



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

June 7, 1993

Docket No. 50-219

Mr. John J. Barton  
Vice President and Director  
GPU Nuclear Corporation  
Oyster Creek Nuclear Generating Station  
Post Office Box 388  
Forked River, New Jersey 08731

Dear Mr. Barton:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M85931)

The Commission has issued the enclosed Amendment No.164 to Facility Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station, in response to your application dated March 3, 1993.

The amendment reduces the setpoint on the ninth (highest) safety valve from 1230 to 1221 psig. Also included in the application was a licensing basis analysis in support of the Technical Specification (TS) change request.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Alexander W. Dromerick, Senior Project Manager  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 164 to DPR-16
2. Safety Evaluation

cc w/enclosures:  
See next page

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Mr. John J. Barton  
GPU Nuclear Corporation

Oyster Creek Nuclear  
Generating Station

cc:

Ernest L. Blake, Jr., Esquire  
Shaw, Pittman, Potts & Trowbridge  
2300 N Street, NW.  
Washington, DC 20037

Resident Inspector  
c/o U.S. Nuclear Regulatory Commission  
Post Office Box 445  
Forked River, New Jersey 08731

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, Pennsylvania 19406

Kent Tosch, Chief  
New Jersey Department of  
Environmental Protection  
Bureau of Nuclear Engineering  
CN 415  
Trenton, New Jersey 08625

BWR Licensing Manager  
GPU Nuclear Corporation  
1 Upper Pond Road  
Parsippany, New Jersey 07054

Mayor  
Lacey Township  
818 West Lacey Road  
Forked River, New Jersey 08731

Licensing Manager  
Oyster Creek Nuclear Generating Station  
Mail Stop: Site Emergency Bldg.  
Post Office Box 388  
Forked River, New Jersey 08731

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Original signed by

Alexander W. Dromerick, Senior Project Manager  
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Division of Reactor Projects - I/II  
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See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20556-0001

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 164  
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by GPU Nuclear Corporation, et al., (the licensee), dated March 3, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 164, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented during the 15R refueling outage.

FOR THE NUCLEAR REGULATORY COMMISSION

*Ronald W. Hornan for J Stolz*

John F. Stolz, Director  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: June 7, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 164

FACILITY OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

2.3-2

4.3-1

Insert

2.3-2

4.3-1

FUNCTIONLIMITING SAFETY SYSTEM SETTINGSB. Neutron Flux,  
Control Rod Block

The Rod Block setting shall be

$$S \leq [(0.90 \times 10^{-6}) W + 53.1] \left[ \frac{\text{FRP}}{\text{MFLPD}} \right]$$

with a maximum setpoint of 108% for core flow equal to  $61 \times 10^6$  lb/hr and greater.

The definitions of S, W, FRP and MFLPD used above for the APRM scram trip apply.

The ratio of FRP to MFLPD shall be set equal to 1.0 unless the actual operating value is less than 1.0, in which case the actual operating value will be used.

This adjustment may be accomplished by increasing the APRM gain and thus reducing the flow referenced APRM rod block curve by the reciprocal of the APRM gain change.

C.	Reactor High, Pressure, Scram	$\leq 1060$ psig	
D.	Reactor High Pressure, Relief Valves Initiation	2 @ $\leq 1070$ psig 3 @ $\leq 1090$ psig	
E.	Reactor High Pressure, Isolation Condenser Initiation	$\leq 1060$ psig with time delay $\leq 3$ seconds	
F.	Reactor High Pressure, Safety Valve Initiation	4 @ 1212 psig $\pm 12$ psi 5 @ 1221 psig $\pm 12$ psi	
G.	Low Pressure Main Steam MSIV Closure	$\geq 825$ psig (initiated in IRM Line, range 10)	
H.	Main Steam Line Isolation Valve Closure, Scram	$\leq 10\%$ Valve Closure from full open	
I.	Reactor Low Water Level, Scram	$\geq 11'5"$ above the top of the active fuel as indicated under normal operating conditions	
J.	Reactor Low-Low Water Level, Main Steam Line Isolation Valve Closure	$\geq 7'2"$ above the top of the active fuel as indicated under normal operating conditions	

#### 4.3 REACTOR COOLANT

Applicability:

Applies to the surveillance requirements for the reactor coolant system.

Objective:

To determine the condition of the reactor coolant system and the operation of the safety devices related to it.

Specification:

- A. Materials surveillance specimens and neutron flux monitors shall be installed in the reactor vessel adjacent to the wall at the midplane of the active core. Specimens and monitors shall be periodically removed, tested, and evaluated to determine the effects of neutron fluence on the fracture toughness of the vessel shell materials. The results of these evaluations shall be used to assess the adequacy of the P-T curves (a),(b) and (c) in Figure 3.3.1. New curves shall be generated as required.
- B. Inservice inspection of ASME Code Class 1, Class 2 and Class 3 systems and components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10 CFR, Section 50.55a(g)(6)(i).
- C. Inservice testing of ASME Code Class 1, Class 2 and Class 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10 CFR, Section 50.55a(g)(6)(i).
- D. A visual examination for leaks shall be made with the reactor coolant system at pressure during each scheduled refueling outage or after major repairs have been made to the reactor coolant system in accordance with Article 5000, Section XI. The requirements of specification 3.3.A shall be met during the test.
- E. Each replacement safety valve or valve that has been repaired shall be tested in accordance with subsection IWV-3510 of Section XI of the ASME Boiler and Pressure Vessel Code. Setpoints shall be as follows:

<u>Number of Valves</u>	<u>Set Points (psig)</u>
4	1212 ± 12
5	1221 ± 12

- F. A sample of reactor coolant shall be analyzed at least every 72 hours for the purpose of determining the content of chloride ion and to check the conductivity.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 164

TO FACILITY OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND  
JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated March 3, 1993, the GPU Nuclear Corporation, the licensee for the Oyster Creek Nuclear Generating Station, submitted a request to reduce the setpoint on the ninth (highest) safety valve from 1230 to 1221 psig. The licensee also included a licensing basis analysis in support of the Technical Specification (TS) change request.

2.0 EVALUATION

The reduction in setpoint of the ninth safety valve from 1230 to 1221 psig is based on licensing basis analysis presented in the amendment request. The safety valves are not actuated during normal operations. The system satisfies the overpressure protection requirement of the ASME Code. However, Oyster Creek has not had transients in which a safety valve has been actuated. A plant performance evaluation showed a low probability of a safety valve opening due to an anticipated transient.

The most limiting overpressure event that was analyzed in the Final Safety Analysis Report (FSAR) was re-analyzed, and the results were presented in the amendment request. The event was the main steam isolation valve closure with scram on high neutron flux and failure of electromatic relief valves, turbine bypass valves, isolation condensers and recirculation pump trip. In the re-analysis, the single (ninth) safety valve was assigned an analysis setpoint of 1236 psig which was the same setpoint used for the four safety valves that are set at 1221 psig. The remaining four safety valves were assigned an analysis setpoint of 1227 psig and are set at 1212 psig. The resulting peak pressure at the bottom of the reactor vessel was 1368 psig which is below the ASME Code limit of 1375 psig (110% of design pressure). This is lower than the peak pressure of 1369 psig that was the result in the original FSAR analysis. The recirculation piping had a peak pressure of 1376.5 psig which is also below the Code limit of 1380 psig (115% of design pressure).

Since the event acceptance limits have been satisfied with peak pressures that are below the Code limits, the analysis is acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 16861). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: A. Wilford

Date: June 7, 1993