential pressure is monitored and a control room alarm is annunciated whenever the differential pressure is less than 15 psi.⁴

²Safety Evaluation supporting Amendments Nos. 5 and 3 to DPR-44 and 56, February 12, 1975.

³PBAPS FSAR, Section 4.8.

⁴Letter, PECO (Hankins) to NRC (Lear), October 8, 1976.

We conclude from the above that the modifications which have been completed on the RHR system, the analysis of RHR/HPSW discharge pressures, installed instrumentation, and implemented procedures are adequate to preclude significant unmonitored radioactive releases and that a HPSW system discharge pressure of 233 psig is acceptable.

B. Addition of Safety Related Shock Suppressors

The licensee proposed to add certain snubbers to the Table of Safety Related Shock Suppressors to reflect the recent addition of snubbers to the Residual Heat Removal System and to the Reactor Cleanup System of Unit No. 2. This request is consistent with the current specifications as implemented by Amendment No. 33 to DPR-44, dated April 28, 1977, which states that snubbers may be added to safety related systems without prior approval provided a proposed revision to the Table is included in the next amendment request. Therefore, we find the change to be an administrative action which implements a requirement of the existing Technical Specifications.

C. Administrative Controls

The licensee proposed to revise the wording of certain administrative controls specifications by removing the word "all" in sections pertaining to audits. We have reviewed the proposed change and determined that it is more descriptive of an audit function rather than a complete review, is consistent with the language used in the staff's Standard Technical Specifications (NUREG 0123 Rev. 1, April 1, 1978) and is therefore acceptable.

Environmental Considerations

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR Section 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusions

We have concluded, based on the considerations discussed above, that:
(1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not

involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 15, 1979

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-277 AND 50-278PHILADELPHIA ELECTRIC COMPANY, ET AL.NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 55 and 55 to Facility Operating License Nos. DPR-44 and DPR-56, issued to Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company and Atlantic City Electric Company, which revised Technical Specifications for operation of the Peach Bottom Atomic Power Station Units Nos. 2 and 3 (the facility) located in York County, Pennsylvania. The amendments are effective as of the date of issuance.

These amendments revise the Technical Specifications to (1) permit a decrease in the discharge pressure of the High Pressure Service Water pumps from 280 psig to 233 psig; (2) add certain snubbers to the table of Safety Related Shock Suppressors, and (3) revise the wording of the Administrative Controls specifications to clarify staff requirements for audits of facility activities.

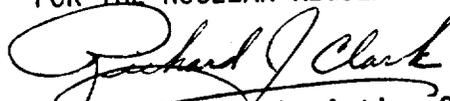
The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated January 12, 1979, (2) Amendment Nos. and to License Nos. DPR-44 and DPR-56, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D.C. and at the Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this

FOR THE NUCLEAR REGULATORY COMMISSION


Richard J. Clark, Acting Chief
Operating Reactors Branch #3
Division of Operating Reactors