

March 9, 1992

Docket No. 50-219

Mr. John J. Barton
Vice President and Director
GPU Nuclear Corporation
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

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Dear Mr. Barton:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M64381)

The Commission has issued the enclosed Amendment No. 157 to Facility Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station, in response to your application dated October 23, 1991, as supplemented December 20, 1991.

The amendment revises Section 3.13 of the Technical Specifications to have available a preplanned alternate method capable of being implemented to provide an estimate of the radioactive material in containment under accident conditions if both containment high range monitors are inoperable for more than 7 days.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Alexander W. Dromerick, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 157 to DPR-16
- 2. Safety Evaluation

cc w/enclosures:
See next page

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Document Name: 64381AMD

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*Notice issued
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Mr. John J. Barton
GPU Nuclear Corporation

Oyster Creek Nuclear
Generating Station

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 157
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear Corporation, et al., (the licensee), dated October 23, 1991, as supplemented December 20, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 157, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 9, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 157

FACILITY OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
3.13-3	3.13-3
3.13-4	3.13-4

2. With the number of operable channels less than the total number of channels shown in Table 3.13.1, restore the inoperable channel to operable status within 30 days or place the reactor in the shutdown condition within the next 24 hours.
3. With the number of operable channels less than the minimum channels operable requirements of Table 3.13.1, restore at least one channel to operable status within 7 days or place the reactor in the shutdown condition within the next 24 hours.

G. Containment High-Range Radiation Monitor

1. Two containment high-range radiation monitors shall be OPERABLE when PRIMARY CONTAINMENT INTEGRITY is required.
2. With the number of OPERABLE monitors less than 2:
 - a. Take appropriate action to restore the inoperable monitor(s) to OPERABLE status as soon as possible.
 - b. Perform any actions required by Table 3.1.1.
 - c. Restore the inoperable monitor(s) to OPERABLE status within 7 days of the failure or prepare and submit a Special Report within 14 days following the failure outlining the cause of inoperability, actions taken, and the planned schedule for restoring the monitors to OPERABLE status.
3. With the number of OPERABLE monitors less than 1, in addition to the actions of 3.13.G.2 above, restore at least 1 monitor to OPERABLE status within 7 days of the failure or have available a preplanned alternate method capable of being implemented to provide an estimate of the radioactive material in containment under accident conditions.

H. High-Range Radioactive Noble Gas Effluent Monitor

1. The high range radioactive noble gas effluent monitors listed in Table 3.13.1 shall be OPERABLE during POWER OPERATION.
2. With the number of OPERABLE channels less than required by the minimum channels OPERABLE requirements, restore the inoperable channel(s) to OPERABLE status within 7 days of the event or prepare and submit a Special Report within 30 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the equipment to OPERABLE status.

BASES

The purpose of the safety/relief valve accident monitoring instrumentation is to alert the operator to a stuck open safety/relief valve which could result in an inventory threatening event.

As the safety valves present distinctly different concerns than those related to relief valves, the technical specifications are separated as to the actions taken upon inoperability. Clearly, the actuation of a safety valve will be immediately detectable by observed increase in drywell pressure. Further confirmation can be gained by observing reactor pressure and water level. Operator action in response to these symptoms would be taken regardless of the acoustic monitoring system status. Acoustic monitors act only to confirm the reseating of the safety valve. In actuality, the operator actions in response to the lifting of a safety valve will not change whether or not the safety valve reseats. Therefore, the actions taken for inoperable acoustic monitors on safety valves are significantly less stringent than that taken for those monitors associated with relief valves.

Should an acoustic monitor on a safety valve become inoperable, setpoints on adjacent monitors will be reduced to assure alarm actuation should the safety valve lift, since it is of no importance to the operator as to which valves lift but only that one has lifted. Analyses, using very conservative blowdown forces and attenuation factors, show that reducing the alarm setpoint on adjacent monitors to less than 1.4g will assure alarm actuation should the adjacent safety valve lift. Minimum blowdown force considered was 30g with a maximum attenuation of 27dB. In actuality, a safety valve lift would result in considerably larger blowdown force. The maximum attenuation of 27dB was determined based on actual testing of a similar monitoring system installed in a similar configuration.

The operability of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. The capability is consistent with NUREGs 0578 and 0737.

The capability is provided to detect and measure concentrations of noble gas fission products in (1) plant gaseous effluents and (2) in containment during and following an accident. For the plant gaseous effluent capability, two Radioactive Gaseous Effluent Monitoring Systems (RAGEMS) are installed at Oyster Creek. One system monitors releases at the main stack (RAGEMS I) and the other monitors the turbine building vents (RAGEMS II). For the in containment post-accident capability, two high range radiation monitors are installed in the drywell. These monitors augment the capabilities provided by the Post Accident Sampling System (Technical Specification 6.17 and FSAR Section 11.5) and the Offsite Thermoluminescent Dosimeter Program (Emergency Plan Section 7.5.2.2b). The Post Accident Sampling System represents a preplanned alternate method to the high range radiation monitors capable of being implemented to provide an estimate of radioactive material in containment under accident conditions.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 157

TO FACILITY OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND
JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated June 24, 1991, the NRC denied a request by the licensee, GPU Nuclear Corporation (GPUN), that would have obviated the requirement to have a Technical Specification (TS) consistent with NUREG-0123, Revision 4. The applicable TS required a preplanned alternate method of monitoring the containment radiation if both containment high range area radiation monitors are inoperable for more than 7 days.

GPUN responded in a letter dated October 23, 1991, with Technical Specification Change Request (TSCR) No. 204 which proposed to have available a preplanned alternate method consistent with NUREG-0123, Revision 4. Additionally, the licensee submitted a revised BASES section in a letter dated December 20, 1991, which superseded the one in their October 23, 1991 letter. The TSCR complies with the NRC approved Amendment No. 116 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. The licensee's request will modify TS Section 3.13 Item G, Containment High-Range Radiation Monitor and its associated BASES section by declaring the post accident sampling system (PASS) as the preplanned alternate for measuring the containment radiation in the event the containment high-range area radiation monitors are out of service for more than 7 days.

2.0 EVALUATION

The change requested by GPUN would add a statement to TS Section 3.13 Item G, Containment High-Range Radiation Monitor, which requires, with the number of operable monitors less than 1 for more than 7 days, the availability of a "preplanned alternate method capable of being implemented to provide an estimate of the radioactive material in containment under accident conditions."

The licensee's proposed use of the PASS as the preplanned alternate monitoring method is considered technically acceptable. The PASS is required by TS 6.17 to have the capability to obtain and analyze containment atmosphere samples

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(as well as from other key systems) under accident conditions. It includes requirements for: procedures for sampling and analysis; training of personnel in sampling and analysis; and provisions for verifying operability of the systems. This system is considered adequate to provide information on containment radiation levels in lieu of the containment high-range area radiation monitors for use in assessing the magnitude of an accident and for use in determining offsite protective action recommendations.

A review of NUREG-1435, Volume 1, Status of Safety Issues at Licensed Power Plants, documents that the licensee has implemented the requirements of NUREG-0737, Item II.B.3, Post Accident Sampling Capability.

Based on the above, the staff finds that, consistent with NUREG-0123, Revision 4, the licensee has proposed an adequate preplanned alternate method of monitoring the containment radiation if both containment high-range area radiation monitors are inoperable for more than 7 days. Consequently, the staff recommends that TSCR No. 204 be approved.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 4487). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Klementowicz

Date: March 9, 1992