

September 6, 1995

Mr. John J. Barton  
Vice President and Director  
GPU Nuclear Corporation  
Oyster Creek Nuclear Generating Station  
Post Office Box 388  
Forked River, NJ 08731

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M92772)

Dear Mr. Barton:

The Commission has issued the enclosed Amendment No. 182 to Facility Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station, in response to your application dated June 26, 1995.

The amendment revises the snubber visual inspection intervals to match the schedule developed by the NRC staff for use with a 24-month refueling interval. This schedule was documented in Generic Letter 90-09. The amendment also revises the bases for the snubber visual inspection interval to be consistent with the bases described in Generic Letter 90-09.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Alexander W. Dromerick, Senior Project Manager  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-219

- Enclosures: 1. Amendment No. 182 to DPR-16
- 2. Safety Evaluation

cc w/encls: See next page

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DATE	08/12/95	07/28/95	07/28/95	08/12/95	08/19/95

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one more correction DO NOT ISS UP before Sept -2, 1995



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in cursive script that reads "Alexander W. Dromerick".

Alexander W. Dromerick, Senior Project Manager  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-219

Enclosures: 1. Amendment No. 182 to DPR-16  
2. Safety Evaluation

cc w/encls: See next page

J. Barton

Oyster Creek Nuclear  
Generating Station

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 182  
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by GPU Nuclear Corporation, et al. (the licensee) dated June 26, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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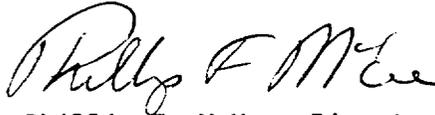
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.182 , are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Phillip F. McKee, Director  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: September 6, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 182

FACILITY OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace the following pages of the Appendix A, Technical Specifications, with the attached pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

4.5-9  
4.5-15  
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Insert

4.5-9  
4.5-15  
4.5-18  
4.5-19

Q. Shock Suppressors (Snubbers)

As used in this specification, "type of snubber" shall mean snubbers of the same design and manufacturer, irrespective of capacity.

1. Each snubber shall be demonstrated OPERABLE by performance of the following inspection program:

a. Visual Inspections

Snubbers are categorized as inaccessible or accessible during reactor operation. Each of the categories (inaccessible and accessible) may be inspected independently according to the schedule determined by Table 4.5-1. The visual inspection interval for each type of snubber shall be determined based upon the criteria provided in Table 4.5-1.

b. Visual Inspection Acceptance Criteria

Visual inspections shall verify that: (1) that there are no visible indications of damage or impaired OPERABILITY; (2) attachments to the foundation or supporting structure are functional; and (3) fasteners for the attachment of the snubber to the component and to the snubber anchorage are functional. Snubbers which appear inoperable as a result of visual inspections shall be classified as unacceptable and may be reclassified as acceptable for the purpose of establishing the next visual inspection interval, providing that: (1) the cause of the rejection is clearly established and remedied for that particular snubber and for other snubbers irrespective of type that may be generically susceptible; and (2) the affected snubber is functionally tested in the as-found condition and determined OPERABLE per Specification 4.5.Q.d or 4.5.Q.e. A review and evaluation shall be performed and documented to justify continued operation with an unacceptable snubber. If continued operation cannot be justified, the snubber shall be declared inoperable and the ACTION requirements shall be met.

c. Functional Tests

At least once every 24 months, a representative sample (10% of the total of each type of snubber in use in the plant) shall be functionally tested either in place or in a bench test. For each snubber that

developed at the Savannah River Laboratory which were described at the Ninth AEC Cleaning Conference.\*

High efficiency particulate filters are installed before and after the charcoal filters to minimize potential releases of particulates to the environment and to prevent clogging of the iodine filters. An efficiency of 99% is adequate to retain particulates that may be released to the reactor building following an accident. This will be demonstrated by testing with DOP at testing medium.

If laboratory tests for the adsorber material in one circuit of the Standby Gas Treatment System are unacceptable, all adsorber material in that circuit shall be replaced with adsorbent qualified according to Regulatory Guide 1.52. Any HEPA filters found defective shall be replaced with those qualified with Regulatory Position C.3.d of Regulatory Guide 1.52.

The snubber inspection frequency is based upon the number of unacceptable snubbers found during the previous inspection, the total population or category size for each snubber type, and the previous inspection interval. A snubber is considered unacceptable if it fails to satisfy the acceptance criteria of the visual inspection. Snubbers may be categorized, based upon their accessibility during power operation, as accessible or inaccessible. These categories may be examined separately or jointly. However, that decision must be made and documented before any inspection and used as the basis upon which to determine the next inspection interval for that category.

If continued operation cannot be justified with an unacceptable snubber, the snubber shall be declared inoperable and the applicable action requirements met. To determine the next surveillance interval, the snubber may be reclassified as acceptable if it can be demonstrated that the snubber is operable in its as-found condition by the performance of a functional test and if it satisfies the acceptance criteria for functional testing.

The next visual inspection interval may be twice, the same, or reduced by as much as two-thirds of the previous inspection interval. This interval depends on the number of unacceptable snubbers found in proportion to the size of the population or category for each type of snubber included in the previous inspection. Table 4.5-1 establishes the length of the next visual inspection interval.

To further increase the assurance of snubber reliability, functional tests should be performed once each refueling cycle. These tests will include stroking of the snubbers to verify proper piston movement, lock-up and bleed. Ten percent represents an adequate sample for such tests. Observed failures of these samples require testing of additional units.

After the containment oxygen concentration has been reduced to meet the specification initially, the containment atmosphere is maintained above atmospheric pressure by the primary containment inerting system. This system supplies nitrogen makeup to the containment so that the very slight leakage from the containment is replaced by nitrogen, further reducing the oxygen concentration. In addition, the oxygen concentration is continuously recorded and high oxygen concentration is annunciated. Therefore, a weekly check of oxygen concentration is adequate. This system also provides the capability for determining if there is gross leakage from the containment.

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\*D.R. Muhàbier, "In Place Nondestructive Leak Test for Iodine Adsorbers," Proceedings of the Ninth AEC Air Cleaning Conference, USAEC Report CONF-660904, 1966

**TABLE 4.5-1**  
**SNUBBER VISUAL INSPECTION INTERVAL**  
Page 1 of 2

Population or Category (Notes 1,2)	Column A Extend Interval (Notes 3,6)	Column B Repeat Interval (Notes 4,6)	Column C Reduce Interval (Notes 5,6)
1	0	0	1
80	0	0	2
100	0	1	4

Note 1: The next visual inspection interval for a snubber population or category size shall be determined based upon the previous inspection interval and the number of unacceptable snubbers found during that interval. Snubbers may be categorized, based upon their accessibility during power operation, as accessible or inaccessible. These categories may be examined separately or jointly. However, the decision on how to categorize the snubbers must be made and documented before any inspection and shall use that decision as the basis upon which to determine the next inspection interval for that category.

Note 2: Interpolation between population or category sizes and the number of unacceptable snubbers is permissible. Use next lower integer for the value of the limit for Columns A, B, or C if that integer includes a fractional value of unacceptable snubbers as determined by interpolation.

Note 3: If the number of unacceptable snubbers is equal to or less than the number in Column A, the next inspection interval may be twice the previous interval but not greater than 48 months.

Note 4: If the number of unacceptable snubbers is equal to or less than the number in Column B but greater than the number in Column A, the next inspection interval shall be the same as the previous interval.

**TABLE 4.5-1**  
**SNUBBER VISUAL INSPECTION INTERVAL**  
Page 2 of 2

- Note 5: If the number of unacceptable snubbers is equal to or greater than the number in Column C, the next inspection interval shall be two-thirds of the previous interval. However, if the number of unacceptable snubbers is less than the number in Column C but greater than the number in Column B, the next interval shall be reduced proportionally by interpolation, that is, the previous interval shall be reduced by a factor that is one-third of the ratio of the difference between the number of unacceptable snubbers found during the previous interval and the number in Column B to the difference in the numbers in Column B and C.
- Note 6: Each inspection interval shall be subject to the limitations of Technical Specification 1.24.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 182

TO FACILITY OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND

JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated June 26, 1995, GPU Nuclear Corporation (GPUN\the licensee) proposed changes to the Technical Specifications (TS) for the Oyster Creek Nuclear Generating Station. The amendment proposes removing the snubber visual examination schedule in the existing TS and replacing it with a refueling outage based visual examination schedule as shown in Table 4.7-2, "Snubber Visual Inspection Interval" of Enclosure B to Generic Letter (GL) 90-09, "Alternate Requirements for Snubber Visual Inspection Interval and Corrective Actions." The proposed change revises the snubber visual inspection intervals to match the schedule developed by the NRC staff for use with a 24-month refueling interval. GPUN also proposed to revise the bases for the snubber visual inspection interval to be consistent with the bases described in Generic Letter 90-09.

2.0 EVALUATION

The snubber visual examination schedule in the existing TS is based on the permissible number of inoperable snubbers found during the visual examination. Because the existing snubber visual examination schedule is based only on the absolute number of inoperable snubbers found during the visual examination irrespective of the total population of snubbers, licensees with a large snubber population find the visual examination schedule excessively restrictive. The purpose of the alternative visual examination schedule is to allow the licensee to perform visual examinations and corrective actions during plant outages without reduction of the confidence level provided by the existing visual examination schedule. The new visual examination schedule specifies the permissible number of inoperable snubbers for various snubber populations. The basic examination interval is the normal fuel cycle up to 24-months. This interval may be extended to as long as twice the fuel cycle or reduced to as small as two-thirds of the fuel cycle depending on the number of unacceptable snubbers found during the visual examination. The examination interval may vary by  $\pm 25$  percent to coincide with the actual outage.

During the recent 15R refueling outage, one snubber failed a scheduled visual inspection. This failure was located on the main steamline. An engineering evaluation was performed by GPUN as per Technical Specifications which determined that no damage had occurred on any snubber. This snubber was in service since 1977.

There were thirteen additional snubbers on the main steamline. Eleven of these were replaced and two were tested satisfactorily and reinstalled (one was originally installed in 1988 and the other in 1993).

The sample size for mechanical snubber visual inspections was 100% as required by technical specifications. The sample size for mechanical snubber functional inspections was increased from 10% to 42% since one functional failure was found on the other main steamline and subsequently was replaced. This snubber was in service since 1977.

It was determined that the causes of the snubber failures were sustained high temperatures and high frequency vibration for an extended length of service. The high temperature caused the snubber grease to degrade, whereupon the extended high frequency vibration caused excessive wear. The snubbers had been in service since 1977.

The existing Technical Specification would require a reactor shutdown and drywell entry one year into the operating cycle solely for the purpose of performing an inspection on the snubbers which were replaced or reinstalled on the main steam system in 15R. The purpose of this change request is to amend the technical specifications to not require the reactor shutdown, and update the Technical Specification requirements to those previously approved in Generic Letter 90-09.

The exact wording of GL 90-09 has been utilized by GPUN to the greatest extent practical. However, minor changes have been requested to allow for the design specifics of the Oyster Creek Plant. Each change from the prescribed wording in GL 90-09 is discussed and evaluated separately.

GPUN proposes the following plant specific wording changes:

Section 4.5.Q.1 GL 90-09 wording: "...performance of the following augmented inservice inspection program in addition to the requirements of 4.0.5."

Technical specification change request (TSCR) wording:  
"...performance of the following inspection program."

Reason for the change: Oyster Creek controls the inspection and testing of the snubbers in the Technical Specifications and not in the Augmented Inservice Testing Program. Further, Section 4.0.5 referenced in the Generic Letter states in Section e "Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification."

The staff agrees with GPUN that since Oyster Creek controls the inspection and testing of the snubbers in the TS and not in the Augmented Inservice Testing Program the wording change is appropriate.

Section 4.5.Q.1.a GL 90-09 wording: "based on the criteria of Table 4.7.2 and the first inspection interval determined using the criteria shall be based upon the previous inspection interval established by the requirements in effect before Amendment (\*)"

TSCR wording: "...based on the criteria provided in Table 4.5-1."

Reason for the Change: 1) Table 4.7.2 in the Generic Letter is Table 4.5-1 in the TSCR, 2) Although there was a single visual failure during the last interval, all snubbers in the same temperature and vibration environment were either replaced or tested satisfactorily and reinstalled. There is no need to perform a plant shutdown for the sole purpose of inspecting snubbers which have seen one year of service when the single failed snubber had been in service for seventeen years.

The replacement/reinstallation of all snubbers in a similar application (main steam system) has effectively removed the failure mechanism for the single visual inspection failure that was observed last outage. Additionally, the replacement/reinstallation of all the snubbers in similar applications (main steam system) has significantly decreased the probability of occurrence and consequences of any accident previously evaluated as all snubbers in this application have been functionally tested during the last surveillance interval. Therefore, the one time increase in interval from the existing 12 months to 24 months is within the inspection interval which would have been in effect for the majority of the snubbers had the single failure not occurred.

The staff agrees with GPUN that since all snubbers were either replaced or tested satisfactorily and reinstalled there is no need to perform a plant shutdown for the reasons stated above.

Section 4.5.Q.1.b GL 90-09 wording: "...All snubbers found connected to an inoperable common hydraulic fluid reservoir shall be counted as unacceptable for determining the next inspection interval.

TSCR wording: --Sentence was deleted--

GPUN has stated that Oyster Creek does not have any snubbers sharing a common reservoir.

The staff finds this change acceptable but notes that if Oyster Creek Nuclear Generating Station implements this type of system, GPUN must submit the appropriate changes.

GPUN has proposed changes to TS 4.5.Q.a and the associated Bases that are consistent with the guidance provided in GL 90-09 for the replacements of the visual examination schedule with Table 4.7-2 (including footnotes 1 through 6) of the Generic Letter 90-09. On the basis of its review of this matter, the staff finds that the proposed changes to the TS for the Oyster Creek Nuclear Generating Station are acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 39440). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Alexander W. Dromerick

Date: September 6, 1995