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April 30, 2001
BYR 2001-040

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

References: (a) License No. DPR-3 (Docket No. 50-29)

Subject: 2000 ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

Enclosed is the 2000 Annual Radioactive Effluent Release Report. This report summarizes the quantities of radioactive liquid and gaseous effluent and solid waste released from the Yankee Nuclear Power Station (YNPS) in Rowe, Massachusetts. This report also summarizes the estimated dose commitments from all radioactive liquids and gaseous effluents released during 2000. This information is submitted in accordance with YNPS Defueled Technical Specification 6.8.2.b and the YNPS Off-Site Dose Calculation Manual. In addition, enclosed are complete copies of the YNPS Off-Site Dose Calculation Manual.

We trust this information is satisfactory; however, if you have any questions, please contact Mr. James A. Kay (978) 568-2302.

Very truly yours,

YANKEE ATOMIC ELECTRIC COMPANY

J. A. Kay

James A. Kay
Manager of Regulatory affairs

Enclosure

c: J. Hickman, USNRC, Project Manager
R. B. Bellamy, USNRC, Region I

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2000 ANNUAL

RADIOACTIVE EFFLUENT RELEASE REPORT

Yankee Atomic Electric Company
Rowe, Massachusetts

NOTES:

1. Yankee Nuclear Power Station's last day at any power level was October 1, 1991. The facility is permanently shut down and in the process of decommissioning. Due to ceased operations, short-lived nuclides have been deleted from the gaseous and liquid effluent tables. Their activity concentrations in the fuel inventory have decayed to zero values.

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2000 ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

1.0 INTRODUCTION

Tables 1 through 3 list the recorded radioactive gaseous and liquid effluents and solid waste, respectively, with data summarized on a quarterly basis for the year. Table 4 summarizes the estimated radiological dose commitments from all radioactive liquid and gaseous effluents released during the year 2000

As required by Control 7.2.b, dose commitments resulting from the release of radioactive materials in liquids and gases were estimated in accordance with the Yankee Nuclear Power Station Off-Site Dose Calculation Manual (ODCM). These dose estimates were made using a Method II analysis as described in the ODCM. A Method II analysis incorporates the methodology of Regulatory Guide 1.109 (Reference 1) using historic meteorological data. For gaseous releases, five years of historic (1992-1996) quarterly meteorological data were used for determining the gaseous pathway doses. As required by Control 7.2.b, this report also shall include an assessment of the radiation doses from radioactive effluents to member(s) of the public due to allowed recreational activities inside the site boundary during the year. However, for this reporting period, no recreational activities inside the site boundary were permitted. As a result, recreational activities are not addressed. The limited use of the Information Center on-site is associated with educational activities as they pertain to the operation/decommissioning of the plant and as such, is not included under Control 7.2.b. Assessment of radiation doses (including direct radiation) to the likely most exposed real member(s) of the public for the calendar year for the purposes of demonstrating conformance with 40CFR Part 190, "Environmental Radiation Protection Standards for Nuclear Power Operations," also are required to be included in this report, if conditions indicated in Control 3.2 have been exceeded during the year. Since the conditions indicated in the action statement under Control 3.2.a were not entered into during the year, no additional radiation dose assessments are required.

All calculated dose estimates for this reporting period are well below the dose criteria of 10CFR Part 50, Appendix I.

Appendices A through H indicate the status of reportable items per the requirements of Controls 5.1, 5.2, Technical Specification 3.4, Controls 4.1, 4.2, 2.0 (PCP), 7.2, and 7.3, respectively. Appendix I of this report provides supplemental information on effluent releases

for this reporting period and Appendix J provides information concerning the disposal of any sewage sludge which may have occurred during this reporting period.

2.0 METEOROLOGICAL DATA

Five years of historic meteorological data (1992-1996) collected from the site's 200-foot meteorological tower, located approximately 180 meters north of the vapor container, were used to model the atmospheric dispersion of airborne effluents. The tower instrumentation was designed to meet the requirements of Regulatory Guide 1.23 (Reference 2) for meteorological monitoring. A summary of the 1992-1996 meteorological data is provided in Table 5 of this report.

The main release point for gases discharged from the plant is via the 150-foot primary vent stack, located between the vapor container and the primary auxiliary building. The primary vent stack is treated as a mixed mode elevated release point dependent upon wind speed, as described in Regulatory Guide 1.111 (Reference 3).

Atmospheric diffusion was calculated using quarterly historical data along with the recorded quarterly effluent information. CHI/Q and D/Q values were derived for all receptor points using a straight-line airflow model. All dispersion and deposition factors have been calculated employing appropriate source configuration considerations and removal mechanism (e.g., dry deposition) described in Regulatory Guide 1.111 (Reference 3). Terrain elevations, including downwind valley flow corrections for the surrounding area, were factored into the calculation of CHI/Q and D/Q values at each receptor location.

3.0 DOSE ASSESSMENT

3.1 Doses From Liquid Effluents

Control 3.1 limits total body (1.5 mrem per quarter and 3 mrem per year) and organ (5 mrem per quarter and 10 mrem per year) doses from liquid effluents to a member of the public to those specified in 10CFR Part 50, Appendix I. By implementing the requirements of 10CFR Part 50, Appendix I, Control 3.1 assures that the release of radioactive material in liquid effluents will be kept "as low as is reasonably achievable."

Exposure pathways that could exist as a result of liquid effluents are fish, direct exposure from river shoreline sedimentation, milk and meat via animal ingestion of the Deerfield River water, and meat, milk, and vegetable pathways via crop irrigation with water withdrawn from the Deerfield River. Drinking water and aquatic invertebrate pathways do not exist downriver of the Yankee plant at Rowe. The dose analysis for the liquid pathways assumes a dilution based on the monthly average flow at the Sherman Dam.

The whole body and organ doses due to liquid effluents were determined by summing the contributions from all pathways. The whole body and organ doses to a member of the public from liquid effluents are given in Table 4. The estimated quarterly and annual doses due to liquid effluents are well below the 10CFR Part 50, Appendix I dose criteria of Control 3.1.

3.2 Doses From Noble Gases

Control 3.4 limits the gamma air (5 mrad per quarter and 10 mrad per year) and beta air (10 mrad per quarter and 20 mrad per year) doses from noble gases released in gaseous effluents from the site to areas at and beyond the site boundary to those specified in 10CFR Part 50, Appendix I. By implementing the requirements of 10CFR Part 50, Appendix I, Control 3.4 assures that the release of radioactive noble gases in gaseous effluents will be kept "as low as is reasonably achievable."

If noble gases are determined to be present in effluent discharge, the dose estimates are calculated at the site boundary, nearest resident, nearest vegetable garden, and nearest milk animal in each of the sixteen principle compass directions, as well as the point of highest off-site ground level air concentrations of radioactive materials. Gamma and beta air doses, as well as whole body and skin doses, are calculated at each of the above locations.

To determine the beta contribution to the skin dose, a semi-infinite cloud model is utilized. The whole body gamma dose is calculated using a finite cloud sector average model with a Gaussian distribution of activity in the vertical plane. The gamma radiation received from the cloud at a point of interest is determined by integrating the contribution from a differential volume over the entire cloud, taking into account the geometry of the cloud, variation in concentration, attenuation by the interaction of photons with matter in the path between the source and receptor point, and scattering of radiation from material outside the direct path to the point of interest. No additional credit is taken for decay of radionuclides in transit to the receptor point.

3.3 Doses From Tritium and Radionuclides in Particulate Form With Half-Lives Greater Than 8 Days

Control 3.5 limits the organ doses to a member of the public from tritium and radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents released from the site to areas at and beyond the site boundary to those specified in 10CFR Part 50, Appendix I (7.5 mrem per quarter and 15 mrem per year). By implementing the requirements of 10CFR Part 50, Appendix I, Control 3.5 assures that the releases of tritium and particulates in gaseous effluents will be kept "as low as is reasonably achievable." It should be noted that due to the permanent shutdown of the plant (last power operation was in October 1991), the Iodine-131 source term has decayed away and no longer has the potential to affect dose assessment.

Exposure pathways that could exist as a result of the release of particulates and tritium to the atmosphere include external irradiation from activity deposited onto the ground surface, inhalation, and ingestion of vegetables, meat, and milk. Dose estimates for 2000 were made at the site boundary, nearest resident, nearest vegetable garden, and nearest meat animal in each of the sixteen principle compass directions. The nearest resident, vegetable garden and milk animal in each sector were searched for by the most recent Land Use Census, as required by Control 4.2. (Note that no milk animals were identified within 5 miles in this year's land use census and, therefore, this pathway is not included for this report period). Doses were calculated for pathways that were determined by the field survey to actually exist. Conservatively, a vegetable garden is assumed to exist at each milk animal location when the milk pathway is included. Furthermore, the meat pathway is assumed to exist at locations

where identified in the past (a meat animal inventory is not required by the annual Land Use Census) and at each milk animal location (when milk is identified). Meat and milk animals are assumed to receive their entire intake from pasture during the second and third quarters. This assumption is conservative since most dairy operations utilize supplemental feeding of animals when on pasture or actually restrict animals to full time silage feeding throughout the entire year.

The organ doses were determined after adding the contributions from all pathways at each location. Doses were calculated for the whole body, GI-tract, bones, liver, kidneys, thyroid, lungs, and skin for adults, teenagers, children, and infants. The maximum estimated quarterly and annual organ doses due to tritium and particulates at any of the off-site receptor locations are reported in Table 4. The doses to all other organs at all other locations for all other age groups are less than the doses reported in Table 4. The estimated organ doses from tritium and particulates in gaseous effluents are well below the 10CFR Part 50, Appendix I dose criteria of Control 3.5.

4.0 REFERENCES

1. Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Release of Reactor Effluents for the Purpose of Evaluating Compliance With 10CFR Part 50, Appendix I," U.S. Nuclear Regulatory Commission, Office of Standards Development, Revision 1, October 1977.

2. Regulatory Guide 1.23, "On-Site Meteorological Programs (Safety Guide 23)," U.S. Nuclear Regulatory Commission, Office of Standards Development, February 1972.

3. Regulatory Guide 1.111, "Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light - Water - Cooled Reactors," U.S. Nuclear Regulatory Commission, Office of Standards Development, Revision 1, October 1977.

TABLE 1A
(Sheet 1 of 2)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
Gaseous Effluents – Summation of All Releases

	Unit	Quarter 1	Quarter 2	Est. Total Error, %
A. Fission and Activation Gases				
1. Total Release	Ci	ND	ND	±2.50E+01
2. Average Release Rate for Period	µCi/sec	ND	ND	
3. Percent of Control Limit[a]	%	0.00E+00	0.00E+00	
B. Iodines[b]				
C. Particulates				
1. Particluates with Half-lives > 8 days	Ci	4.76E-07	ND	±3.00E+01
2. Average Release Rate for Period	µCi/sec	6.06E-08	ND	
3. Percent of Control Limit[c]	%	7.59E-04	1.23E-03	
4. Gross Alpha Radioactivity	Ci	ND	ND	
D. Tritium				
1. Total Release	Ci	2.30E-02	3.18E-02	±3.00E+01
2. Average Release Rate for Period	µCi/sec	2.93E-03	4.05E-03	
3. Percent of Control Limit	%	(d)	(d)	

ND Not detected in gaseous effluents.
(a) ODCM Control 3.4.a for gamma-air dose. Percent values for ODCM Control 3.4.b for beta-air dose would be approximately the same.
(b) Iodine data have been deleted. These nuclides are no longer available for discharge.
(c) Per ODCM Control 3.5, the percentage of the limit is based on the combined dose contribution from iodines, tritium, and particulates with half lives greater than 8 days. Percent of limits are calculated using ODCM Method I dose equations.
(d) Per ODCM Control 3.5, percentage dose contribution from tritium is included in Part C.3.

TABLE 1A
(Sheet 2 of 2)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
Gaseous Effluents – Summation of All Releases

	Unit	Quarter 3	Quarter 4	Est. Total Error, %
A. Fission and Activation Gases				
1. Total Release	Ci	ND	ND	±2.50E+01
2. Average Release Rate for Period	µCi/sec	ND	ND	
3. Percent of Control Limit[a]	%	0.00E+00	0.00E+00	
B. Iodines[b]				
C. Particulates				
1. Particluates with Half-lives > 8 days	Ci	2.10E-08	3.55E-08	±3.00E+01
2. Average Release Rate for Period	µCi/sec	2.64E-09	4.47E-09	
3. Percent of Control Limit[c]	%	1.45E-03	8.29E-04	
4. Gross Alpha Radioactivity	Ci	ND	ND	
D. Tritium				
1. Total Release	Ci	3.63E-02	2.55E-02	±3.00E+01
2. Average Release Rate for Period	µCi/sec	4.57E-03	3.21E-03	
3. Percent of Control Limit	%	(d)	(d)	

ND Not detected in gaseous effluents.
(a) ODCM Control 3.4.a for gamma-air dose. Percent values for ODCM Control 3.4.b for beta-air dose would be approximately the same.
(b) Iodine data have been deleted. These nuclides are no longer available for discharge.
(c) Per ODCM Control 3.5, the percentage of the limit is based on the combined dose contribution from iodines, tritium, and particulates with half lives greater than 8 days. Percent of limits are calculated using ODCM Method I dose equations.
(d) Per ODCM Control 3.5, percentage dose contribution from tritium is included in Part C.3.

TABLE 1B
(Sheet 1 of 2)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
Gaseous Effluents – Elevated Releases

		Continuous Mode		Batch Mode[a]	
Nuclides Released	Unit	Quarter 1	Quarter 2	Quarter 1	Quarter 2
1. Fission Gases					
Krypton-85	Ci	ND	ND	-	-
Total for Period	Ci	ND	ND	-	-
2. Iodines[b]					
3. Particulates					
Strontium-89	Ci	ND	ND	-	-
Strontium-90	Ci	ND	ND	-	-
Cesium-134	Ci	ND	ND	-	-
Cesium-137	Ci	4.90E-08	ND	-	-
Zinc-65	Ci	ND	ND	-	-
Cobalt-58	Ci	ND	ND	-	-
Cobalt-60	Ci	4.27E-07	ND	-	-
Cerium-144	Ci	ND	ND	-	-
Manganese-54	Ci	ND	ND	-	-
Total for Period	Ci	4.76E-07	ND	-	-

ND Not detected in gaseous effluents.
(a) There are no longer any batch mode gaseous releases.
(b) Iodine-131, Iodine-133, and Iodine-135 activities have been deleted. These nuclides are no longer available for discharge.
- Dash indicates no release of this type.

TABLE 1B
(Sheet 2 of 2)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
Gaseous Effluents – Elevated Releases

		Continuous Mode		Batch Mode[a]	
Nuclides Released	Unit	Quarter 3	Quarter 4	Quarter 3	Quarter 4
1. Fission Gases					
Krypton-85	Ci	ND	ND	-	-
Total for Period	Ci	ND	ND	-	-
2. Iodines[b]					
3. Particulates					
Strontium-89	Ci	ND	ND	-	-
Strontium-90	Ci	ND	ND	-	-
Cesium-134	Ci	ND	ND	-	-
Cesium-137	Ci	ND	1.26E-08	-	-
Zinc-65	Ci	ND	ND	-	-
Cobalt-58	Ci	ND	ND	-	-
Cobalt-60	Ci	2.10E-08	2.29E-08	-	-
Cerium-144	Ci	ND	ND	-	-
Manganese-54	Ci	ND	ND	-	-
Total for Period	Ci	2.10E-08	3.55E-08	-	-

ND Not detected in gaseous effluents.
(a) There are no longer any batch mode gaseous releases.
(b) Iodine-131, Iodine-133, and Iodine-135 activities have been deleted. These nuclides are no longer available for discharge.
- Dash indicates no release of this type.

TABLE 1C
(Sheet 1 of 1)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
Gaseous Effluents – Ground Level Releases

There were no gaseous ground level releases during the reporting period.

TABLE 2A
(Sheet 1 of 2)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
Liquid Effluents – Summation of All Releases

	Unit	Quarter 1	Quarter 2	Est. Total Error, %
A. Fission and Activation Products				
1. Total Release (not including tritium, gases, alpha)	Ci	-	ND	±2.00E+01
2. Average Diluted Concentration During Period	µCi/ml	-	ND	
3. Percent of Applicable Limit[a]	%	-	0.00E+00	
B. Tritium				
1. Total Release	Ci	-	7.34E-04	±1.00E+01
2. Average Diluted Concentration During Period	µCi/ml	-	9.99E-09	
3. Percent of Applicable Limit[a]	%	-	3.33E-04	
C. Dissolved and Entrained Gases				
1. Total Release	Ci	-	ND	±2.00E+01
2. Average Diluted Concentration During Period	µCi/ml	-	ND	
3. Percent of Applicable Limit[b]	%	-	0.00E+00	
D. Gross Alpha Radioactivity				
1. Total Release	Ci	-	ND	±3.50E+01
E. Volume of Waste Release (prior to dilution)	Liters	0.00E+00	4.73E+04	±1.00E+01
F. Volume of Dilution Water Used During Period	Liters	7.29E+07	7.35E+07	±1.50E+01

ND Not detected in liquid effluents.
(a) Concentration limits specified in Appendix B to 10CFR20.1-20.602, Table II, Column 2 (ODCM Control 2.1). The percent of applicable limit reported is based on the average diluted concentration during the period. At no time did any release exceed the concentration limit.
(b) Concentration limits for dissolved and entrained noble gases is 2.00E-04 µCi/ml (ODCM Control 2.1). The percent of applicable limit reported is based on the average diluted concentration during the period. At no time did any release exceed the concentration limit.
- Dash indicates no release of this type.

TABLE 2A
(Sheet 2 of 2)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
Liquid Effluents – Summation of All Releases

	Unit	Quarter 3	Quarter 4	Est. Total Error, %
A. Fission and Activation Products				
1. Total Release (not including tritium, gases, alpha)	Ci	-	-	±2.00E+01
2. Average Diluted Concentration During Period	µCi/ml	-	-	
3. Percent of Applicable Limit[a]	%	-	-	
B. Tritium				
1. Total Release	Ci	-	-	±1.00E+01
2. Average Diluted Concentration During Period	µCi/ml	-	-	
3. Percent of Applicable Limit[a]	%	-	-	
C. Dissolved and Entrained Gases				
1. Total Release	Ci	-	-	±2.00E+01
2. Average Diluted Concentration During Period	µCi/ml	-	-	
3. Percent of Applicable Limit[b]	%	-	-	
D. Gross Alpha Radioactivity				
1. Total Release	Ci	-	-	±3.50E+01
E. Volume of Waste Release (prior to dilution)	Liters	0.00E+00	0.00E+00	±1.00E+01
F. Volume of Dilution Water Used During Period	Liters	6.70E+07	6.27E+07	±1.50E+01

ND Not detected in liquid effluents.
(a) Concentration limits specified in Appendix B to 10CFR20.1-20.602, Table II, Column 2 (ODCM Control 2.1). The percent of applicable limit reported is based on the average diluted concentration during the period. At no time did any release exceed the concentration limit.
(b) Concentration limits for dissolved and entrained noble gases is 2.00E-04 µCi/ml (ODCM Control 2.1). The percent of applicable limit reported is based on the average diluted concentration during the period. At no time did any release exceed the concentration limit.
- Dash indicates no release of this type.

TABLE 2B
(Sheet 1 of 2)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
Liquid Effluents – Routine Releases

		Continuous Mode		Batch Mode	
Nuclides Released	Unit	Quarter 1	Quarter 2	Quarter 1	Quarter 2
Strontium-89	Ci	-	-	-	ND
Strontium-90	Ci	-	-	-	ND
Cesium-134	Ci	-	-	-	ND
Cesium-137	Ci	-	-	-	ND
Cobalt-58	Ci	-	-	-	ND
Cobalt-60	Ci	-	-	-	ND
Zinc-65	Ci	-	-	-	ND
Manganese-54	Ci	-	-	-	ND
Cerium-144	Ci	-	-	-	ND
Carbon-14	Ci	-	-	-	ND
Iron-55	Ci	-	-	-	ND
Unidentified	Ci	-	-	-	ND
Total for Period (above)	Ci	-	-	-	ND
Krypton-85	Ci	-	-	-	ND

ND Not detected in liquid effluents.

- Dash indicates no release of this type.

TABLE 2B
(Sheet 2 of 2)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
Liquid Effluents – Routine Releases

		Continuous Mode		Batch Mode	
Nuclides Released	Unit	Quarter 3	Quarter 4	Quarter 3	Quarter 4
Strontium-89	Ci	-	-	-	-
Strontium-90	Ci	-	-	-	-
Cesium-134	Ci	-	-	-	-
Cesium-137	Ci	-	-	-	-
Cobalt-58	Ci	-	-	-	-
Cobalt-60	Ci	-	-	-	-
Zinc-65	Ci	-	-	-	-
Manganese-54	Ci	-	-	-	-
Cerium-144	Ci	-	-	-	-
Carbon-14	Ci	-	-	-	-
Iron-55	Ci	-	-	-	-
Unidentified	Ci	-	-	-	-
Total for Period (above)	Ci	-	-	-	-
Krypton-85	Ci	-	-	-	-

ND Not detected in liquid effluents.

- Dash indicates no release of this type.

TABLE 3
(Sheet 1 of 3)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
Solid Waste and Irradiated Fuel Shipments

I. First and Second Quarters

A. SOLID WASTE SHIPPED FOR BURIAL OR DISPOSAL (not irradiated fuel)

1. Type of Waste	Unit	6-month Period	Est. Total Error, %
a. Dry Active Waste: Class A	m^3	2.16E+03	+/-25%
Containers: steel box (46, 95, 100, 101, 290 ft^3), drum (7.2 ft^3), sealand (1280, 2180 ft^3), steel liner (201 ft^3)	Ci (est.)	5.76E-01	
b. Evaporator Concentrates: Class A	m^3	5.08E+02	+/-25%
Containers: sealand (1280 ft^3), steel box (100, 101 ft^3)	Ci (est.)	4.86E-02	
c. Dry Active Waste: Class A	m^3	2.87E+01	+/-25%
Containers (a)	Ci (est.)	4.05E-02	
d. Resins: Class C	m^3	3.41E+00	+/-25%
Containers: HIC (120 ft^3)	Ci (est.)	1.27E+01	

2. Estimate of Nuclide Composition > 1% (by type of waste)

a.	Iron-55	%	49.6
	Cobalt-60	%	21.1
	Nickel-63	%	22.6
	Cesium-137	%	3.8
	Cerium-144	%	1.0
b.	Tritium	%	3.0
	Iron-55	%	17.2
	Cobalt-60	%	19.8
	Nickel-63	%	28.4
	Strontium-90	%	4.5
	Cesium-137	%	25.2
c.	Iron-55	%	37.0
	Cobalt-60	%	20.5
	Nickel-63	%	37.3
	Cesium-137	%	3.5
d.	Iron-55	%	16.3
	Cobalt-60	%	39.2
	Nickel-63	%	16.2
	Cesium-137	%	1.0
	Cerium-144	%	26.8

(a) Partial shipments by the processor to disposal.

TABLE 3
(Sheet 2 of 3)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
Solid Waste and Irradiated Fuel Shipments

3. Solid Waste Disposition

Number of Shipments	Mode of Transportation	Destination
39	Truck	Clive, UT
23(a)	Truck	Clive, UT
11(b)	Truck	Oak Ridge, TN
3(b)	Truck	Memphis, TN
4(b)	Truck	Kingston, TN
2	Truck	Barnwell, SC

B. IRRADIATED FUEL SHIPMENTS (Disposition): None

II. Third and Fourth Quarters

A. SOLID WASTE SHIPPED FOR BURIAL OR DISPOSAL (not irradiated fuel)

1. Type of Waste	Unit	6-month Period	Est. Total Error, %
a. Dry Active Waste: Class A Containers: steel box (95, 100, 101 ft^3), sealand (1280 ft^3), drum (7.2 ft^3)	m^3 Ci (est.)	2.34E+02 1.76E-02	+/-25%
b. Dry Active Waste: Class A Containers (a)	m^3 Ci (est.)	1.93E+01 4.58E-03	+/-25%
c. Dry Active Waste: Class A Containers: GTS 6Pak (141 ft^3), steel box (95 ft^3), sealand (1280 ft^3)	m^3 Ci (est.)	1.18E+02 1.15E-01	+/-25%

(a) Partial shipments by the processor to disposal.
(b) Waste shipments to processor.

TABLE 3
(Sheet 3 of 3)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
Solid Waste and Irradiated Fuel Shipments

2. Estimate of Nuclide Composition > 1% (by type of waste)

a.	Tritium	%	1.2
	Iron-55	%	33.4
	Cobalt-60	%	19.3
	Nickel-63	%	31.0
	Cesium-137	%	13.7

b.	Iron-55	%	37.0
	Cobalt-60	%	20.5
	Nickel-63	%	37.3
	Cesium-137	%	3.5

c.	Iron-55	%	36.6
	Cobalt-60	%	20.5
	Nickel-63	%	37.5
	Cesium-137	%	3.7

3. Solid Waste Disposition

Number of Shipments	Mode of Transporation	Destination
4	Truck	Clive, UT
17(a)	Truck	Clive, UT
4(b)	Truck	Oak Ridge, TN
3(b)	Truck	Kingston, TN

B. IRRADIATED FUEL SHIPMENTS (Disposition): None

(a) Partial shipments by the processor to disposal.
(b) Waste shipments to processor.

TABLE 4
(Sheet 1 of 1)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
Maximum[a] Off-Site Doses and Dose Commitments to Members of the Public[b]

Source	Unit	1st Quarter	2nd Quarter	3rd Quarter	4th Quarter	Year[c]
1. Liquid Effluents						
Whole Body	mrem	0.00E+00[d]	2.35E-07[e]	0.00E+00[d]	0.00E+00[d]	2.35E-07
Critical Organ	mrem	0.00E+00[d]	2.35E-07[f]	0.00E+00[d]	0.00E+00[d]	2.35E-07
2. Airborne Effluents						
Tritium and Particulates	mrem	5.69E-05[g]	9.08E-05[h]	1.09E-04[i]	6.22E-05[j]	3.19E-04
Noble Gases (Beta Air)	mrad	0.00E+00[k]	0.00E+00[k]	0.00E+00[k]	0.00E+00[k]	0.00E+00
Noble Gases (Gamma Air)	mrad	0.00E+00[k]	0.00E+00[k]	0.00E+00[k]	0.00E+00[k]	0.00E+00

(a) "Maximum" means the largest fraction of corresponding 10 CFR Part 50, Appendix I, dose design objective.

(b) The numbered footnotes indicate the location of the dose receptor, age group, and organ, where appropriate.

(c) "Maximum" dose for the year is the sum of the maximum doses for each quarter. This results in a conservative yearly dose estimate, but still within the limits of 10 CFR Part 50.

(d) There were no liquid releases during the first, third and fourth quarters of 2000.

(e) Child.

(f) Liver, kidney, lung, GI-LII, and thyroid of child.

(g) SW, 1200 meters; lung of child.

(h) SW, 1200 meters; liver, kidney, lung, GI-LII, thyroid, and whole body of child.

(i) SW, 1200 meters; liver, lung, GI-LII, whole body, kidney, and thyroid of child.

(j) SW, 1200 meters; liver of child.

(k) There were no noble gases released during 2000.

TABLE 5
(Sheet 1 of 8)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
1992-1996 Meteorological Data Joint Frequency Distribution

1. 199.0 FT WIND DATA STABILITY CLASS A CLASS FREQUENCY (PERCENT) = .09

WIND DIRECTION FROM

SPEED(MPH)	N	NNE	NE	ENE	E	ESE	SE	SSE	S	SSW	SW	WSW	W	WNW	NW	NNW	VRBL	TOTAL
CALM	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
C-3	0	0	1	2	0	0	0	0	0	0	0	0	0	0	0	1	0	4
(1)	.00	.00	2.56	5.13	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	2.56	.00	10.26
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.01
4-7	0	0	0	0	0	0	0	1	0	2	0	0	0	0	0	0	0	3
(1)	.00	.00	.00	.00	.00	.00	.00	2.56	.00	5.13	.00	.00	.00	.00	.00	.00	.00	7.69
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.01
8-12	0	1	0	0	0	0	0	0	2	3	13	6	1	0	0	0	0	26
(1)	.00	2.56	.00	.00	.00	.00	.00	.00	5.13	7.69	33.33	15.38	2.56	.00	.00	.00	.00	66.67
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.01	.03	.01	.00	.00	.00	.00	.00	.06
13-18	0	0	0	0	0	0	0	0	0	0	5	1	0	0	0	0	0	6
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	12.82	2.56	.00	.00	.00	.00	.00	15.38
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.01	.00	.00	.00	.00	.00	.00	.01
19-24	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
GT 24	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
ALL SPEEDS	0	1	1	2	0	0	0	1	2	5	18	7	1	0	0	1	0	39
(1)	.00	2.56	2.56	5.13	.00	.00	.00	2.56	5.13	12.82	46.15	17.95	2.56	.00	.00	2.56	.00	100.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.01	.04	.02	.00	.00	.00	.00	.00	.09

(1)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PAGE
(2)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PERIOD
C=CALM (WIND SPEED LESS THAN OR EQUAL TO .95 MPH)

TABLE 5
(Sheet 2 of 8)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
1992-1996 Meteorological Data Joint Frequency Distribution

2. 199.0 FT WIND DATA STABILITY CLASS B CLASS FREQUENCY (PERCENT) = .36

WIND DIRECTION FROM

SPEED(MPH)	N	NNE	NE	ENE	E	ESE	SE	SSE	S	SSW	SW	WSW	W	WNW	NW	NNW	VRBL	TOTAL
CALM	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
C-3	1	1	1	1	1	0	0	0	0	0	0	0	0	0	0	0	0	5
(1)	.68	.68	.68	.68	.68	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	3.42
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.01
4-7	1	0	0	0	0	0	0	2	11	5	6	3	1	0	0	0	0	29
(1)	.68	.00	.00	.00	.00	.00	.00	1.37	7.53	3.42	4.11	2.05	.68	.00	.00	.00	.00	19.86
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.03	.01	.01	.01	.00	.00	.00	.00	.00	.07
8-12	0	1	0	0	0	0	0	3	6	14	49	20	0	0	0	0	0	93
(1)	.00	.68	.00	.00	.00	.00	.00	2.05	4.11	9.59	33.56	13.70	.00	.00	.00	.00	.00	63.70
(2)	.00	.00	.00	.00	.00	.00	.00	.01	.01	.03	.12	.05	.00	.00	.00	.00	.00	.23
13-18	0	1	0	0	0	0	0	0	0	0	12	6	0	0	0	0	0	19
(1)	.00	.68	.00	.00	.00	.00	.00	.00	.00	.00	8.22	4.11	.00	.00	.00	.00	.00	13.01
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.03	.01	.00	.00	.00	.00	.00	.05
19-24	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
GT 24	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
ALL SPEEDS	2	3	1	1	1	0	0	5	17	19	67	29	1	0	0	0	0	146
(1)	1.37	2.05	.68	.68	.68	.00	.00	3.42	11.64	13.01	45.89	19.86	.68	.00	.00	.00	.00	100.00
(2)	.00	.01	.00	.00	.00	.00	.00	.01	.04	.05	.16	.07	.00	.00	.00	.00	.00	.36

(1)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PAGE
(2)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PERIOD
C=CALM (WIND SPEED LESS THAN OR EQUAL TO .95 MPH)

TABLE 5
(Sheet 3 of 8)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
1992-1996 Meteorological Data Joint Frequency Distribution

3. 199.0 FT WIND DATA STABILITY CLASS C CLASS FREQUENCY (PERCENT) = 1.24

WIND DIRECTION FROM

SPEED(MPH)	N	NNE	NE	ENE	E	ESE	SE	SSE	S	SSW	SW	WSW	W	WNW	NW	NNW	VRBL	TOTAL
CALM	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
C-3	1	1	5	3	0	0	0	1	1	0	0	0	0	0	1	0	0	13
(1)	.20	.20	.98	.59	.00	.00	.00	.20	.20	.00	.00	.00	.00	.00	.20	.00	.00	2.55
(2)	.00	.00	.01	.01	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.03
4-7	0	3	2	0	1	3	2	17	24	21	22	16	6	3	1	0	0	121
(1)	.00	.59	.39	.00	.20	.59	.39	3.33	4.71	4.12	4.31	3.14	1.18	.59	.20	.00	.00	23.73
(2)	.00	.01	.00	.00	.00	.01	.00	.04	.06	.05	.05	.04	.01	.01	.00	.00	.00	.29
8-12	1	4	1	0	0	0	2	7	17	42	122	107	10	2	1	2	0	318
(1)	.20	.78	.20	.00	.00	.00	.39	1.37	3.33	8.24	23.92	20.98	1.96	.39	.20	.39	.00	62.35
(2)	.00	.01	.00	.00	.00	.00	.00	.02	.04	.10	.30	.26	.02	.00	.00	.00	.00	.77
13-18	2	2	0	0	0	0	0	0	1	4	20	27	2	0	0	0	0	58
(1)	.39	.39	.00	.00	.00	.00	.00	.00	.20	.78	3.92	5.29	.39	.00	.00	.00	.00	11.37
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.01	.05	.07	.00	.00	.00	.00	.00	.14
19-24	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
GT 24	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
ALL SPEEDS	4	10	8	3	1	3	4	25	43	67	164	150	18	5	3	2	0	510
(1)	.78	1.96	1.57	.59	.20	.59	.78	4.90	8.43	13.14	32.16	29.41	3.53	.98	.59	.39	.00	100.00
(2)	.01	.02	.02	.01	.00	.01	.01	.06	.10	.16	.40	.36	.04	.01	.01	.00	.00	1.24

(1)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PAGE
(2)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PERIOD
C=CALM (WIND SPEED LESS THAN OR EQUAL TO .95 MPH)

TABLE 5
(Sheet 4 of 8)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
1992-1996 Meteorological Data Joint Frequency Distribution

4. 199.0 FT WIND DATA STABILITY CLASS D CLASS FREQUENCY (PERCENT) = 46.68

WIND DIRECTION FROM

SPEED (MPH)	N	NNE	NE	ENE	E	ESE	SE	SSE	S	SSW	SW	WSW	W	WNW	NW	NNW	VRBL	TOTAL
CALM	0	0	0	0	0	0	0	0	0	0	0	0	0	1	0	0	0	1
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.01	.00	.00	.00	.01
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
C-3	299	588	303	121	89	87	85	80	81	132	224	161	97	79	99	124	0	2649
(1)	1.56	3.07	1.58	.63	.46	.45	.44	.42	.42	.69	1.17	.84	.51	.41	.52	.65	.00	13.81
(2)	.73	1.43	.74	.29	.22	.21	.21	.19	.20	.32	.55	.39	.24	.19	.24	.30	.00	6.45
4-7	786	1407	437	174	174	183	237	322	561	795	1107	761	509	358	376	454	0	8641
(1)	4.10	7.33	2.28	.91	.91	.95	1.24	1.68	2.92	4.14	5.77	3.97	2.65	1.87	1.96	2.37	.00	45.05
(2)	1.91	3.42	1.06	.42	.42	.45	.58	.78	1.36	1.93	2.69	1.85	1.24	.87	.91	1.10	.00	21.02
8-12	935	1264	163	35	37	32	39	82	162	525	792	942	569	258	241	441	0	6517
(1)	4.87	6.59	.85	.18	.19	.17	.20	.43	.84	2.74	4.13	4.91	2.97	1.34	1.26	2.30	.00	33.97
(2)	2.27	3.08	.40	.09	.09	.08	.09	.20	.39	1.28	1.93	2.29	1.38	.63	.59	1.07	.00	15.86
13-18	216	446	19	3	0	2	1	1	11	39	172	265	68	13	11	47	0	1314
(1)	1.13	2.32	.10	.02	.00	.01	.01	.01	.06	.20	.90	1.38	.35	.07	.06	.25	.00	6.85
(2)	.53	1.09	.05	.01	.00	.00	.00	.00	.03	.09	.42	.64	.17	.03	.03	.11	.00	3.20
19-24	12	28	1	0	0	0	0	0	0	2	5	10	2	0	0	1	0	61
(1)	.06	.15	.01	.00	.00	.00	.00	.00	.00	.01	.03	.05	.01	.00	.00	.01	.00	.32
(2)	.03	.07	.00	.00	.00	.00	.00	.00	.00	.00	.01	.02	.00	.00	.00	.00	.00	.15
GT 24	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
ALL SPEEDS	2248	3733	923	333	300	304	362	485	815	1493	2300	2139	1245	709	727	1067	0	19183
(1)	11.72	19.46	4.81	1.74	1.56	1.58	1.89	2.53	4.25	7.78	11.99	11.15	6.49	3.70	3.79	5.56	.00	100.00
(2)	5.47	9.08	2.25	.81	.73	.74	.88	1.18	1.98	3.63	5.60	5.20	3.03	1.73	1.77	2.60	.00	46.68

(1)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PAGE
(2)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PERIOD
C=CALM (WIND SPEED LESS THAN OR EQUAL TO .95 MPH)

TABLE 5
(Sheet 5 of 8)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
1992-1996 Meteorological Data Joint Frequency Distribution

5. 199.0 FT WIND DATA STABILITY CLASS E CLASS FREQUENCY (PERCENT) = 40.08

WIND DIRECTION FROM

SPEED(MPH)	N	NNE	NE	ENE	E	ESE	SE	SSE	S	SSW	SW	WSW	W	WNW	NW	NNW	VRBL	TOTAL
CALM	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
C-3	999	2950	1363	527	310	271	231	250	295	401	523	315	191	171	204	309	0	9310
(1)	6.06	17.91	8.27	3.20	1.88	1.65	1.40	1.52	1.79	2.43	3.17	1.91	1.16	1.04	1.24	1.88	.00	56.51
(2)	2.43	7.18	3.32	1.28	.75	.66	.56	.61	.72	.98	1.27	.77	.46	.42	.50	.75	.00	22.65
4-7	455	2014	392	83	66	59	74	168	242	501	730	340	172	92	151	164	0	5703
(1)	2.76	12.23	2.38	.50	.40	.36	.45	1.02	1.47	3.04	4.43	2.06	1.04	.56	.92	1.00	.00	34.62
(2)	1.11	4.90	.95	.20	.16	.14	.18	.41	.59	1.22	1.78	.83	.42	.22	.37	.40	.00	13.88
8-12	121	331	52	2	1	4	2	17	38	238	176	122	55	22	22	33	0	1236
(1)	.73	2.01	.32	.01	.01	.02	.01	.10	.23	1.44	1.07	.74	.33	.13	.13	.20	.00	7.50
(2)	.29	.81	.13	.00	.00	.01	.00	.04	.09	.58	.43	.30	.13	.05	.05	.08	.00	3.01
13-18	16	64	7	2	0	0	0	0	4	40	37	26	6	0	1	4	0	207
(1)	.10	.39	.04	.01	.00	.00	.00	.00	.02	.24	.22	.16	.04	.00	.01	.02	.00	1.26
(2)	.04	.16	.02	.00	.00	.00	.00	.00	.01	.10	.09	.06	.01	.00	.00	.01	.00	.50
19-24	0	7	0	0	0	0	0	0	0	0	9	2	0	0	0	0	0	18
(1)	.00	.04	.00	.00	.00	.00	.00	.00	.00	.00	.05	.01	.00	.00	.00	.00	.00	.11
(2)	.00	.02	.00	.00	.00	.00	.00	.00	.00	.00	.02	.00	.00	.00	.00	.00	.00	.04
GT 24	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
ALL SPEEDS	1591	5366	1814	614	377	334	307	435	579	1180	1475	805	424	285	378	510	0	16474
(1)	9.66	32.57	11.01	3.73	2.29	2.03	1.86	2.64	3.51	7.16	8.95	4.89	2.57	1.73	2.29	3.10	.00	100.00
(2)	3.87	13.06	4.41	1.49	.92	.81	.75	1.06	1.41	2.87	3.59	1.96	1.03	.69	.92	1.24	.00	40.08

(1)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PAGE
(2)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PERIOD
C=CALM (WIND SPEED LESS THAN OR EQUAL TO .95 MPH)

TABLE 5
(Sheet 6 of 8)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
1992-1996 Meteorological Data Joint Frequency Distribution

6. 199.0 FT WIND DATA STABILITY CLASS F CLASS FREQUENCY (PERCENT) = 8.76

WIND DIRECTION FROM

SPEED(MPH)	N	NNE	NE	ENE	E	ESE	SE	SSE	S	SSW	SW	WSW	W	WNW	NW	NNW	VRBL	TOTAL
CALM	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
C-3	156	433	448	220	148	132	112	86	105	133	152	99	92	50	71	83	0	2520
(1)	4.33	12.02	12.44	6.11	4.11	3.67	3.11	2.39	2.92	3.69	4.22	2.75	2.55	1.39	1.97	2.30	.00	69.98
(2)	.38	1.05	1.09	.54	.36	.32	.27	.21	.26	.32	.37	.24	.22	.12	.17	.20	.00	6.13
4-7	69	257	83	27	17	10	25	39	54	100	144	68	38	32	34	26	0	1023
(1)	1.92	7.14	2.30	.75	.47	.28	.69	1.08	1.50	2.78	4.00	1.89	1.06	.89	.94	.72	.00	28.41
(2)	.17	.63	.20	.07	.04	.02	.06	.09	.13	.24	.35	.17	.09	.08	.08	.06	.00	2.49
8-12	4	13	0	0	0	0	0	0	2	6	14	7	4	1	0	0	0	51
(1)	.11	.36	.00	.00	.00	.00	.00	.00	.06	.17	.39	.19	.11	.03	.00	.00	.00	1.42
(2)	.01	.03	.00	.00	.00	.00	.00	.00	.00	.01	.03	.02	.01	.00	.00	.00	.00	.12
13-18	1	5	0	0	0	0	0	0	0	0	1	0	0	0	0	0	0	7
(1)	.03	.14	.00	.00	.00	.00	.00	.00	.00	.00	.03	.00	.00	.00	.00	.00	.00	.19
(2)	.00	.01	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.02
19-24	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
GT 24	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
ALL SPEEDS	230	708	531	247	165	142	137	125	161	239	311	174	134	83	105	109	0	3601
(1)	6.39	19.66	14.75	6.86	4.58	3.94	3.80	3.47	4.47	6.64	8.64	4.83	3.72	2.30	2.92	3.03	.00	100.00
(2)	.56	1.72	1.29	.60	.40	.35	.33	.30	.39	.58	.76	.42	.33	.20	.26	.27	.00	8.76

(1)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PAGE
(2)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PERIOD
C=CALM (WIND SPEED LESS THAN OR EQUAL TO .95 MPH)

TABLE 5
(Sheet 7 of 8)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
1992-1996 Meteorological Data Joint Frequency Distribution

7. 199.0 FT WIND DATA STABILITY CLASS G CLASS FREQUENCY (PERCENT) = 2.79

WIND DIRECTION FROM

SPEED(MPH)	N	NNE	NE	ENE	E	ESE	SE	SSE	S	SSW	SW	WSW	W	WNW	NW	NNW	VRBL	TOTAL
CALM	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
C-3	30	88	83	54	60	41	47	46	44	41	52	31	15	13	16	16	0	677
(1)	2.62	7.68	7.24	4.71	5.24	3.58	4.10	4.01	3.84	3.58	4.54	2.71	1.31	1.13	1.40	1.40	.00	59.08
(2)	.07	.21	.20	.13	.15	.10	.11	.11	.11	.10	.13	.08	.04	.03	.04	.04	.00	1.65
4-7	14	54	27	16	12	9	18	38	26	73	86	29	14	10	10	4	0	440
(1)	1.22	4.71	2.36	1.40	1.05	.79	1.57	3.32	2.27	6.37	7.50	2.53	1.22	.87	.87	.35	.00	38.39
(2)	.03	.13	.07	.04	.03	.02	.04	.09	.06	.18	.21	.07	.03	.02	.02	.01	.00	1.07
8-12	0	3	0	0	0	0	0	0	3	8	8	4	2	0	0	0	0	28
(1)	.00	.26	.00	.00	.00	.00	.00	.00	.26	.70	.70	.35	.17	.00	.00	.00	.00	2.44
(2)	.00	.01	.00	.00	.00	.00	.00	.00	.01	.02	.02	.01	.00	.00	.00	.00	.00	.07
13-18	0	0	0	0	0	0	0	0	0	0	1	0	0	0	0	0	0	1
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.09	.00	.00	.00	.00	.00	.00	.09
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
19-24	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
GT 24	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
ALL SPEEDS	44	145	110	70	72	50	65	84	73	122	147	64	31	23	26	20	0	1146
(1)	3.84	12.65	9.60	6.11	6.28	4.36	5.67	7.33	6.37	10.65	12.83	5.58	2.71	2.01	2.27	1.75	.00	100.00
(2)	.11	.35	.27	.17	.18	.12	.16	.20	.18	.30	.36	.16	.08	.06	.06	.05	.00	2.79

(1)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PAGE
(2)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PERIOD
C=CALM (WIND SPEED LESS THAN OR EQUAL TO .95 MPH)

TABLE 5
(Sheet 8 of 8)

Yankee Atomic Electric Company, Rowe, Massachusetts
2000 Annual Radioactive Effluent Release Report
1992-1996 Meteorological Data Joint Frequency Distribution

8. 199.0 FT WIND DATA STABILITY CLASS ALL CLASS FREQUENCY (PERCENT) = 100.00

WIND DIRECTION FROM

SPEED(MPH)	N	NNE	NE	ENE	E	ESE	SE	SSE	S	SSW	SW	WSW	W	WNW	NW	NNW	VRBL	TOTAL
CALM	0	0	0	0	0	0	0	0	0	0	0	0	0	1	0	0	0	1
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
C-3	1486	4061	2204	928	608	531	475	463	526	707	951	606	395	313	391	533	0	15178
(1)	3.62	9.88	5.36	2.26	1.48	1.29	1.16	1.13	1.28	1.72	2.31	1.47	.96	.76	.95	1.30	.00	36.93
(2)	3.62	9.88	5.36	2.26	1.48	1.29	1.16	1.13	1.28	1.72	2.31	1.47	.96	.76	.95	1.30	.00	36.93
4-7	1325	3735	941	300	270	264	356	587	918	1497	2095	1217	740	495	572	648	0	15960
(1)	3.22	9.09	2.29	.73	.66	.64	.87	1.43	2.23	3.64	5.10	2.96	1.80	1.20	1.39	1.58	.00	38.83
(2)	3.22	9.09	2.29	.73	.66	.64	.87	1.43	2.23	3.64	5.10	2.96	1.80	1.20	1.39	1.58	.00	38.83
8-12	1061	1617	216	37	38	36	43	109	230	836	1174	1208	641	283	264	476	0	8269
(1)	2.58	3.93	.53	.09	.09	.09	.10	.27	.56	2.03	2.86	2.94	1.56	.69	.64	1.16	.00	20.12
(2)	2.58	3.93	.53	.09	.09	.09	.10	.27	.56	2.03	2.86	2.94	1.56	.69	.64	1.16	.00	20.12
13-18	235	518	26	5	0	2	1	1	16	83	248	325	76	13	12	51	0	1612
(1)	.57	1.26	.06	.01	.00	.00	.00	.00	.04	.20	.60	.79	.18	.03	.03	.12	.00	3.92
(2)	.57	1.26	.06	.01	.00	.00	.00	.00	.04	.20	.60	.79	.18	.03	.03	.12	.00	3.92
19-24	12	35	1	0	0	0	0	0	0	2	14	12	2	0	0	1	0	79
(1)	.03	.09	.00	.00	.00	.00	.00	.00	.00	.00	.03	.03	.00	.00	.00	.00	.00	.19
(2)	.03	.09	.00	.00	.00	.00	.00	.00	.00	.00	.03	.03	.00	.00	.00	.00	.00	.19
GT 24	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
(1)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
(2)	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00	.00
ALL SPEEDS	4119	9966	3388	1270	916	833	875	1160	1690	3125	4482	3368	1854	1105	1239	1709	0	41099
(1)	10.02	24.25	8.24	3.09	2.23	2.03	2.13	2.82	4.11	7.60	10.91	8.19	4.51	2.69	3.01	4.16	.00	100.00
(2)	10.02	24.25	8.24	3.09	2.23	2.03	2.13	2.82	4.11	7.60	10.91	8.19	4.51	2.69	3.01	4.16	.00	100.00

(1)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PAGE
(2)=PERCENT OF ALL GOOD OBSERVATIONS FOR THIS PERIOD
C=CALM (WIND SPEED LESS THAN OR EQUAL TO .95 MPH)

APPENDIX A

Radioactive Liquid Effluent Monitoring Instrumentation

Requirement: Radioactive liquid effluent monitoring instrumentation channels are required to be operable in accordance with ODCM Control 5.1. With less than the minimum number of channels operable and reasonable efforts to return the instrument(s) to operable status within 30 days being unsuccessful, ODCM Control 5.1 requires an explanation for the delay in correcting the inoperability in the next Annual Radioactive Effluent Release Report.

Response: *The requirements of ODCM Control 5.1 governing the operability of radioactive liquid effluent monitoring instrumentation were met for this reporting period.*

APPENDIX B

Radioactive Gaseous Effluent Monitoring Instrumentation

Requirement: Radioactive gaseous effluent monitoring instrumentation channels are required to be operable in accordance with ODCM Control 5.2. With less than the minimum number of channels operable and reasonable efforts to return the instrument(s) to operable status within 30 days being unsuccessful, ODCM Control 5.2 requires an explanation for the delay in correcting the inoperability in the next Annual Radioactive Effluent Release Report.

Response: *The requirements of ODCM Control 5.2 governing the operability of radioactive gaseous effluent monitoring instrumentation were met for this reporting period.*

APPENDIX C

Liquid Holdup Tanks

Requirement: Defueled Technical Specification 3.4 limits the quantity of radioactive material contained in any outside temporary tank. With the quantity of radioactive material in any outside temporary tank exceeding the limits of Technical Specification 3.4, a description of the events leading to this condition is required in the next Annual Radioactive Effluent Release Report.

Response: *The limits of Technical Specification 3.4 were not exceeded during this reporting period.*

APPENDIX D

Radiological Environmental Monitoring Program

Requirement: The Radiological Environmental Monitoring Program is conducted in accordance with ODCM Control 4.1. With the Radiological Environmental Monitoring Program not being conducted a specified in Table 4.1, ODCM Control 4.1 requires a description of the reasons for not conducting the program as required and the plans for preventing a recurrence be included in the next Annual Radioactive Effluent Report.

Response: *The requirements of ODCM Control 4.1 governing the conduction of the REMP were met for this reporting period.*

Requirement: With milk samples no longer available from one or more of the required sample locations, ODCM Control 4.1 requires the identification of the new location(s) for obtaining replacement sample(s) in the next Annual Radioactive Effluent Release Report and inclusion of revised Off-Site Dose Calculation Manual figure(s) and table(s) reflecting the new location(s).

Response: *A total of two milk sampling locations are called for in the REMP; one indicator location and one control. However, the most recent Land Use Census found no locations within five miles from which milk samples can be collected. If future Land Use Census identifies an available sampling location within five miles, it will be included in the REMP.*

APPENDIX E

Land Use Census

Requirement: A land use census is conducted in accordance with ODCM Control 4.2. With a land use census identifying a location(s) which yields at least a 20 percent greater dose or dose commitment than the values currently being calculated in ODCM Control 3.5, ODCM Control 4.2 requires the identification of the new location(s) in the next Annual Radioactive Effluent Release Report.

Response: *The land use census for this reporting period did not identify any locations yielding at least 20 percent greater dose or dose commitment than the values currently being calculated in ODCM Control 3.5.*

Requirement: With a land use census identifying a location(s) which yields a calculated dose or dose commitment (via the same exposure pathway) at least 20 percent greater than at a location from which samples are currently being obtained in accordance with ODCM Control 4.1, ODCM Control 4.2 requires that the new location(s) be added to the Radiological Environmental Monitoring Program if permission from the owner to collect samples can be obtained and sufficient sample volume is available. If a new location is found, then it must be identified in the next Annual Radioactive Effluent Release Report.

Response: *No new locations were added to the Radiological Environmental Monitoring Program as a result of the 2000 land use census.*

APPENDIX F

Process Control Program (PCP)

Requirement: PCP Control 2.0 requires that licensee-initiated changes to the PCP be submitted to the Commission in the Annual Radioactive Effluent Release Report for the period in which the change(s) was made.

Response: *There were no licensee-initiated changes to the PCP during this reporting period.*

APPENDIX G

Off-Site Dose Calculation Manual (ODCM)

Requirement: ODCM Control 7.2 requires that licensee-initiated changes to the ODCM be submitted to the Commission in the Annual Radioactive Effluent Release Report for the period in which the change(s) was made effective.

Response: *Revision 14 to the ODCM was issued during the reporting period. The following explanation provides the basis for this revision.*

Table 5.1 of the Yankee ODCM lists the required radioactive liquid effluent monitoring instrument channels. Notes to the table provide remedial actions if a required monitoring channel is not operable. In the current plant configuration, one liquid radiation instrument performs a dual role by providing monitoring for both Test Tank releases and Auxiliary Service Water monitoring for potential leaks from Spent Fuel Pit (SFP) cooling operation. Because of this dual use, two different remedial ACTION statements are provided to identify what needs to be done depending which of the monitoring roles the liquid effluent channel is being applied to.

A review of plant procedures identified the need for additional ODCM clarifications (in the ACTION STATEMENTS to Table 5.1) to the user on what needs to be done if the liquid effluent radiation monitor is declared inoperable.

The single liquid effluent monitoring instrument is situated on the ASW line down stream of both the liquid Test Tank discharge line and the SFP heat exchanger used for the periodic cooling of the Spent Fuel Pit. Table 5.1, ACTION 1 refers back to the use of the radiation monitor in its role as a gross radioactivity instrument providing automatic isolation of the liquid discharge from the Test Tanks (batch releases) on the detection of radioactive concentrations in excess of the expected values or concentration limits of 10CFR20, Appendix B. The liquid waste effluent line is designed to contain radioactive materials. Therefore, the automatic alarm/valve trip function provided by the monitor for the Test Tank discharge line gives assurance that all required conditions for radioactive effluent releases will be met or releases stopped. If the monitor is out of service, Test Tank releases can continue with the stipulated ACTIONs taken to ensure that radioactivity released from the tanks are within the values expected.

Table 5.1, Action 4 refers back to the monitor when its intended use is to provide alarm for unexpected gross radioactivity in the ASW while Spent Fuel Pit cooling is going on. The ASW is expected to be clean of any plant radioactivity during normal operations. In this mode of use, the instrument channel's detection of gross radioactivity provides indication of a potential leak of contamination into the ASW.

Both ACTION Statements provide operational flexibility. Discharges of liquid

waste from the Test Tanks can be resumed once the appropriate actions are taken to ensure that radioactive materials released from the plant will be quantified and within release limits. Similarly, continued cooling of the Spent Fuel Pit with ASW flow (as needed to ensure proper temperature control of the pool) can be re-established once additional effluent sampling and analysis for potential contamination leaks is implemented.

The clarification added to ACTION 1 on Table 5.1 states that the ASW flow shall be secured if the radiation monitor is not operable. This statement is consistent with the existing requirement in ACTION 4 that requires the ASW flow be stopped if the same radiation monitor (now looking at the ASW stream alone) is declared inoperable. The addition of note c. to ACTION 1 clearly states that ASW flow can be re-established if Test Tank discharges are planned. This addition is consistent with the existing requirements of ACTION 4 that already allows effluent releases (i.e., ASW flow) to continue if appropriate grab sampling and analysis is implemented. This same grab sampling and analysis of ASW flow during Test Tank discharges is implied by the new note c to ACTION 1 when the effluent monitor is out of service.

The text changes to ACTION Statements 1 and 4 do not change the original intent of either remedial ACTION to allow continued effluent releases from the Test Tanks and the ASW system as long as appropriate administrate controls, sampling, and analyses are performed to ensure that any radioactive materials discharged from the plant are controlled and quantified. ASW flow is assumed available for dilution of Test Tank discharges whether the monitor is in service or not. The clarification added to ACTION 1 to secure ASW flow if radioactive releases are suspended, or continuation of ASW flow if radioactive effluents (or potential radioactive effluents) are to be discharged (along with appropriate sampling and analysis requirements), is in step with the conditions for ASW flow previously stated in ACTION 4. The rewording of ACTION 4 to clearly state these same conditions of securing ASW flow if the monitor is out of service, and allowing for its re-establishment if SFP cooling, or Test Tank releases are required, is not a change from the intent of the existing ODCM requirements. These changes are therefore administrative in nature. No new effluent release points have been added to the plant configuration, nor any increase in expected quantities of radioactivity expected to be released to the environment. No changes have been made to any dose calculations or setpoint methodologies contained in the ODCM as part of this revision.

It is therefore determined that the changes to the ODCM (Revision 14) will maintain the level of radioactive effluent control required by 10CFR20.106 (10CFR20.1301), 40CFR190, 10CFR50.36a, and Appendix I to 10CFR50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.

A copy of the resulting revised ODCM is attached to this report.

APPENDIX H

Radioactive Liquid, Gaseous, and Solid Waste Treatment Systems

Requirement: ODCM Control 7.3 requires that licensee-initiated major changes to the radioactive waste systems (liquid, gaseous, and solid) be reported to the Commission in the Annual Radioactive Effluent Release Report for the period in which the evaluation was reviewed by the Plant Operation Review Committee.

Response: *There were no licensee-initiated changes to the radioactive liquid, gaseous, or solid waste treatment systems during this reporting period.*

APPENDIX I

Supplemental Information

1. Control Limits – Dose and Dose Rate

Control and Category	Limit
a. Noble Gases	
Control 3.3, Total Body Dose Rate	500 mrem/year
Control 3.3, Skin Dose Rate	3000 mrem/year
Control 3.4, Gamma Air Dose	5 mrad/quarter
Control 3.4, Gamma Air Dose	10 mrad/year
Control 3.4, Beta Air Dose	10 mrad/quarter
Control 3.4, Beta Air Dose	20 mrad/year
b. Iodine-131, Tritium, and Radionuclides in Particulate Form With Half-Lives Greater Than 8 Days	
Control 3.3, Organ Dose Rate	1500 mrem/year
Control 3.5, Organ Dose	7.5 mrem/quarter
Control 3.5, Organ Dose	15 mrem/year
c. Liquids	
Control 3.1, Total Body Dose	1.5 mrem/quarter
Control 3.1, Total Body Dose	3 mrem/year
Control 3.1, Organ Dose	5 mrem/quarter
Control 3.1, Organ Dose	10 mrem/year

2. Control Limits – Concentration

Control and Category	Limit
a. Liquids	
Control 2.1, Total Sum of the Fraction of MPC (10CFR20, Appendix B, Table II, Column 2), excluding Noble Gases less than:	1.0
Control 2.1, Total Noble Gas Concentration	2.00E-4 μCi/cc

3. Measurements and Approximations of Total Radioactivity

a. Noble Gases, Krypton-85

Continuous discharges are determined by direct measurements. A primary vent stack gas sample is taken monthly and analyzed for Krypton-85. A review of the weekly primary vent stack noble gas integrator readings for any increase in values above the background level also is used as a reference. There are no longer any batch discharges. Errors associated with the above measurements are estimated to be ±25 percent.

b. Iodines, Particulates

There are no longer any iodine isotopes available for discharge. The sampling system design requires the use of a charcoal cartridge as a support for the particulate filter during particulate collection. The sampling system continuously draws a sample from the primary vent stack through a filter and charcoal cartridge. The particulate filter is removed and analyzed weekly. The errors associated with the determination of particulate effluents are estimated to be ±30 percent.

c. Liquid Effluents

A gamma isotopic analysis is performed on a representative sample using a Marinelli Beaker geometry for both a batch or continuous discharge. Composite samples for batch and continuous discharges are analyzed for strontium-89, strontium-90, iron-55, gross alpha activity, and carbon-14.

Tritium analysis is performed on composite samples for continuous discharges and on each batch discharge. The errors associated with these measurements are as follows: fission and activation products, ±20 percent; tritium, ±10 percent; dissolved fission gases, ±20 percent; and alpha activity, ±35 percent.

4. Batch Releases

a. Liquids

First Quarter	Routine Batches
Number of batch releases	0
Total time period for batch releases (minutes)	0
Maximum time period for a batch release (minutes)	0
Average time period for batch releases (minutes)	0
Minimum time period for a batch release (minutes)	0
Average stream flow (Sherman Dam) during period (cfs)	856
Average discharge rate (gpm)	0

Second Quarter	Routine Batches
Number of batch releases	2
Total time period for batch releases (minutes)	2,345
Maximum time period for a batch release (minutes)	1,455
Average time period for batch releases (minutes)	1,173
Minimum time period for a batch release (minutes)	890
Average stream flow (Sherman Dam) during period (cfs)	754
Average discharge rate (gpm)	5.3

Third Quarter	Routine Batches
Number of batch releases	0
Total time period for batch releases (minutes)	0
Maximum time period for a batch release (minutes)	0
Average time period for batch releases (minutes)	0
Minimum time period for a batch release (minutes)	0
Average stream flow (Sherman Dam) during period (cfs)	680
Average discharge rate (gpm)	0

Fourth Quarter	Routine Batches
Number of batch releases	0
Total time period for batch releases (minutes)	0
Maximum time period for a batch release (minutes)	0
Average time period for batch releases (minutes)	0
Minimum time period for a batch release (minutes)	0
Average stream flow (Sherman Dam) during period (cfs)	411
Average discharge rate (gpm)	0

b. Gases

There are no longer any batch-mode gaseous releases associated with plant systems.

5. Abnormal Releases

ODCM Control 7.2 requires the reporting of any unplanned releases from the site to the site boundary of radioactive material in gaseous and liquid effluents made during the reporting period.

a. Liquid

There were no non-routine liquid releases during the reporting period.

b. Gases

There were no non-routine gaseous releases during the reporting period.

APPENDIX J

Sewage Sludge Disposal

Requirement: For periods in which disposal of septage occurs, the licensee shall report in the Annual Radioactive Effluent Release Report, the volume discharged, liquid and solid fractions, and total activity discharged.

Response: The following information is provided for the disposal of sewage sludge during the reporting period:

Volume discharge (gallons):	6,500
Liquid fraction (by weight) of waste:	0.90
Solid fraction (by weight) of waste:	0.10
Nuclide content in liquid fraction (µCi/g):	not detectable
Nuclide content in solid fraction (µCi/g wet):	
Cesium 137:	6.88E-08
Cobalt 60:	2.60E-07
Total activity discharge (µCi):	0.81
Cesium 137:	0.17
Cobalt 60:	0.64