Tennessee Valley Authority, Post Office Box 2000, Soddy-Daisy, Tennessee 37384-2000

April 26, 2001

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Gentlemen:

In the Matter of	)	Docket Nos. 50-327
Tennessee Valley Authority	)	50-328

SEQUOYAH NUCLEAR PLANT - ANNUAL RADIOLOGICAL ENVIRONMENTAL OPERATING REPORT - 2000

Enclosed is the subject report for the period of January 1 to December 31, 2000. This report is being submitted in accordance with Sequoyah Technical Specification 6.9.1.6 and Sequoyah's Offsite Dose Calculation Manual Administrative Control Section 5.1.

Please direct questions concerning this issue to me at (423) 843-7170 or J. D. Smith at (423) 843-6672.

Sincerely,

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Licensing and Industry Affairs Manager

Enclosure
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Cool

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Annual Radiological Environmental Operating Report

Sequoyah
Nuclear Plant
2000

ANNUAL RADIOLOGICAL ENVIRONMENTAL OPERATING REPORT
SEQUOYAH NUCLEAR PLANT
2000

TENNESSEE VALLEY AUTHORITY

April 2001

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EXECUTIVE SUMMARY

This report describes the radiological environmental monitoring program conducted by TVA in the vicinity of the Sequoyah Nuclear Plant (SQN) in 2000. The program includes the collection of samples from the environment and the determination of the concentrations of radioactive materials in the samples. Samples were collected from locations in the general area of the plant and from areas not influenced by plant operations. Monitoring includes the sampling of air, water, milk, foods, vegetation, soil, fish, clams, sediment and the measurement of direct radiation levels. Results from stations near the plant are compared with data from control stations and with preoperational measurements to determine potential impacts of plant operations.

The vast majority of the radioactivity measured in environmental samples from the SQN program was contributed by naturally occurring radioactive materials or by radionuclides found in the environment as a result of fallout from past atmospheric nuclear weapons testing. Trace quantities of Sr-90 were detected in samples of milk and vegetation and low levels of Cs-137 were measured in soil, sediment and fish. The concentrations of these radionuclides were typical of the levels present in the environment from past nuclear weapons testing or operation of other nuclear facilities in the region. In addition, a small amount of Co-60 was measurable in samples of bottom sediment and low levels of tritium were found in ground water from the on site monitoring well. These levels would not represent a significant contribution above natural backgrounds to the radiation exposure to Members of the Public.

INTRODUCTION

This report describes and summarizes the results of radioactivity measurements made in the vicinity of SQN and laboratory analyses of samples collected in the area. The measurements are made to comply with the requirements of 10 CFR 50, Appendix A, Criterion 64 and 10 CFR 50, Appendix I, Sections IV.B.2, IV.B.3 and IV.C and to determine potential effects on public health and safety. This report satisfies the annual reporting requirements of SQN Technical Specification 6.9.1.6 and Offsite Dose Calculation Manual (ODCM) Administrative Control 5.1. In addition, estimates of the maximum potential doses to the surrounding population are made from radioactivity measured both in plant effluents and in environmental samples. The data presented in this report include results from the prescribed program and other information to help correlate the significance of results measured by this monitoring program to the levels of environmental radiation resulting from naturally occurring radioactive materials.

Naturally Occurring and Background Radioactivity

Many materials in our world contain trace amounts of naturally occurring radioactivity. Approximately 0.01 percent of all potassium is radioactive potassium-40 (K-40). K-40, with a half-life of 1.3 billion years, is one of the major types of radioactive materials found naturally in our environment. An individual weighing 150 pounds contains about 140 grams of potassium (Reference 1). This is equivalent to approximately 100,000 pCi of K-40 which delivers a dose of 15 to 20 mrem per year to the bone and soft tissue of the body. Naturally occurring radioactive materials have always been in our environment. Other examples of naturally occurring radioactive materials are beryllium (Be)-7, bismuth (Bi)-212 and 214, lead (Pb)-212 and 214, thallium (TI)-208, actinium (Ac)-228, uranium (U)-238 and 235, thorium (Th)-234, radium (Ra)-226, radon (Rn)-222, carbon (C)-14, and hydrogen (H)-3 (generally called tritium). These naturally occurring radioactive materials are in the soil, our food, our drinking water, and our bodies. The radiation from these materials makes up a part of the low level natural background

radiation. The remainder of the natural background comes from cosmic ray radiation from outer space. We are all exposed to this natural radiation 24 hours per day. It is possible to get an idea of the relative hazard of different types of radiation sources by evaluating the amount of radiation the U.S. population receives from each general type of radiation source. The information in the following table is primarily adapted from References 2 and 3.

U.S. GENERAL POPULATION AVERAGE DOSE EQUIVALENT ESTIMATES

Source	Millirem/Year Per Person
Natural background dose equivalent	
Cosmic	27
Cosmogenic	1
Terrestrial	28
In the body	39
Radon	200
Total	295
Release of radioactive material in natural gas, mining, ore processing, etc.	5
Medical (effective dose equivalent)	53
Nuclear weapons fallout	less than 1
Nuclear energy	0.28
Consumer products	0.03
Total	355 (approximately)

As can be seen from the table, natural background radiation dose equivalent to the U.S. population normally exceeds that from nuclear plants by several hundred times. This indicates that nuclear plant operations normally result in a population radiation dose equivalent which is insignificant compared to that which results from natural background radiation.

Electric Power Production

Nuclear power plants are similar in many respects to conventional coal burning (or other fossil fuel) electric generating plants. The basic process behind electrical power production in both types of plants is that fuel is used to heat water to produce steam which provides the force to turn turbines and generators. In a nuclear power plant, the fuel is uranium and the heat is produced in the reactor through the fission of the uranium. Nuclear plants include many complex systems to control the nuclear fission process and to safeguard against the possibility of reactor malfunction. The nuclear reactions produce radionuclides commonly referred to as fission and activation products. Very small amounts of these fission and activation products are released into the plant systems. This radioactive material can be transported throughout plant systems and some of it released to the environment.

The pathways through which radioactivity is released are monitored. Liquid and gaseous effluent monitors record the radiation levels for each release. These monitors also provide alarm mechanisms to prompt termination of any release above limits.

Releases are monitored at the onsite points of release and through the environmental monitoring program which measures the environmental radiation in outlying areas around the plant. In this way, not only is the release of radioactive materials from the plant tightly controlled, but measurements are made in surrounding areas to verify that the population is not being exposed to significant levels of radiation or radioactive materials.

The SQN ODCM, which is required by the plant Technical Specifications, prescribes limits for the release of radioactive effluents, as well as limits for doses to the general public from the release of these effluents.

The dose to a member of the general public from radioactive materials released to unrestricted areas, as given in NRC guidelines and the ODCM, is limited as follows:

Liquid Effluents

Total body	≤3 mrem/year
Any organ	≤10 mrem/year

Gaseous Effluents

Noble gases:

Gamma radiation	≤10 mrad/year
Beta radiation	≤20 mrad/year

Particulates:

Any organ	≤15 mrem/year

The EPA limits for the total dose to the public in the vicinity of a nuclear power plant, established in the Environmental Dose Standard of 40 CFR 190, are as follows:

Total body	≤25 mrem/year
Thyroid	≤75 mrem/year
Any other organ	≤25 mrem/year

Appendix B to 10 CFR 20 presents annual average limits for the concentrations of radioactive materials released in gaseous and liquid effluents at the boundary of the unrestricted areas. Table 1 of this report compares the nominal lower limits of detection for the SQN monitoring program with the regulatory limits for maximum annual average effluent concentrations released to unrestricted areas and levels requiring special reports to the NRC. It should be noted that the levels of radioactive materials measured in the environment are typically only slightly above the lower limit of detection. The data presented in this report indicate compliance with the regulation.

SITE/PLANT DESCRIPTION

Sequoyah is located on a site near the geographical center of Hamilton County, Tennessee, on a peninsula on the western shore of Chickamauga Lake at Tennessee River Mile (TRM) 484.5. Figure 1 shows the site in relation to other TVA projects. The SQN site, containing approximately 525 acres, is approximately 7.5 miles northeast of the nearest city limit of Chattanooga, Tennessee, 14 miles west-northwest of Cleveland, Tennessee, and approximately 31 miles south-southwest of TVA's Watts Bar Nuclear Plant (WBN) site.

Population is distributed rather unevenly within 10 miles of the SQN site. Approximately 60 percent of the population is in the general area between 5 and 10 miles from the plant in the sectors ranging from the south, clockwise, to the northwest sector. This concentration is a reflection of suburban Chattanooga and the town of Soddy-Daisy. This area is characterized by considerable vacant land with scattered residential subdivisions. The northern most extent of the urbanization around Chattanooga is approximately 4 miles from the site.

Residential subdivision growth has continued within a 10-mile radius of the plant. There is also some small-scale farming and at least two dairy farms are located within 5 miles of the plant.

Chickamauga Reservoir is one of a series of highly controlled multiple-use reservoirs whose primary uses are flood control, navigation, and the generation of electric power. Secondary uses include industrial and public water supply and waste disposal, commercial fishing, and recreation. Public access areas, boat docks, and residential subdivisions have been developed along the reservoir shoreline.

SQN consists of two pressurized water reactors: each unit is rated at 1183 megawatts (electrical). Fuel was loaded in Unit 1 on March 1, 1980, and the unit achieved critically on July 5, 1980. Fuel was loaded in Unit 2 in July 1981, and the unit achieved initial criticality on November 5, 1981.

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

Most of the radiation and radioactivity generated in a nuclear power reactor is contained within the reactor itself or one of the other plant systems. Plant effluent monitors are designed to detect the small amounts of radioactive material released to the environment. Environmental monitoring is a final verification that the systems are performing as planned. The monitoring program is designed to check the pathways between the plant and the people in the immediate vicinity and to most efficiently monitor these pathways. Sample types are chosen so that the potential for detection of radioactivity in the environment will be maximized. The radiological environmental monitoring program is outlined in Appendix A.

There are two primary pathways by which radioactivity can move through the environment to humans: air and water (see Figure 2). The air pathway can be separated into two components: the direct (airborne) pathway and the indirect (ground or terrestrial) pathway. The direct airborne pathway consists of direct radiation and inhalation by humans. In the terrestrial pathway, radioactive materials may be deposited on the ground or on plants and subsequently be ingested by animals and/or humans. Human exposure through the liquid pathway may result from drinking water, eating fish, or by direct exposure at the shoreline. The types of samples collected in this program are designed to monitor these pathways.

A number of factors were considered in determining the locations for collecting environmental samples. The locations for the atmospheric monitoring stations were determined from a critical pathway analysis based on weather patterns, dose projections, population distribution, and land use. Terrestrial sampling stations were selected after reviewing such factors as the locations of dairy animals and gardens in conjunction with the air pathway analysis. Liquid pathway stations were selected based on dose projections, water use information, and availability of media such as fish and sediment. Table A-2 (Appendix A, Table 2: This identification system is used for all tables and figures in the appendices.) lists the sampling stations and the types of samples collected. Modifications made to the program in 2000 are described in Appendix B and exceptions to the sampling and analysis schedule are presented in Appendix C.

To determine the amount of radioactivity in the environment prior to the operation of SQN, a preoperational radiological environmental monitoring program was initiated in 1971 and operated until the plant began operation in 1980. Measurements of the same types of radioactive materials that are measured currently were assessed during the preoperational phase to establish normal background levels for various radionuclides in the environment.

The preoperational monitoring program is a very important part of the overall program. The knowledge of pre-existing radionuclide patterns in the environment permits a determination, through comparison and trending analyses, of whether the operation of SQN is impacting the environment and thus the surrounding population.

The determination of impact from the plant during the operating phase also utilizes the data from control stations that have been established in the monitoring program. Results of environmental samples taken at control stations (far from the plant) are compared with those from indicator stations (near the plant) to establish the extent of SQN influence.

Samples are analyzed by the Radioanalytical Laboratory of TVA's Environmental Radiological Monitoring and Instrumentation group located at the Western Area Radiological Laboratory (WARL) in Muscle Shoals, Alabama. Analyses are conducted in accordance with written and approved procedures and are based on accepted methods. A summary of the analysis techniques and methodology is presented in Appendix D. Data tables summarizing the sample analysis results are presented in Appendix H.

The radiation detection devices and analysis methods used to determine the radionuclide content of samples collected in the environment are very sensitive to small amounts of radioactivity. The sensitivity of the measurements process is defined in terms of the lower limit of detection (LLD). A description of the nominal LLDs for the Radioanalytical Laboratory is presented in Appendix E.

The Radioanalytical Laboratory employs a comprehensive quality assurance/quality control program to monitor laboratory performance throughout the year. The program is intended to detect any problems in the measurement process as soon as possible so they can be corrected. This program includes equipment checks to ensure that the radiation detection instruments are working properly and the analysis of quality control samples which are included alongside routine environmental samples. In 2000, the laboratory participated in a blind cross check program administrated by a vendor. In addition, samples split with the State of Tennessee provide an independent verification of the overall performance of the laboratory. A complete description of the program is presented in Appendix F.

DIRECT RADIATION MONITORING

Direct radiation levels are measured at a number of stations around the plant site. These measurements include contributions from cosmic radiation, radioactivity in the ground, fallout from atmospheric nuclear weapons tests conducted in the past, and any radioactivity that may be present as a result of plant operations. Because of the relatively large variations in background radiation as compared to the small levels from the plant, contributions from the plant may be difficult to distinguish.

Radiation levels measured in the area around the SQN site in 2000 were consistent with levels from previous years and with levels measured at other locations in the region.

Measurement Techniques

Direct radiation measurements are made with thermoluminescent dosimeters (TLDs). When certain materials are exposed to ionizing radiation, many of the electrons which become displaced are trapped in the crystalline structure of the material. They remain trapped for long periods of time as long as the material is not heated. When heated (thermo), the electrons are released, producing a pulse of light (luminescence). The intensity of the light pulse is proportional to the amount of radiation to which the material was exposed. Materials which display these characteristics are used in the manufacture of TLDs.

The Panasonic UD-814 dosimeter is used in the radiological environmental monitoring program for the measurement of direct radiation. This dosimeter contains four elements consisting of one lithium borate and three calcium sulfate phosphors. The calcium sulfate phosphors are shielded by approximately 1000 mg/cm^2 plastic and lead to compensate for the over-response of the detector to low energy radiation.

TLDs are placed approximately 1 meter above the ground, with two or more TLDs at each monitoring location. Sixteen monitoring points are located around the plant near the site

boundary, one location in each of the 16 compass sectors. One monitoring point is also located in each of the 16 compass sectors at a distance of approximately four to five miles from the plant. Dosimeters are also placed at the perimeter and remote air monitoring sites and at 13 additional monitoring locations out to approximately 32 miles from the site. The TLDs are exchanged every 3 months and the accumulated exposure on the detectors is read with a Panasonic Model UD-710A automatic reader interfaced with a computer system.

Since the calcium sulfate phosphor is much more sensitive than the lithium borate, the measured exposure is taken as the median of the results obtained from the calcium sulfate phosphors in the dosimeter badge. The values are corrected for gamma response, system variations, and transit exposure, with individual gamma response calibrations for each element. The system meets or exceeds the performance specifications outlined in Regulatory Guide 4.13 for environmental applications of TLDs.

Since 1974, TVA has participated in intercomparisons of environmental dosimeters conducted by the U.S. Department of Energy and other interested parties. The last intercomparison results from this study are from 1996. The results, shown in Table 2, demonstrate that direct radiation levels determined by TVA are generally within ten percent of the calculated or known values.

Results

Results are normalized to a standard quarter (91.25 days or 2190 hours). The monitoring locations are grouped according to the distance from the plant. The first group consists of all monitoring points within 1 mile of the plant. The second group lies between 1 and 2 miles, the third group between 2 and 4 miles, the fourth between 4 and 6 miles, and the fifth group is made up of all locations greater than 6 miles from the plant. Past data have shown that the average results from the locations more than 2 miles from the plant are essentially the same. Therefore, for purposes of this report, monitoring points 2 miles or less from the plant are identified as "onsite" stations and locations greater than 2 miles are considered "offsite."

Prior to 1976, direct radiation measurements in the environment were made with dosimeters that were not as precise at lower exposures. Consequently, environmental radiation levels reported in the early years of the preoperational phase of the SQN monitoring program exceed current measurements of background radiation levels. For this reason, data collected prior to 1976 are not utilized for the comparison to preoperational results.

The quarterly gamma radiation levels determined from the TLDs deployed around SQN in 2000 are summarized in Table H-1. The results from all measurements at individual stations are presented in Table H-2. The exposures are measured in milliroentgens (mR). For purposes of this report, one milliroentgen, one millirem (mrem) and one millirad (mrad) are assumed to be numerically equivalent. The rounded average annual exposures, as measured in 2000, are shown below. For comparison purposes, the average direct radiation measurements made in the preoperational phase of the monitoring program are also shown.

	Annual Average Direct Radiation Levels SQN mR/Year	
	2000	1976-79
Onsite Stations	59	79
Offsite Stations	54	63

The data in Table H-1 indicate that the average quarterly direct radiation levels at the SQN onsite stations are approximately 1.3 mR/quarter higher than levels at the offsite stations. This difference is consistent with levels measured for the preoperation and construction phases of TVA nuclear power plant sites where the average levels onsite were generally 2-6 mR/quarter higher than levels offsite. The causes of these differences have not been isolated; however, it is postulated that the differences are attributable to combinations of influences such as natural variations in environmental radiation levels, earth-moving activities onsite, and the mass of concrete employed in the construction of the plant. Other undetermined influences may also play a part.

Figure H-1 compares plots of the data from the onsite or site boundary stations with those from the offsite stations over the period from 1976 through 2000.

The results reported in 2000 are consistent with direct radiation levels identified at locations which are not influenced by the operation of SQN. There is no indication that SQN activities increased the background radiation levels normally observed in the areas surrounding the plant.

ATMOSPHERIC MONITORING

The atmospheric monitoring network is divided into three groups identified as local, perimeter, and remote. Four local air monitoring stations are located on or adjacent to the plant site in the general directions of greatest wind frequency. Four perimeter air monitoring stations are located in communities out to about 10 miles from the plant, and four remote air monitors are located out to approximately 20 miles. The monitoring program and the locations of monitoring stations are identified in the tables and figures of Appendix A. The remote stations are used as control or baseline stations.

Sample Collection and Analysis

Air particulates are collected by continuously sampling air at a flow rate of approximately 2 cubic feet per minute (cfm) through a 2-inch glass fiber filter. The sampling system consists of a pump, magnehelic gauge for measuring the drop in pressure across the system, and a dry gas meter. This allows an accurate determination of the volume of air passing through the filter. This sampling system is housed in a metal building. The filter is contained in a sampling head mounted on the outside of the monitor building. The filter is replaced weekly. Each filter is analyzed for gross beta activity about 3 days after collection to allow time for the radon daughters to decay. Every 4 weeks composites of the filters from each location are analyzed by gamma spectroscopy.

Gaseous radioiodine is sampled using a commercially available cartridge containing TEDA impregnated charcoal. This system is designed to collect iodine in both the elemental form and as organic compounds. The cartridge is located in the same sampling head as the air particulate filter and is downstream of the particulate filter. The cartridge is changed at the same time as the particulate filter and samples the same volume of air. Each cartridge is analyzed for I-131 by gamma spectroscopy analysis.

Rainwater is sampled by use of a collection tray attached to the monitor building. The collection tray is protected from debris by a screen cover. As water drains from the tray, it is collected in one of two 5-gallon containers inside the monitor building. A 1-gallon sample is removed from the container every 4 weeks. Any excess water is discarded. Rainwater samples are held to be analyzed only if the air particulate samples indicate the presence of elevated activity levels or if fallout is expected. For example, rainwater samples were analyzed during the period of fallout following the accident at Chernobyl in 1986. Since no plant related air activity was detected in other atmospheric monitoring media in 2000, no rainwater samples from SQN were analyzed in this reporting period.

Results

The results from the analysis of air particulate samples are summarized in Table H-3. Gross beta activity in 2000 was consistent with levels reported in previous years. The average gross beta activity for samples for indicator locations was 0.022 pCi/m^3 while the average for samples from control locations was 0.023 pCi/m^3. The annual average of the gross beta activity in air particulate filters at these stations for the years 1971-2000 are presented in Figure H-2. Increased levels due to fallout from atmospheric nuclear weapons testing are evident, especially in 1971, 1977, 1978, and 1981. Evidence of a small increase resulting from the Chernobyl accident can also be seen in 1986. These patterns are consistent with data from monitoring programs conducted during the preoperation and construction phases at other TVA nuclear plant sites.

Only naturally occurring radionuclides were identified by the monthly gamma spectral analysis of the air particulate samples. No fission or activation products were detected. As shown in Table H-4, I-131 was not detected in any of the charcoal cartridge samples collected in 2000.

TERRESTRIAL MONITORING

Terrestrial monitoring is accomplished by collecting samples of environmental media that may transport radioactive material from the atmosphere to humans. For example, radioactive material may be deposited on a vegetable garden and be ingested along with the vegetables or it may be deposited on pasture grass where dairy cattle are grazing. When the cow ingests the radioactive material, some of it may be transferred to the milk and consumed by humans who drink the milk. Therefore, samples of milk, vegetation, soil, and food crops are collected and analyzed to determine potential impacts from exposure through this pathway. The results from the analysis of these samples are shown in Tables H-5 through H-13.

A land use survey is conducted annually to locate milk producing animals and gardens within a 5-mile radius of the plant. Two dairy farms were located on the east side of the river between 4 and 6 miles from the plant and one small farm with a milk cow is located approximately 2 miles northwest of the plant. These three locations were sampled in the SQN sampling program. The results of the 2000 land use survey are presented in Appendix G.

Sample Collection and Analysis

Milk samples are collected every 2 weeks from the three indicator locations and from at least one control dairy. These samples are placed on ice for transport to the Radioanalytical Laboratory. A specific analysis for I-131 and a gamma spectroscopy analysis are performed on each sample and Sr-89,90 analysis is performed quarterly.

Vegetation is being sampled every 4 weeks from one farm that had milk producing animals in the past. An additional sample is collected from one control station. The samples are collected by cutting or breaking enough vegetation to provide between 100 and 200 grams of sample. Care is taken not to include any soil with the vegetation. The sample is placed in a container with 1650 ml of 0.5 N NaOH for transport back to the Radioanalytical Laboratory for I-131 analysis.

A second sample of between 750 and 1000 grams is also collected from each location. After drying and grinding, these samples are analyzed by gamma spectroscopy. Once each quarter, the samples are ashed after the gamma analysis is completed and analyzed for Sr-89,90.

Soil samples are collected annually from the air monitoring locations. The samples are collected with either a "cookie cutter" or an auger type sampler. After drying and grinding, the sample is analyzed by gamma spectroscopy. When the gamma analysis is complete, the sample is ashed and analyzed for Sr-89,90.

Samples representative of food crops raised in the area near the plant are obtained from individual gardens. Types of foods may vary from year to year as a result of changes in the local vegetable gardens. In 2000 samples of apples, cabbage, corn, green beans, potatoes, and tomatoes were collected from local gardens. Samples of these same food crops were purchased from area produce markets to serve as control samples. The edible portion of each sample is analyzed by gamma spectroscopy.

Results

The results from the analysis of milk samples are presented in Table H-5. No radioactivity attributable to SQN operations was identified. All I-131 results were less than the established nominal LLD of 0.4 pCi/liter. Strontium-90 was detected above the nominal LLD in one sample. The Sr-90 concentration measured in 2000 was 3.05 pCi/liter. This concentration is consistent with historical data reported in milk as a result of fallout from atmospheric nuclear weapons tests (Reference 1). Figure H-3 displays the average Sr-90 concentrations measured in milk since 1971. The concentrations have steadily decreased as a result of the 28-year half-life of Sr-90 and the washout and transport of the element through the soil over the period. By far the predominant isotope reported in milk samples was the naturally occurring K-40. The average K-40 concentration was approximately 1370 pCi/liter for milk samples analyzed in 2000.

Results from the analysis of vegetation samples (Table H-6) were similar to those reported for milk. All I-131 values were less than the nominal LLD. All strontium-89 results were less than the analysis specific LLD. Strontium-90 was identified in a total of five samples at concentrations ranging from 13.0 to 23.9 pCi/Kg. These concentrations are consistent with results produced by nuclear weapons fallout. The highest radionuclide concentrations identified in vegetation were for the naturally occurring isotopes K-40 and Be-7.

A total of twelve soil samples were collected and analyzed. Eleven of soil samples contained measurable levels of Cs-137 with the maximum concentration being 0.73 pCi/g. These concentrations are consistent with levels previously reported from fallout. All other radionuclides reported were naturally occurring isotopes (Table H-7).

A plot of the annual average Cs-137 concentrations in soil is presented in Figure H-4. Like the levels of Sr-90 in milk, concentrations of Cs-137 in soil are steadily decreasing as a result of the cessation of weapons testing in the atmosphere, the 30-year half-life of Cs-137 and transport through the environment.

Radionuclides reported in food samples were all naturally occurring. The maximum K-40 value was 3600 pCi/kg in potatoes. Analysis of these samples indicated no contribution from plant activities. The results are reported in Tables H-8 through H-13.

LIQUID PATHWAY MONITORING

Potential exposures from the liquid pathway can occur from drinking water, ingestion of edible fish and invertebrates, or from direct radiation exposure from radioactive materials deposited in the river sediment. The monitoring program includes the collection of samples of surface water, groundwater, drinking water supplies, fish, Asiatic clams (there is no known human consumption of these clams from the Tennessee River), and bottom and shoreline sediment. Samples from the reservoir are collected both upstream and downstream from the plant.

Sample Collection and Analysis

Samples of surface water are collected from the Tennessee River downstream and upstream of the plant using automatic sampling systems. A timer turns on the system at least once every 2 hours and the sample is collected into a composite jug. A 1-gallon sample is removed from the composite jug at 4-week intervals and the remaining water in the jug is discarded. The composite sample is analyzed for gamma emitting radionuclides and for gross beta activity. A quarterly composite sample is analyzed for Sr-89,90 and tritium.

Samples are collected by an automatic sampling system at the first downstream drinking water intake and at the water intake for the city of Dayton located approximately 20 miles upstream. These samples are collected in the same manner as the surface water samples and analyzed by gamma spectroscopy and for gross beta activity. At other selected locations, grab samples are collected from drinking water systems which use the Tennessee River as their source. These samples are analyzed every 4 weeks by gamma spectroscopy and for gross beta activity. A quarterly composite sample from each station is analyzed for Sr-89,90 and tritium. The sample collected at the water intake for the city of Dayton also serves as control sample for surface water.

Groundwater is sampled from an onsite well and from a private well in an area unaffected by SQN. The quarterly composite samples are prepared for each location and analyzed by gamma spectroscopy. Analyses are also performed for gross beta activity, Sr-89,90 and tritium.

Samples of commercial and game fish species are collected semiannually from each of two reservoirs: the reservoir on which the plant is located (Chickamauga Reservoir) and the upstream reservoir (Watts Bar Reservoir). The samples are collected using a combination of netting techniques and electrofishing. Samples of all species are prepared from filleted fish. After drying and grinding, the samples are analyzed by gamma spectroscopy.

Bottom sediment samples are collected semiannually from monitoring locations using a dredging apparatus or divers. Samples of shoreline sediment are collected from two downstream recreational use areas and one upstream location. The samples are dried and ground and analyzed by gamma spectroscopy.

Samples of Asiatic clams are collected semiannually from one location below the plant and one location above the plant. There is no known use of these clams for human consumption. The clams are usually collected in the dredging or diving process with the sediment. Enough clams are collected to produce approximately 50 grams of wet flesh. The flesh is separated from the shells, and the dried flesh samples are analyzed by gamma spectroscopy.

Results

There were no fission or activation product radionuclides identified from the gamma spectroscopy or specific analyses performed on surface water samples. Gross beta activity above the nominal LLD value was measured in most surface water samples. The gross beta concentrations in samples from the indicator locations averaged 2.9 pCi/liter and average for control locations was 3.4 pCi/liter. The values were consistent with previously reported levels. A trend plot of the gross beta activity in surface water samples from 1971 through 2000 is presented in Figure H-5. A summary table of the results is shown in Table H-14.

There were no fission or activation product radionuclides identified in drinking water samples. Average gross beta activity was 2.8 pCi/liter for the downstream stations and 3.4 pCi/liter at the control stations. The results are shown in Table H-15 and a trend plot of the gross beta activity in drinking water from 1971 to the present is presented in Figure H-6.

No fission or activation products were detected by the gamma spectroscopy analyses performed on well water. Gross beta concentrations in samples from the onsite well averaged 2.8 pCi/liter, while the average from the offsite well was 7.7 pCi/liter. Measurable levels of tritium continued to be present in the samples collected from the onsite well. The radiological environmental monitoring program schedule requires tritium analysis of quarterly composite samples from this well. The tritium analysis results for these quarterly composite samples are reported in Table H-16 of this report. In addition to the routine scheduled quarterly composite samples, tritium analyses were also performed on the samples collected every four weeks that are used to prepare the quarterly composite samples. The tritium concentration measured in routine quarterly composite samples averaged 693 pCi/liter.

Cesium-137 was identified in a total of five fish samples. All of these samples were from control sampling locations. The maximum Cs-137 concentration for control samples was 0.06 pCi/g. There was no Cs-137 detected above the LLD value of 0.03 pCi/g in samples from the indicator sampling locations. The plot of the annual Cs-137 concentration in samples of crappie is presented in Figure H-7. Other radioisotopes found in fish were naturally occurring with the most notable being K-40. The concentrations of K-40 ranged from 8.0 pCi/g to 17.9 pCi/g. The results are summarized in Tables H-17, H-18, and H-19.

Radionuclides of the types that can be produced by nuclear power plant operations were identified in bottom sediment samples. The radionuclides identified were Cs-137, and Co-60.

The average Cs-137 concentration measured for samples from the downstream locations was 0.64 pCi/g and the average concentration for control locations was 0.56 pCi/g. The presence of Cs-137 was measured in half of the samples collected from downstream shoreline sediment monitoring locations. The maximum concentration for shoreline sediment collected from the downstream location was 0.08 pCi/g. A value of 0.08 pCi/g for Cs-137 was also measured in one of the samples collected from the control location. The concentrations of Cs-137 in sediment are consistent with previously identified fallout levels. Two samples of bottom sediment collected from a downstream location contained measurable levels of Co-60. The average concentration of Co-60 in these samples was 0.14 pCi/g. There was no Co-60 detected in samples from an upstream location. A dose assessment of the impact to the general public from this activity produces a negligible dose equivalent. Results from the analysis of bottom sediment samples are shown in Table H-20. Results from the analysis of shoreline sediment samples are shown in Table H-21.

Graphs of the Cs-137 and Co-60 concentrations in bottom sediment are presented in Figures H-8 and H-9, respectively. Figure H-10 presents a plot of the Cs-137 concentrations measured in shoreline sediment since 1980.

Only naturally occurring radioisotopes were identified in clam flesh samples. The results from the analysis of these samples are presented in Table H-22.

ASSESSMENT AND EVALUATION

Potential doses to the public are estimated from measured effluents using computer models. These models were developed by TVA and are based on methodology provided by the NRC in Regulatory Guide 1.109 for determining the potential dose to individuals and populations living in the vicinity of a nuclear power plant. The doses calculated are a representation of the dose to a "maximum exposed individual." Some of the factors used in these calculations (such as ingestion rates) are maximum expected values which will tend to overestimate the dose to this "hypothetical" person. In reality, the expected dose to actual individuals is significantly lower.

The area around the plant is analyzed to determine the pathways through which the public may receive an exposure. As indicated in Figure 2, the two major ways by which radioactivity is introduced into the environment are through liquid and gaseous effluents.

For liquid effluents, the public can be exposed to radiation from three sources: drinking water from the Tennessee River, eating fish caught in the Tennessee River, and direct exposure to radioactive material due to activities on the banks of the river (recreational activities). Data used to determine these doses are based on guidance given by the NRC for maximum ingestion rates, exposure times, and distribution of the material in the river. Whenever possible, data used in the dose calculation are based on specific conditions for the SQN area.

For gaseous effluents, the public can be exposed to radiation from several sources: direct radiation from the radioactivity in the air, direct radiation from radioactivity deposited on the ground, inhalation of radioactivity in the air, ingestion of vegetation which contains radioactivity deposited from the atmosphere, and ingestion of milk from animals which consumed vegetation containing deposited radioactivity. The concentrations of radioactivity in the air and the soil are estimated by computer models which use the actual meteorological conditions to determine the distribution of the effluents in the atmosphere. Again, as many of the parameters as possible are based on actual site specific data.

Results

The estimated doses to the maximum exposed individual due to radioactivity released from SQN in 2000 are presented in Table 3. These estimates were made using the concentrations of the liquids and gases measured in the effluent monitoring points. Also shown are the regulatory limits for these doses and a comparison between the calculated dose and the corresponding limit. The maximum calculated whole body dose equivalent from measured liquid effluents as reported in Table 3 is 0.027 mrem/year, or less than 1.0 percent of the limit. The maximum organ dose equivalent from gaseous effluents is 0.064 mrem/year. A more complete description of the effluents released from SQN and the corresponding doses projected from these effluents can be found in the SQN Annual Radioactive Effluent Release Report.

As stated earlier in this report, the estimated increase in radiation dose equivalent to the general public resulting from the operation of SQN is negligible when compared to the dose from natural background radiation. The results from environmental samples are compared with the concentrations from the corresponding control stations as well as appropriate preoperational and background data to determine influences from the plant. During this report period, Co-60 and Cs-137 were detected in bottom sediment. Measurable levels of Cs-137 were also detected in fish, soil, and shoreline sediment and tritium was detected in ground water from the on site monitoring well. The Cs-137 concentrations measured in shoreline sediment, bottom sediment and fish are consistent with levels identified previously that are the result of fallout from past atmospheric nuclear weapons testing. The Co-60 identified in sediment samples downstream from the plant would produce no measurable increase in the dose to the general public. The tritium concentrations measured in ground water were well below any levels requiring special actions. The presence of detectable tritium in the on site well does not represent an exposure pathway to the general public.

Dose estimates based on the concentrations of radioactivity found in samples of environmental media show that doses estimated for persons at indicator locations were essentially identical to those determined for persons at control stations. More than 99 percent of the activity detected in environmental samples was the result of the naturally occurring radionuclides. Sr-90 and Cs-137, which are long-lived radioisotopes found in fallout from nuclear weapons testing were the primary man-made radionuclides detected in samples from the SQN REMP. Concentrations of Sr-90 and Cs-137 are consistent with levels measured in TVA's preoperational radiological environmental monitoring programs. Figures H-3 and H-4 and Figure H-7 through H-10 indicate that concentrations of Sr-90 and Cs-137 in the environment have decreased since the cessation of atmospheric weapons testing in 1981. This decrease is the result of the decay of the two nuclides and the redistribution of the materials in the environment.

Conclusions

It is concluded from the above analysis of the environmental sampling results and from the trend plots presented in Appendix H that the exposure to members of the general public which may have been attributable to SQN is negligible. The radioactivity reported herein is primarily the result of fallout or natural background radiation. Any activity which may be present as a result of plant operations does not represent a significant contribution to the radiation exposure to Members of the Public.

REFERENCES

1. Merril Eisenbud, Environmental Radioactivity, Academic Press, Inc., New York, NY, 1987.

2. National Council on Radiation Protection and Measurements, Report No. 93, "Ionizing Radiation Exposure of the Population of the United States," September 1987.

3. United States Nuclear Regulatory Commission, Regulatory Guide 8.29, "Instruction Concerning Risks from Occupational Radiation Exposure," July 1981.

4. Hansen, W.G., Campbell, J. E., Fooks, J. H., Mitchell, H.C., and Eller C.H., Farming Practices and Concentrations of Emission Products in Milk, U.S. Department of Health, Education, and Welfare; Public Health Service Publication No. 999-R-6, May 1964.

Table 1

COMPARISON OF
PROGRAM LOWER LIMITS OF DETECTION WITH THE REGULATORY LIMITS FOR
MAXIMUM ANNUAL AVERAGE EFFLUENT CONCENTRATIONS
RELEASED TO UNRESTRICTED AREAS
AND REPORTING LEVELS

	Concentrations in Water, pCi/Liter			Concentrations in Air, pCi/Cubic Meter		
	Effluent Concentration[1]	Reporting Level[2]	Lower limit of Detection[3]	Effluent Concentration[1]	Reporting Level[2]	Lower limit of Detection[3]
H-3	1,000,000	20,000	300	100,000		
Cr-51	500,000		45	30,000		0.02
Mn-54	30,000	1,000	5	1,000		0.005
Co-58	20,000	1,000	5	1,000		0.005
Co-60	3,000	300	5	50		0.005
Zn-65	5,000	300	10	400		0.005
Sr-89	8,000		5	1,000		0.0011
Sr-90	500		2	6		0.0004
Nb-95	30,000	400	5	2,000		0.005
Zr-95	20,000	400	10	400		0.005
Ru-103	30,000		5	900		0.005
Ru-106	3,000		40	20		0.02
I-131	1,000	2	0.4	200	0.9	0.03
Cs-134	900	30	5	200	10	0.005
Cs-137	1,000	50	5	200	20	0.005
Ce-144	3,000		30	40		0.01
Ba-140	8,000	200	25	2,000		0.015
La-140	9,000	200	10	2,000		0.01

Note: 1 pCi = 3.7 x10^{-2} Bq.

Note: For those reporting levels that are blank, no value is given in the reference.

1 Source: Table 2 of Appendix B to 10 CFR 20.1001-20.2401

2 Source: SQN Offsite Dose Calculation Manual, Table 2.3-2

3 Source: Table E-1 of this report.

Table 2

Results from the
Intercomparison of Environmental Dosimeters

Year	TVA Results mrem	Average, all Respondents mrem	Calculated Exposure (See Note 1) mrem	% Difference TVA: Calculated	% Difference Respondents: Calculated
Field Dosimeters					
74	15.0	16.3	16.3	-8.0	0.0
77	30.4	31.5	34.9	-12.9	-9.7
79	13.8	16.0	14.1	-2.1	13.5
81	31.8	30.2	30.0	6.0	0.7
82	43.2	45.0	43.5	-0.7	3.4
84	73.0	75.1	75.8	-3.7	-0.9
86a	33.2	28.9	29.7	11.8	-2.7
86b	9.4	10.1	10.4	-9.6	-2.9
93a	24.4	26.4	27.0	-9.6	-2.2
93b	27.6	26.4	27.0	2.2	-2.2
96a	16.9	18.9	19.0	-10.9	-0.5
96b	17.6	18.9	19.0	-7.4	-0.5
Low Irradiated Dosimeters					
74	27.9	28.5	30.0	-7.0	-5.0
79	12.1	12.1	12.2	-0.8	-0.8
86	18.2	16.2	17.2	5.8	-5.8
93a	24.9	25.0	25.9	-3.9	-3.5
93b	27.8	25.0	25.9	7.3	-3.5
High Irradiated Dosimeters					
77	99.4	86.2	91.7	8.4	-6.0
79	46.1	43.9	45.8	0.7	-4.1
81a	84.1	75.8	75.2	11.8	0.8
81b	102.0	90.7	88.4	15.4	2.6
82a	179.0	191.0	202.0	-11.4	-5.4
82b	136.0	149.0	158.0	-13.9	-5.7
84a	85.6	77.9	79.9	7.1	-2.5
84b	76.8	73.0	75.0	2.4	-2.7
93a	67.8	69.8	72.7	-6.7	-4.0
93b	80.2	69.8	72.7	10.3	-4.0
96a	60.7	55.2	58.1	4.5	-5.0
96b	59.4	55.2	58.1	2.2	-5.0

Notes: 1. The calculated exposure is the "known" exposure determined by the testing agency.

Table 3

Maximum Dose Due to Radioactive Effluent Releases
Sequoyah Nuclear Plant
2000
mrem/year

Dose From Liquid Effluents

Type	2000 Dose	NRC Limit	Percent of NRC Limit
Total Body	2.69E-02	3	< 1.0
Any Organ	3.42E-02	10	< 1.0

Doses From Gaseous Effluents

Type	2000 Dose	NRC Limit	Percent of NRC Limit
Noble Gas (Gamma)	2.27E-02	10	< 1.0
Noble Gas (Beta)	5.7E-02	20	< 1.0
Any Organ	6.44E-02	15	< 1.0

Total Cumulative Dose

Type	2000 Dose	EPA Limit	Percent of EPA Limit
Total Body or Any Other Organ	1.43E-01	25	< 1.0
Thyroid	1.26E-01	75	< 1.0

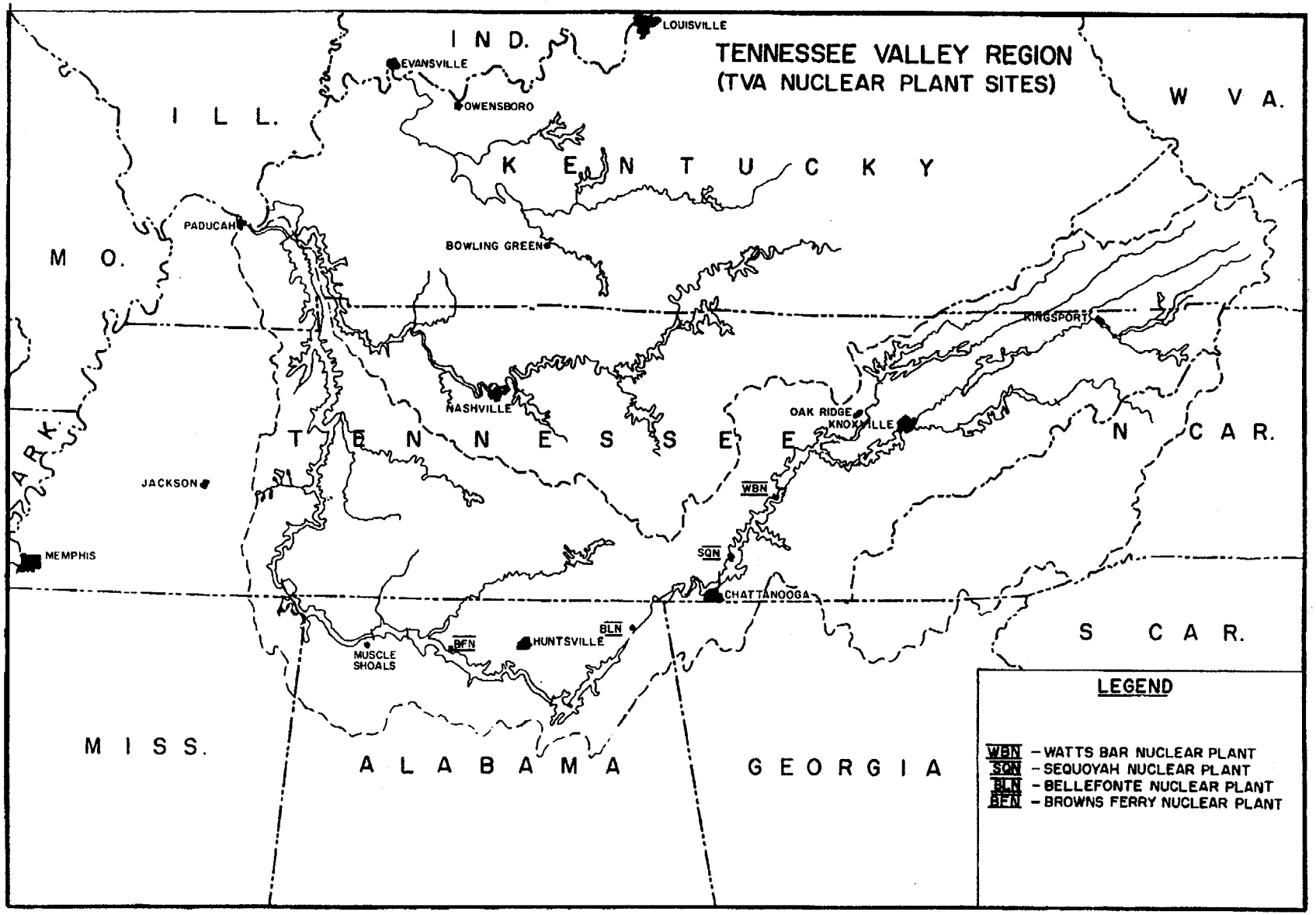
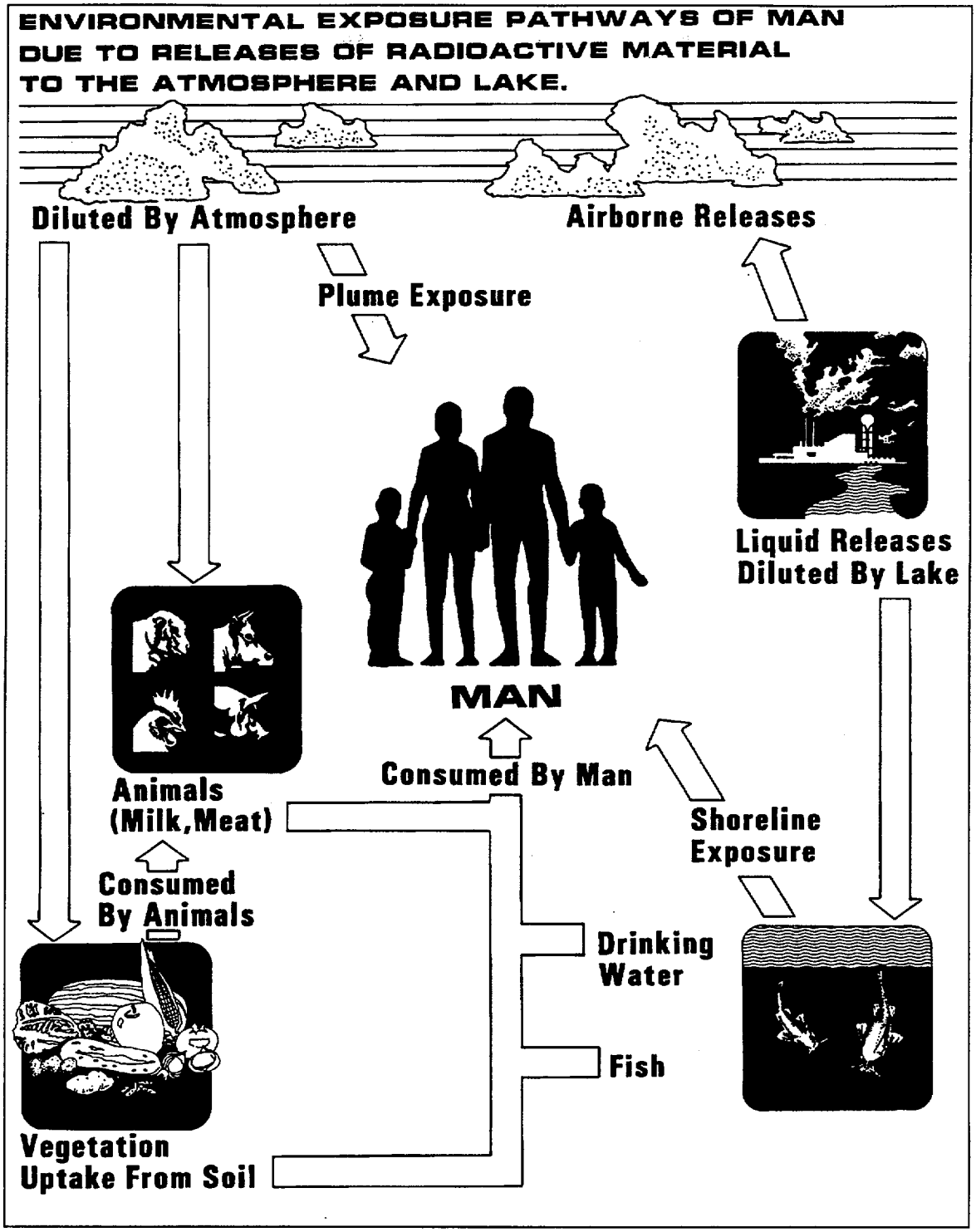

TENNESSEE VALLEY REGION
(TVA NUCLEAR PLANT SITES)
LOUISVILLE
IND.
EVANSVILLE
OWENSBORO
ILL.
W VA.
KENTUCKY
PADUCAH
BOWLING GREEN
MO.
KINGSPORT
NASHVILLE
OAK RIDGE
KNOXVILLE
TENNESSEE
N CAR.
ARK.
JACKSON
WBN
MEMPHIS
SQN
CHATTANOOGA
S CAR.
BLN
BFN
HUNTSVILLE
MUSCLE SHOALS
MISS.
ALABAMA
GEORGIA
LEGEND
WBN - WATTS BAR NUCLEAR PLANT
SQN - SEQUOYAH NUCLEAR PLANT
BLN - BELLEFONTE NUCLEAR PLANT
BFN - BROWNS FERRY NUCLEAR PLANT

Figure 1

Figure 2


ENVIRONMENTAL EXPOSURE PATHWAYS OF MAN
DUE TO RELEASES OF RADIOACTIVE MATERIAL
TO THE ATMOSPHERE AND LAKE.
Diluted By Atmosphere
Airborne Releases
Plume Exposure
Liquid Releases
Diluted By Lake
MAN
Consumed By Man
Animals
(Milk,Meat)
Shoreline
Exposure
Consumed
By Animals
Drinking
Water
Fish
Vegetation
Uptake From Soil

APPENDIX A

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM AND SAMPLING LOCATIONS

Table A-1
SEQUOYAH NUCLEAR PLANT
RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM[a]

Exposure Pathway and/or Sample	Number of Samples and Locations[b]	Sampling and Collection Frequency	Type and Frequency of Analysis
1. AIRBORNE			
a. Particulates	4 samples from locations (in different sectors) at or near the site boundary (LM-2, LM-3, LM-4, and LM-5).	Continuous sampler operation with sample collection once per 7 days (more frequently if required by dust loading).	Analyze for gross beta radioactivity greater than or equal to 24 hours following filter change. Perform gamma isotopic analysis on each sample when gross beta is greater than 10 times yearly mean of control samples. Composite at least once per 31 days (by location) for gamma scan.
	4 samples from communities approximately 6-10 miles from the plant (PM-2, 3, 8, and 9). 4 samples from control locations greater than 10 miles from the plant (RM-1 RM-2, RM-3and RM-4).		
b. Radioiodine	Same locations as air particulates.	Continuous sampler operation with charcoal canister collected at same time as particulate filters at least once per 7 days.	I-131 by gamma scan on each sample.
c. Soil	Samples from same locations as air particulates.	Once per year.	Gamma scan, Sr-89, Sr-90 once per year.
d. Rainwater	Same locations as air particulates.	Composite sample at least once per 31 days.	Analyzed for gamma nuclides only if radioactivity in other media indicates the presence of increased levels of fallout.

Table A-1

SEQUOYAH NUCLEAR PLANT
RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM[a]

Exposure Pathway and/or Sample	Number of Samples and Locations[b]	Sampling and Collection Frequency	Type and Frequency of Analysis
2. DIRECT RADIATION	2 or more dosimeters (TLDs) placed at locations (in different sectors) at or near the site boundary in each of the 16 sectors.	At least once per 92 days.	Gamma dose at least once per 92 days.
	2 or more dosimeters placed at stations located approximately 4 to 5 miles from the plant in each of the 16 sectors.		
	2 or more dosimeters in approximately 20 locations of special interest.		
3. WATERBORNE			
a. Surface water	TRM 503.8[d] TRM 483.4	Collected by automatic sequential-type sampler[c] with composite samples collected over a period of less than or equal to 31 days.	Gross beta and gamma scan on each composite sample. Composite for Sr-89, Sr-90, and tritium analysis at least once per 92 days.
b. Ground water	1 sample adjacent to the plant (Well No. 6).	At least once per 31 days.	Composited for gross beta, gamma scan, Sr-89, Sr-90 and tritium at least once per 92 days.
	1 sample from ground water source upgradient (Farm HW).	At least once per 92 days.	Gross beta, gamma scan, Sr-89, Sr-90 and tritium at least once per 92 days.

Table A-1

SEQUOYAH NUCLEAR PLANT
RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM[a]

Exposure Pathway and/or Sample	Number of Samples and Locations[b]	Sampling and Collection Frequency	Type and Frequency of Analysis
c. Drinking Water	1 sample at the first potable surface water supply downstream from the plant (TRM 473.0).	Collected by automatic sequential-type sampler[c] with composite sample collected over a period of less than or equal to 31 days.	Gross beta and gamma scan on each composite sample. Composite for tritium, Sr-89 and Sr-90 at least once per 92 days.
	1 sample at the next 2 downstream potable water systems (greater than 10 miles downstream) (TRM 469.9 and TRM 465.3).	Grab sample once per 31 days.	
	1 sample at a upstream control location (TRM 503.8[d]).	Samples collected by sequential-type sampler[c] with composite sample collected over a period of less than or equal to 31 days.	
d. Sediment	TRM 496.5 TRM 483.4 TRM 480.8	At least once per 184 days.	Gamma scan of each sample.
e. Shoreline sediment	TRM 485 TRM 480 TRM 479	At least once per 184 days.	Gamma scan of each sample.

Table A-1
SEQUOYAH NUCLEAR PLANT
RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM[a]

Exposure Pathway and/or Sample	Number of Samples and Locations[b]	Sampling and Collection Frequency	Type and Frequency of Analysis
4. INGESTION			
a. Milk	1 sample from milk producing animals in each of 1-3 areas indicated by the cow census where doses are calculated to be highest. If samples are not available from a milk animal location, doses to that area will be estimated by projecting the doses from concentrations detected in milk from other sectors or by sampling vegetation where milk is not available. At least one sample from a control location	At least once per 15 days.	Gamma isotopic and I-131 analysis of each sample. Sr-89 and Sr-90 once per quarter.
b. Fish	1 sample each from Chickamauga and Watts Bar Reservoirs.	At least once per 184 days. One Sample of each of the following species: Channel Catfish Crappie Smallmouth Buffalo	Gamma scan on edible portion.
c. Invertebrates (Asiatic Clams)	1 sample downstream from the discharge. 1 sample upstream from the plant. (No permanent stations established; depends on location of clams).	At least once per 184 days.	Gamma scan on edible portion.

Table A-1
SEQUOYAH NUCLEAR PLANT
RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

Exposure Pathway and/or Sample	Number of Samples and Locations[b]	Sampling and Collection Frequency	Type and Frequency of Analysis
d. Food Products	1 sample each of principal food Products grown at private gardens and/or farms in the immediate vicinity of the plant.	At least once per 365 days at time of harvest. The types of foods available for sampling will vary. Following is A list of typical foods which may be available: Cabbage, lettuce, or greens Corn Green Beans Potatoes Tomatoes	Gamma scan on edible portion.
	One sample of each of the same foods grown at greater than 10 miles distance from the plant.		
e. Vegetation	Samples from a farm that has produced milk in the past but is not currently producing milk. (Farm EM)	At least once per 31 days.	I-131 and gamma scan at least once per 31 days. Sr-89 and Sr-90 analysis at least once per 92 days.
	Control sample from one control dairy. (Farm S)		

a. The sampling program outlined in this table is that which was in effect at the end of 2000.
b. Sample locations, sector and distance from plant, are described in Table A-2 and A-3and shown in Figures A-1, A-2, and A-3.
c. Composite samples shall be collected by collecting an aliquot at intervals not exceeding 2 hours.
d. The sample collected at this location shall be considered a control for the drinking water and surface water.

Table A-2
SEQUOYAH NUCLEAR PLANT
RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM
SAMPLING LOCATIONS

Map Location Number[a]	Station	Sector	Approximate Distance (Miles)	Indicator (I) or Control (C)	Samples Collected[b]
2	LM-2	N	0.8	I	AP,CF,R,S
3	LM-3	SSW	2.0	I	AP,CF,R,S
4	LM-4	NE	1.5	I	AP,CF,R,S
5	LM-5	NNE	1.8	I	AP,CF,R,S
7	PM-2	SW	3.8	I	AP,CF,R,S
8	PM-3	W	5.6	I	AP,CF,R,S
9	PM-8	SSW	8.7	I	AP,CF,R,S
10	PM-9	WSW	2.6	I	AP,CF,R,S
11	RM-1	SW	16.7	C	AP,CF,R,S
12	RM-2	NNE	17.8	C	AP,CF,R,S
13	RM-3	ESE	11.3	C	AP,CF,R,S
14	RM-4	NW	20.0	C	AP,CF,R,S
15	Farm B	NE	43.0	C	M
19	Farm HW	NW	1.2	I	M,W[c]
20	Farm EM	N	2.6	I	V
21	Farm HS	E	4.6	I	M
22	Farm RJ	ESE	3.9	I	M
23	Farm EH	ENE	9.5	C	M,V
24	Well No. 6	NNE	0.15	I	W
31	TRM 473.0 (C. F. Industries)	--	10.7[d]	I	PW
32	TRM 469.9 (E. I. DuPont)	--	13.8[d]	I	PW
33	TRM 465.3 (Chattanooga)	--	18.4[d]	I	PW
35	TRM 503.8 (Dayton)	--	20.1[d]	C	PW,SW
36	TRM 496.5	--	12.8[d]	C	SD
37	TRM 485.0	--	1.3[d]	C	SS
38	TRM 483.4	--	0.3[d]	I	SD,SW
39	TRM 480.8	--	2.9[d]	I	SD
40	TRM 479.0	--	4.7[d]	I	SS
44	TRM 480.0	--	3.7[d]	I	SS
46	Chickamauga Reservoir (TRM 471-530)		--	I/C	F,CL
47	Watts Bar Reservoir (TRM 530-602)		--	C	F

a. See Figures A-1, A-2, and A-3
b. Sample codes:
AP = Air particulate filter
CF = Charcoal filter
CL = Clams
F = Fish
M = Milk
PW = Public Water
R = Rainwater
S = Soil
SD = Sediment
SS = Shoreline Sediment
SW = Surface water
V = Vegetation
W = Well water
c. A control for well water.
d. Distance from plant discharge (TRM 483.7).

Table A-3
SEQUOYAH NUCLEAR PLANT
THERMOLUMINESCENT DOSIMETER (TLD) LOCATIONS

Map Location Number[a]	Station	Sector	Approximate Distance (miles)	Onsite (On)[b] or Offsite (Off)
3	SSW-1C	SSW	2.0	On
4	NE-1A	NE	1.5	On
5	NNE-1	NNE	1.8	On
7	SW-2	SW	3.8	Off
8	W-3	W	5.6	Off
9	SSW-3	SSW	8.7	Off
10	WSW-2A	WSW	2.6	Off
11	SW-3	SW	16.7	Off
12	NNE-4	NNE	17.8	Off
13	ESE-3	ESE	11.3	Off
14	WNW-3	WNW	18.9	Off
49	N-1	N	0.6	On
50	N-2	N	2.1	Off
51	N-3	N	5.2	Off
52	N-4	N	10.0	Off
53	NNE-2	NNE	4.5	Off
54	NNE-3	NNE	12.1	Off
55	NE-1	NE	2.4	Off
56	NE-2	NE	4.1	Off
57	ENE-1	ENE	0.4	On
58	ENE-2	ENE	5.1	Off
59	E-1	E	1.2	On
60	E-2	E	5.2	Off
61	ESE-A	ESE	0.3	On
62	ESE-1	ESE	1.2	On
63	ESE-2	ESE	4.9	Off
64	SE-A	SE	0.4	On
65	E-A	E	0.3	On
66	SE-1	SE	1.4	On
67	SE-2	SE	1.9	On
68	SE-4	SE	5.2	Off
69	SSE-1	SSE	1.6	On
70	SSE-2	SSE	4.6	Off
71	S-1	S	1.5	On
72	S-2	S	4.7	Off
73	SSW-1	SSW	0.6	On
74	SSW-2	SSW	4.0	Off
75	SW-1	SW	0.9	On
76	WSW-1	WSW	0.9	On
77	WSW-2	WSW	2.5	Off
78	WSW-3	WSW	5.7	Off
79	WSW-4	WSW	7.8	Off
80	WSW-5	WSW	10.1	Off
81	W-1	W	0.8	On
82	W-2	W	4.3	Off
83	WNW-1	WNW	0.4	On
84	WNW-2	WNW	5.3	Off
85	NW-1	NW	0.4	On
86	NW-2	NW	5.2	Off
87	NNW-1	NNW	0.6	On
88	NNW-2	NNW	1.7	On
89	NNW-3	NNW	5.3	Off
90	SSW-1B	SSW	1.5	On

a. See Figures A-1, A-2, and A-3.
b. TLDs designated "onsite" are located 2 miles or less from the plant; "offsite" are located more than 2 miles from the plant.

Figure A-1

Radiological Environmental Monitoring Locations

Within 1 mile of the Plant

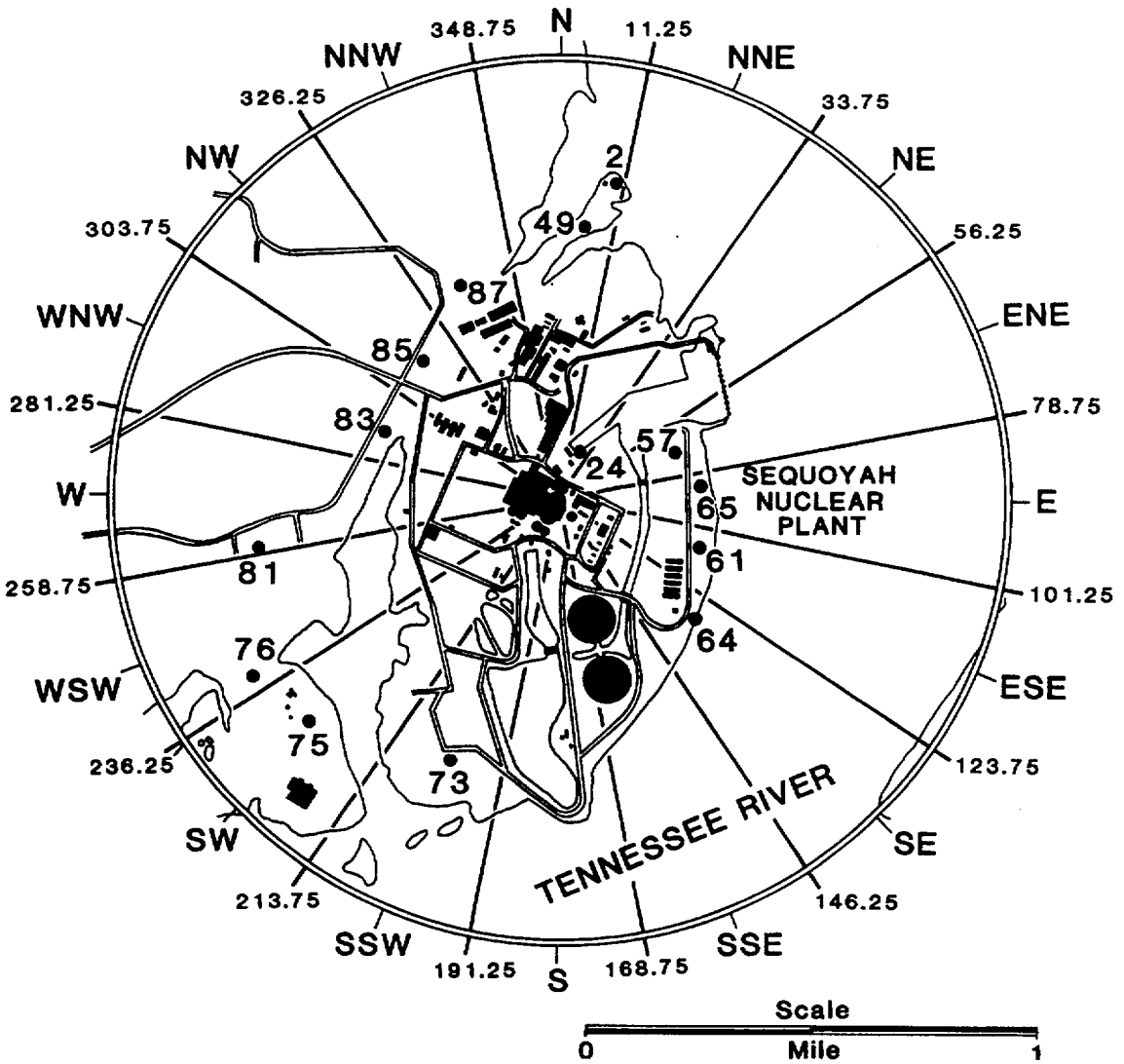


Figure A-2

Radiological Environmental Monitoring Locations

Between 1 and 5 miles from the Plant

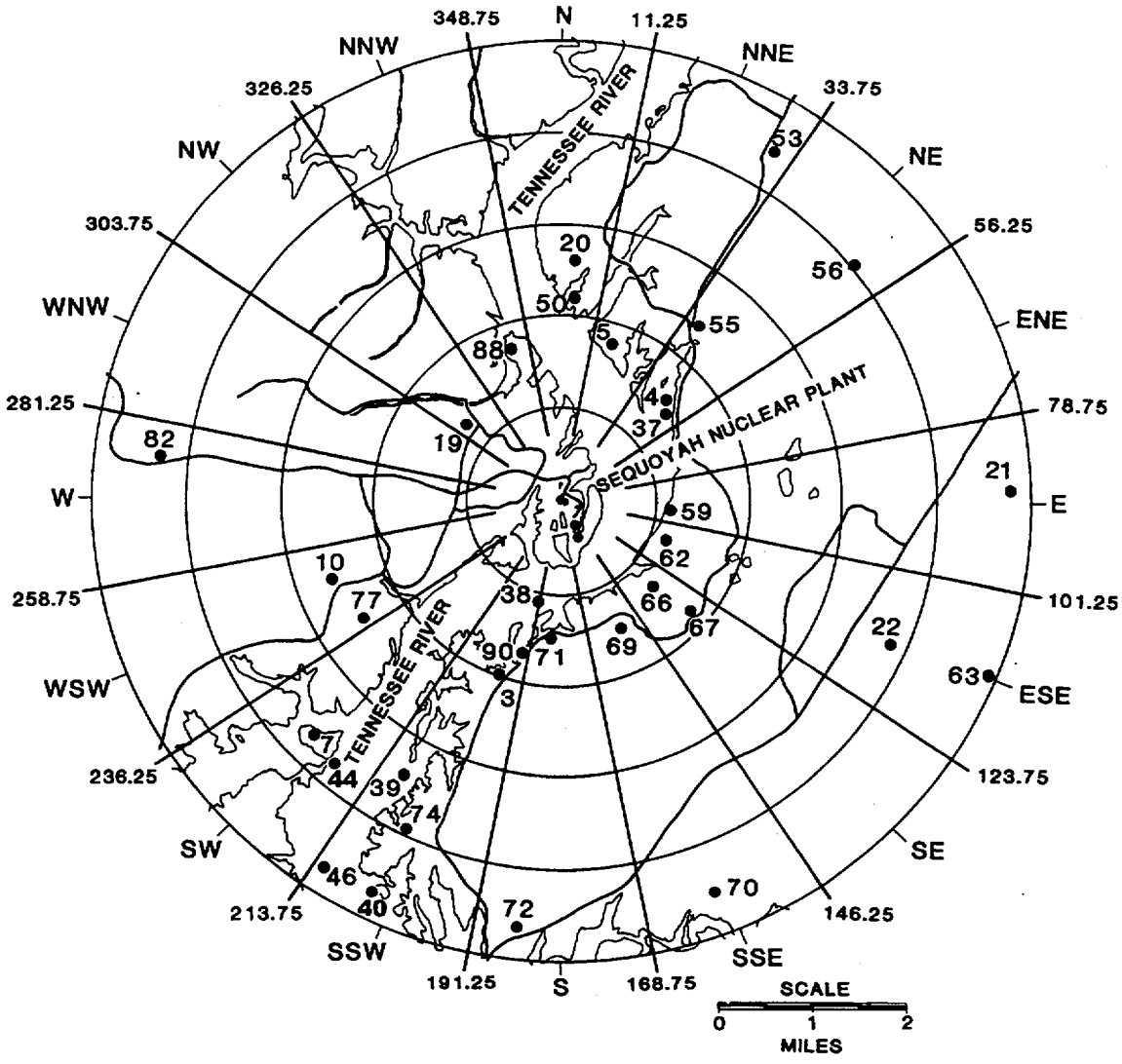
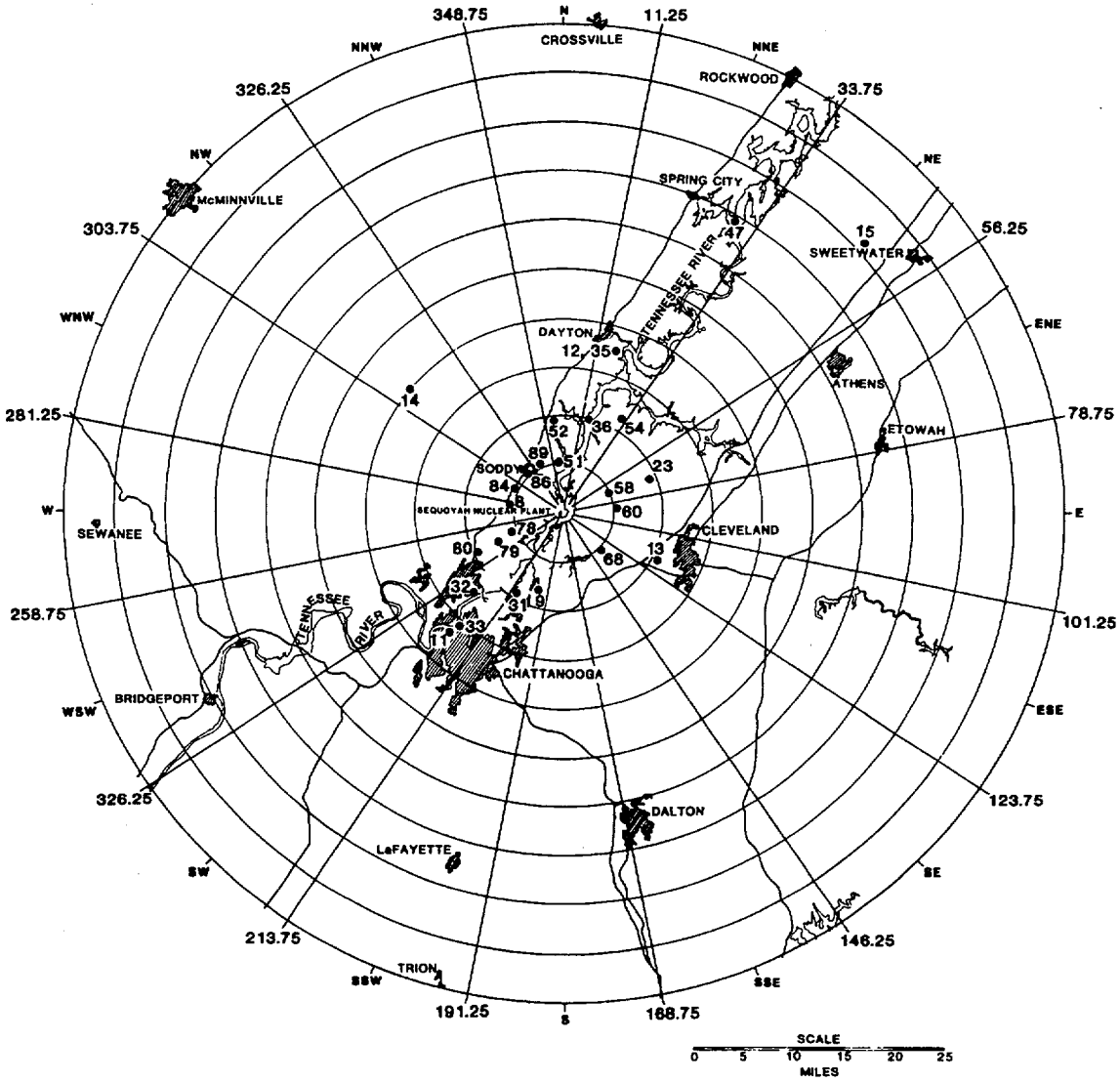


Figure A-3

Radiological Environmental Monitoring Locations

More than 5 miles from the Plant



APPENDIX B

2000 PROGRAM MODIFICATIONS

Appendix B

Radiological Environmental Monitoring Program Modification

Modifications were required in the SQN monitoring program during 2000 to address situations where samples were no longer available from a sampling location. There were three farms that had been sampled for milk, one indicator and two control locations, that permanently ended milk production during 2000. A dairy farm that has been used as a temporary replacement for indicator locations during periods when milk was not available from the normal sampling location was added to the sample schedule to replace the indicator farm. The SQN monitoring program requires that milk be collected from one control location. Historically three dairies meeting the criteria for a control sampling location have been included in the sampling schedule. Two of the three control dairy farms ended operations in 2000. A replacement dairy was identified and added to the schedule to provide at least two control sampling locations.

One of the four RM air sampling locations that serve as control sampling locations for the SQN program had to be relocated during 2000. The sampler designated as RM-4 located in an electrical substation at approximately 18.9 miles WNW had to be moved due to a change in location of the substation. The substation at the old location was shut down when a new substation was put into operation at approximately 20.0 miles NW. The RM-4 sampler was relocated to the new substation.

These program modifications are summarized in Table B-1.

Table B-1

Radiological Environmental Monitoring Program Modifications

Date	Station	Location	Description of Program Change
04/25/00	Farm J	1.1 Miles WNW	Location was dropped from the sampling program. This was a small farm with only one cow. The cow died and the owner decided not to replace the cow. Milk is no longer being produced at this location.
04/25/00	Farm RJ	3.9 Miles ESE	Added to sampling program to replace Farm J.
03/29/00	Farm C	16.0 Miles NE	Location was dropped from the sampling program. The dairy farm at this location went out of business. This was one of three control sampling locations and only one control sample is required.
09/05/00	RM-4	18.9 Miles WNW	Air sampling station had to be moved to new location (20.0 miles NW) due to relocation of electrical substation. This sampling location serves as one of four control locations for air sampling.
11/21/00	Farm S	12.0 Miles NNE	Location was dropped from the sampling program. The dairy farm at this location went out of business. This was one of two control sampling locations for milk sampling. The SQN program is required to have at least one control sampling location. A replacement farm was identified for addition to the program.
12/05/00	Farm EH	9.5 Miles ENE	Location was added to sampling program to provide a second control location for milk sampling.

APPENDIX C

PROGRAM DEVIATIONS

Appendix C

Program Deviations

During 2000, program deviations included missed air particulate and charcoal cartridge samples, unavailability of milk samples, and missed public water samples. A total of four air particulate and charcoal cartridge samples were not collected due to equipment malfunctions such as broken drive belts or problems with the sampling pump. During one sampling period, low air flow due to excessive filter loading caused three of twelve air particulate and charcoal cartridge samples not to be acceptable due to inadequate total volume. One of the four remote air sampling locations that serves as a control location was out of operation for a period of five weeks while it was being relocated.

Samples of public water could not be collected at the first downstream portable water user intake during two sampling periods due to problems with the sampling pump.

Milk samples were not available from a combination of indicator and control sampling locations at various times during the new year. Milk was not available at one or both of the small farms with only one milk cow during the first few months of the year. At one location the cow was dry and at the second location the cow had died. Substitute sampling was performed at the R. Johnson dairy farm during these periods. In late April, the owner of the Jones farm indicated that she did not intend to replace her cow and would no longer be producing milk at the farm. The Johnson dairy was added as a permanent replacement for the Jones farm. Two of the three dairy farms that were being sampled as control locations went out of business during the year. The SQN program only requires collection of one control location sample but extra farms were being sampled to ensure that a control sample was available for each sampling period. When the second control dairy went out of business, a replacement was added to the program.

Table C-1 provides additional details on the missed samples.

Table C-1
Radiological Environmental Monitoring Program Deviations

Date	Station	Location	Remarks
02/11/00-04/25/00	Farm HW	1.2 Miles NW	This is a small farm with only one cow. The cow was not producing milk. Substitute sampling was initiated at the Johnson dairy during this period.
03/29/00	Farm C	16.0 Miles NE	The dairy farm at this location went out of business. This was one of three control sampling locations. Collection is required at only one control location. Samples were collected as scheduled from the remaining two control locations.
01/01/00-04/25/00	Farm J	1.1 Miles WNW	This was a small farm that has had a single milk cow. The cow died in late 1999. The owner decided not to replace the cow. Milk is no longer being produced at this location.
04/12/00	TRM473.0	10.7 Miles Downstream	The water sample was not available from this location due to a failure of the sampling pump. The power was off to the pump. The breaker was reset and the sampler appeared to be operating correctly.
05/09/00	TRM473.0	10.7 Miles Downstream	The water sample was not available from this location due to a problem with the three-way valve that controls collection of the samples. The valve was replaced and a sample was collected as scheduled for the next sampling period.
08/08/00-09/05/00	RM-4	18.9 Miles WNW	The air particulate filter and charcoal cartridge samples were not available from this location. The sampler was out of service while it was being moved to the new location. The substation where the monitor was located was taken out of operation and the monitor had to be moved to the new substation. The sample is one of four air sampling locations used as a control location for the SQN program
09/27/00-10/04/00	LM-4	1.5 Miles NE	The air particulate filter and charcoal cartridge samples could not be collected from this location due to the low flow rate on the sampler. The problem required replacement of the sampling pump and a new pump had to be ordered. The sampler was out of service for two sampling periods.
10/25/00	Farm HS	4.6 Miles E	The dairy farm at this location went out of business and milk was no longer being produced at this location. The first indications from the owner was that this would be permanent but early in 2000 dairy operations were resumed at this farm.
11/7/00	Various	Various	Air particulate and charcoal cartridge samples could not be collected from three (LM-2, PM-3, and RM-2) of the twelve sampling locations due to low flow rate on the samplers. The low flow was caused by unusual loading on the filter. Provisions were made for a more frequent sample change during the next week but was not necessary. The flow rate returned to normal following a period of heavy rain.
11/04/00	PM-9	2.6 Miles WSW	Air particulate filter and charcoal cartridge samples could not be collected due to a failure in the sampling equipment. Power was lost to the sampling pump. The power was restored and the sampling system operated normally during the next sampling period.
11/21/00	Farm S	12.0 Miles NNE	The dairy farm at this location went out of business and milk is no longer being produced. This was one of two farms used as a control sampling location for the SQN program. Only one control sample is required. A sample was collected as scheduled from the other control sampling location.
11/28/00	RM-2	17.8 Miles NNE	The air particulate filter and charcoal cartridge samples could not be collected from this location due to a broken drive belt on the sampling pump. Repairs were made and samples collected as scheduled for the next sampling period.

APPENDIX D

ANALYTICAL PROCEDURES

Appendix D

Analytical Procedures

Analyses of environmental samples are performed by the radioanalytical laboratory located at the Western Area Radiological Laboratory facility in Muscle Shoals, Alabama. All analysis procedures are based on accepted methods. A summary of the analysis techniques and methodology follows.

The gross beta measurements are made with an automatic low background counting system. Normal counting times are 50 minutes. Water samples are prepared by evaporating 500 ml of samples to near dryness, transferring to a stainless steel planchet and completing the evaporation process. Air particulate filters are counted directly in a shallow planchet.

The specific analysis of I-131 in milk, and vegetation samples is performed by first isolating and purifying the iodine by radiochemical separation and then counting the final precipitate on a beta-gamma coincidence counting system. The normal count time is 50 minutes. With the beta gamma coincidence counting system, background counts are virtually eliminated and extremely low levels of activity can be detected.

After a radiochemical separation, samples analyzed for Sr-89, 90 are counted on a low background beta counting system. The sample is counted a second time after a 7-day ingrowth period. From the two counts the Sr-89 and Sr-90 concentrations can be determined.

Water samples are analyzed for tritium content by first distilling a portion of the sample and then counting by liquid scintillation. A commercially available scintillation cocktail is used.

Gamma analyses are performed in various counting geometries depending on the sample type and volume. All gamma counts are obtained with germanium type detectors interfaced with a

computer based multichannel analyzer system. Spectral data reduction is performed by the computer program HYPERMET.

The charcoal cartridges used to sample gaseous radioiodine are analyzed by gamma spectroscopy using a high resolution gamma spectroscopy system with germanium detectors.

The necessary efficiency values, weight-efficiency curves, and geometry tables are established and maintained on each detector and counting system. A series of daily and periodic quality control checks are performed to monitor counting instrumentation. System logbooks and control charts are used to document the results of the quality control checks.

APPENDIX E

NOMINAL LOWER LIMITS OF DETECTION (LLD)

Appendix E

Nominal Lower Limits of Detection

Sensitive radiation detection devices can produce a signal even when no radioactivity is present in a sample being analyzed. This signal may come from trace amounts of radioactivity in the components of the device, from cosmic rays, from naturally occurring radon gas, or from electronic noise. The signal registered when no activity is present in the sample is called the background.

The point at which the signal is determined to represent radioactivity in the sample is called the critical level. This point is based on statistical analysis of the background readings from any particular device. However, any sample measured over and over in the same device will give different readings, some higher than others. The sample should have a well-defined average reading, but any individual reading may vary from that average. In order to determine the activity present in a sample that will produce a reading above the critical level, additional statistical analysis of the background readings is required. The hypothetical activity calculated from this analysis is called the lower limit of detection (LLD). A listing of typical LLD values that a laboratory publishes is a guide to the sensitivity of the analytical measurements performed by the laboratory.

Every time an activity is calculated from a sample, the background must be subtracted from the sample signal. For the very low levels encountered in environmental monitoring, the sample signals are often very close to the background. The measuring equipment is being used at the limit of its capability. For a sample with no measurable activity, which often happens, about half the time its signal should fall below the average machine background and half the time it should be above the background. If a signal above the background is present, the calculated activity is compared to the calculated LLD to determine if there is really activity present or if the number is an artifact of the way radioactivity is measured.

A number of factors influence the LLD, including sample size, count time, counting efficiency, chemical processes, radioactive decay factors, and interfering isotopes encountered in the sample. The most likely values for these factors have been evaluated for the various analyses performed in the environmental monitoring program. The nominal LLDs calculated from these values, in accordance with the methodology prescribed in the ODCM, are presented in Table E-1. The maximum values for the lower limits of detection specified in the ODCM are shown in Table E-2.

The nominal LLDs are also presented in the data tables. For analyses for which LLDs have not been established, an LLD of zero is assumed in determining if a measured activity is greater than the nominal LLD.

TABLE E-1

Nominal LLD Values
A. Radiochemical Procedures

	Air Filters (pCi/m^3)	Water (pCi/L)	Milk (pCi/L)	Wet Vegetation (pCi/Kg wet)	Sediment and Soil (pCi/g dry)
Gross Beta	0.002	1.9			
Tritium		300			
Iodine-131		0.4	0.4	6.0	
Strontium-89		5.0	3.5	31.0	1.6
Strontium-90		2.0	2.0	12.0	0.4

Table E-1
Nominal LLD Values
B. Gamma Analyses

	Air Particulates pCi/m3	Charcoal Filter pCi/m3	Water and Milk pCi/L	Vegetation and Grain pCi/g, dry	Wet Vegetation pCi/kg, wet	Soil and Sediment pCi/g, dry	Fish pCi/g, dry	Clam Flesh pCi/g, dry	Foods Tomatoes Potatoes, etc. pCi/kg, wet
Ce-141	.005	.02	10	.07	35	.10	.07	.35	20
Ce-144	.01	.07	30	.15	115	.20	.15	.85	60
Cr-51	.02	0.15	45	.30	200	.35	.30	2.4	95
I-131	.005	0.03	10	.20	60	.25	.20	1.7	20
Ru-103	.005	0.02	5	.03	25	.03	.03	.25	25
Ru-106	.02	0.12	40	.15	190	.20	.15	1.25	90
Cs-134	.005	0.02	5	.03	30	.03	.03	.14	10
Cs-137	.005	0.02	5	.03	25	.03	.03	.15	10
Zr-95	.005	0.03	10	.05	45	.05	.05	.45	45
Nb-95	.005	0.02	5	.25	30	.04	.25	.25	10
Co-58	.005	0.02	5	.03	20	.03	.03	.25	10
Mn-54	.005	0.02	5	.03	20	.03	.03	.20	10
Zn-65	.005	0.03	10	.05	45	.05	.05	.40	45
Co-60	.005	0.02	5	.03	20	.03	.03	.20	10
K-40	.04	0.30	100	.40	400	.75	.40	3.50	250
Ba-140	.015	0.07	25	.30	130	.30	.30	2.4	50
La-140	.01	0.04	10	.20	50	.20	.20	1.4	25
Fe-59	.005	0.04	10	.08	40	.05	.08	.45	25
Be-7	.02	0.15	45	.25	200	.25	.25	1.9	90
Pb-212	.005	0.03	15	.04	40	.10	.04	.30	40
Pb-214	.005	0.07	20	.50	80	.15	.50	.10	80
Bi-214	.005	0.05	20	.10	55	.15	.10	.50	40
Bi-212	.02	0.20	50	.25	250	.45	.25	2.0	130
Tl-208	.002	0.02	10	.03	30	.06	.03	.25	30
Ra-224	--	--	--	--	--	.75	--	--	--
Ra-226	--	--	--	--	--	.15	--	--	--
Ac-228	.01	0.07	20	.10	70	.25	.10	.75	50

Table E-2

Maximum Values for the Lower Limits of Detection (LLD)
Specified by the SQN Offsite Dose Calculation Manual

Analysis	Water pCi/L	Airborne Particulate or Gases pCi/m^3	Fish pCi/kg, wet	Milk pCi/L	Food Products pCi/kg, wet	Sediment pCi/kg, dry
gross beta	4	1×10^{-2}	N.A.	N.A.	N.A.	N.A.
H-3	2000[a]	N.A.	N.A.	N.A.	N.A.	N.A.
Mn-54	15	N.A.	130	N.A.	N.A.	N.A.
Fe-59	30	N.A.	260	N.A.	N.A.	N.A.
Co-58,60	15	N.A.	130	N.A.	N.A.	N.A.
Zn-65	30	N.A.	260	N.A.	N.A.	N.A.
Zr-95	30	N.A.	N.A.	N.A.	N.A.	N.A.
Nb-95	15	N.A.	N.A.	N.A.	N.A.	N.A.
I-131	1[b]	7×10^{-2}	N.A.	1	60	N.A.
Cs-134	15	5×10^{-2}	130	15	60	150
Cs-137	18	6×10^{-2}	150	18	80	180
Ba-140	60	N.A.	N.A.	60	N.A.	N.A.
La-140	15	N.A.	N.A.	15	N.A.	N.A.

a. If no drinking water pathway exists, a value of 3000 pCi/liter may be used.

b. If no drinking water pathway exists, a value of 15 pCi/liter may be used.

APPENDIX F

QUALITY ASSURANCE/QUALITY CONTROL PROGRAM

Appendix F

Quality Assurance/Quality Control Program

A thorough quality assurance program is employed by the laboratory to ensure that the environmental monitoring data are reliable. This program includes the use of written, approved procedures in performing the work, a complete training and retraining system, internal self assessments of program performance, audits by various external organizations, and a laboratory quality control program.

The quality control program employed by the radioanalytical laboratory is designed to ensure that the sampling and analysis process is working as intended. The program includes equipment checks and the analysis of quality control samples along with routine samples.

Radiation detection devices can be tested in a number of ways. There are two primary tests which are performed on all devices. In the first type, the device is operated without a sample on the detector to determine the background count rate. The background counts are usually low values and are due to machine noise, cosmic rays, and/or trace amounts of radioactivity in the materials used to construct the detector. Charts of background counts are kept and monitored to ensure that no unusually high or low values are encountered.

In the second test, the device is operated with a known amount of radioactivity present. The number of counts registered from such a radioactive standard should be very reproducible. These reproducibility checks are also monitored to ensure that they are neither higher nor lower than expected. When counts from either test fall outside the expected range, the device is inspected for malfunction or contamination. It is not returned to service until it is operating properly.

In addition to these two general checks, other quality control checks are performed on the variety of detectors used in the laboratory. The exact nature of these checks depends on the type of device and the method it uses to detect radiation or store the information obtained.

Quality control samples of a variety of types are used by the laboratory to verify the performance of different portions of the analytical process. These quality control samples may be blanks, replicate samples, blind samples, or cross-checks.

Blanks are samples which contain no measurable radioactivity or no activity of the type being measured. Such samples are analyzed to determine whether there is any contamination of equipment or commercial laboratory chemicals, cross-contamination in the chemical process, or interference from isotopes other than the one being measured.

Duplicate samples are generated at random by the sample computer program which schedules the collection of the routine samples. For example, if the routine program calls for four milk samples every week, on a random basis each farm might provide an additional sample several times a year. These duplicate samples are analyzed along with other routine samples. They provide information about the variability of radioactive content in the various sample media.

If enough sample is available for a particular analysis, the laboratory staff can split it into two portions. Such a sample can provide information about the variability of the analytical process since two identical portions of material are analyzed side by side.

Analytical knowns are another category of quality control sample. A known amount of radioactivity is added to a sample medium. The lab staff know the radioactive content of the sample. Whenever possible, the analytical knowns contain the same amount of radioactivity each time they are run. In this way, analytical knowns provide immediate data on the quality of the measurement process. A portion of these samples are also blanks.

Blind spikes are samples containing radioactivity which are introduced into the analysis process disguised as ordinary environmental samples. The lab staff does not know the sample contains radioactivity. Since the bulk of the ordinary workload of the environmental laboratory contains no measurable activity or only naturally occurring radioisotopes, blind spikes can be

used to test the detection capability of the laboratory or can be used to test the data review process. If an analysis routinely generates numerous zeroes for a particular isotope, the presence of the isotope is brought to the attention of the laboratory supervisor in the daily review process. Blind spikes test this process since the blind spikes contain radioactivity at levels high enough to be detected. Furthermore, the activity can be put into such samples at the extreme limit of detection (near the LLD) to determine whether or not the laboratory can find any unusual radioactivity whatsoever.

At present, 5 percent of the laboratory workload is in the category of internal cross-checks. These samples have a known amount of radioactivity added and are presented to the lab staff labeled as cross-check samples. This means that the quality control staff knows the radioactive content or "right answer" but the lab personnel performing the analysis do not. Such samples test the best performance of the laboratory by determining if the lab can find the "right answer". These samples provide information about the accuracy of the measurement process. Further information is available about the variability of the process if multiple analyses are requested on the same sample. Like blind spikes or analytical knowns, these samples can also be spiked with low levels of activity to test detection limits. During 2000, all analysis results for internal cross-check samples were within agreement limits when compared to the known value.

To provide for an independent verification of the laboratory's ability to make accurate measurements, the laboratory participated in an environmental level cross-check program available through Analytics, Inc., during 2000. The results of TVA's participation in this cross-check program are presented in Table F-1.

TVA splits certain environmental samples with laboratories operated by the States of Alabama and Tennessee and the EPA National Air and Radiation Environmental Laboratory in

Montgomery, Alabama. When radioactivity has been present in the environment in measurable quantities, such as following atmospheric nuclear weapons testing, following the Chernobyl incident, or as naturally occurring radionuclides, the split samples have provided TVA with another level of information about laboratory performance. These samples demonstrate performance on actual environmental sample matrices rather than on the constructed matrices used in cross-check programs.

The quality control data are routinely collected, examined and reported to laboratory supervisory personnel. They are checked for trends, problem areas, or other indications that a portion of the analytical process needs correction or improvement. The end result is a measurement process that provides reliable and verifiable data and is sensitive enough to measure the presence of radioactivity far below the levels which could be harmful to humans.

Table F-1

Results For 2000 External Cross Checks

Test Period	Sample Type / Analysis	Results		Agreement Range
		Known	TVA	
First Quarter	Water (pCi/L)			
	Gross Beta	210	197	179 - 242
First Quarter	Charcoal Filter (pCi/Filter)			
	^{131}I	82	66	57 - 107
First Quarter	Water (pCi/L)			
	^{131}I	74	72	52 - 96
	^{141}Ce	427	420	363 - 491
	^{51}Cr	238	230	167 - 309
	^{134}Cs	139	124	118 - 160
	^{137}Cs	128	129	109 - 147
	^{58}Co	44	43	29 - 59
	^{54}Mn	159	163	135 - 183
	^{59}Fe	92	92	77 - 107
	^{65}Zn	196	198	137 - 255
	^{60}Co	116	119	99 - 133
First Quarter	Water (pCi/L)			
	^{89}Sr	94	82	79 - 109
	^{90}Sr	51	54	36 - 66
Third Quarter	Water (pCi/L)			
	^{3}H	8947	9172	6263 - 11631
Third Quarter	Sand (pCi/g)			
	^{141}Ce	0.256	0.235	0.218 - 0.294
	^{51}Cr	0.309	0.308	0.216 - 0.402
	^{134}Cs	0.172	0.183	0.146 - 0.198
	^{137}Cs	0.293	0.266	0.249 - 0.337
	^{58}Co	0.080	0.076	0.068 - 0.092
	^{54}Mn	0.120	0.129	0.102 - 0.138
	^{59}Fe	0.073	0.080	0.062 - 0.084
	^{65}Zn	0.180	0.170	0.126 - 0.234
	^{60}Co	0.331	0.313	0.281 - 0.381
Third Quarter	Air Filter (pCi/Filter)			
	Gross Beta	62.7	56.0	47.7 - 77.7
Third Quarter	Air Filter (pCi/Filter)			
	^{141}Ce	106.0	103.2	90.1 - 121.9
	^{51}Cr	128.0	116.0	89.6 - 166.4
	^{134}Cs	71.0	60.6	56.0 - 86.0
	^{137}Cs	121.0	116.0	102.9 - 139.2
	^{58}Co	33.0	32.5	18.0 - 48.0
	^{54}Mn	50.0	53.7	35.0 - 65.0
	^{59}Fe	30.0	30.5	15.0 - 45.0
	^{65}Zn	74.0	75.5	51.8 - 96.2
	^{60}Co	137.0	140.4	116.5 - 157.6

APPENDIX G

LAND USE SURVEY

Appendix G

Land Use Survey

A land use survey is conducted annually to identify the location of the nearest milk producing animal, the nearest residence, and the nearest garden of greater than 500 square feet producing fresh leafy vegetables in each of 16 meteorological sectors within a distance of 5 miles from the plant.

The land use survey is conducted between April 1 and October 1 using appropriate techniques such as door-to-door survey, mail survey, telephone survey, aerial survey, or information from local agricultural authorities or other reliable sources.

In order to identify the locations around SQN which have the greatest relative potential for impact by the plant, radiation doses are projected for individuals living near SQN. These projections use the data obtained in the survey and historical meteorological data. They also assume that releases are equivalent to the design basis source terms. The calculated doses are relative in nature and do not reflect actual exposures received by individuals living near SQN. Calculated doses to individuals based on measured effluents from the plant are well below applicable dose limits (see Assessment and Evaluation Section and Table 3).

In response to the 2000 SQN land use survey, annual dose projections were calculated for air submersion, vegetable ingestion, and milk ingestion. External doses due to radioactivity in air (air submersion) are calculated for the nearest resident in each sector, while doses from drinking milk or eating foods produced near the plant are calculated for the areas with milk producing animals and gardens, respectively.

There were two changes in the location of the nearest resident as identified in 2000 compared to 1999. There were two changes in the location for nearest gardens as identified in 2000.

For milk ingestion, projected doses were consistent with those calculated for 1999, except for small variances due to change in the feeding factor values at some locations. The farm in the WNW sector was not producing milk in 2000.

Tables G-1, G-2, and G-3 show the comparative relative calculated doses for 1999 and 2000.

Table G-1

SEQUOYAH NUCLEAR PLANT

Relative Projected Annual Air Submersion Dose to the Nearest Resident
Within Five Miles of Plant
mrem/year

	1999 Survey		2000 Survey	
Sector	Approximate Distance Miles	Annual Dose	Approximate Distance Miles	Annual Dose
N	0.8	0.12	0.8	0.12
NNE	1.5	0.07	1.5	0.07
NE	1.5	0.06	1.5	0.06
ENE	1.3	0.02	1.3	0.02
E	1.0	0.02	1.0	0.02
ESE	1.0	0.02	1.0	0.02
SE	1.1	0.02	1.1	0.02
SSE	1.3	0.03	1.3	0.03
S	1.2	0.09	1.1	0.11
SSW	1.3	0.15	1.3	0.15
SW	1.4	0.06	1.4	0.06
WSW	0.6	0.05	0.6	0.05
W	0.6	0.06	0.6	0.06
WNW	0.9	0.02	1.1	0.02
NW	0.8	0.04	0.8	0.04
NNW	0.5	0.14	0.5	0.14

Table G-2

SEQUOYAH NUCLEAR PLANT

Relative Projected Annual Dose to Child's Bone from
Ingestion of Home-Grown Foods
mrem/year

	1999 Survey		2000 Survey	
Sector	Approximate Distance Miles	Annual Dose	Approximate Distance Miles	Annual Dose
N	1.1	2.25	1.1	2.25
NNE	1.6	2.10	2.1	1.42
NE	2.7	0.78	2.7	0.78
ENE	2.7	0.26	3.4	0.20
E	1.9	0.27	1.9	0.27
ESE	1.5	0.31	1.5	0.31
SE	2.0	0.30	2.0	0.30
SSE	1.3	1.00	1.3	1.00
S	2.0	1.36	2.0	1.36
SSW	1.7	3.50	1.7	3.50
SW	2.4	1.02	2.4	1.02
WSW	0.7	1.32	0.7	1.32
W	1.2	0.63	1.2	0.63
WNW	1.1	0.62	1.1	0.62
NW	0.9	1.26	0.9	1.26
NNW	0.5	4.26	0.5	4.26

Table G-3

SEQUOYAH NUCLEAR PLANT

Relative Projected Annual Dose to Receptor Thyroid
from Ingestion of Milk
mrem/year

Location	Sector	Approximate Distance (Miles)[a]	Annual Dose 1999	Annual Dose 2000	XQ s/m^3
Farm HS[b]	E	4.6	0.008	0.008	6.74 E-8
Farm JH[b]	ESE	3.9	0.004	0.004	6.79 E-8
Farm J	WNW	1.1	0.033	c	3.99 E-7
Farm HW	NW	1.2	0.045	0.050	5.48 E-7

a. Distances measured to nearest property line.
b. Grade A dairy.
c. No milk production at this location in 2000.

APPENDIX H

DATA TABLES AND FIGURES

Table H-1

DIRECT RADIATION LEVELS

Average External Gamma Reduction Levels at Various Distances from Sequoyah Nuclear Plant for Each Quarter-2000
mR / Quarter (a)

Distance Miles	Average External Gamma Radiation Levels (b)				per annum mR/yr
	1st qtr	2nd qtr	3rd qtr	4th qtr	
0 - 1	16.8 ± 2.7	16.0 ± 1.6	15.7 ± 1.7	15.8 ± 2.2	63
1 - 2	13.9 ± 1.7	13.4 ± 1.8	13.1 ± 1.9	13.2 ± 1.8	53
2 - 4	14.0 ± 2.3	13.5 ± 2.3	13.1 ± 2.3	13.0 ± 2.2	53
4 - 6	13.8 ± 1.5	13.6 ± 1.7	13.3 ± 1.9	13.0 ± 1.6	53
> 6	13.6 ± 1.3	13.7 ± 1.4	13.3 ± 1.5	13.0 ± 1.2	53
Average, 0 - 2 miles (onsite)	15.5 ± 2.7	14.8 ± 2.1	14.5 ± 2.2	14.6 ± 2.4	59
Average, >2 miles	13.8 ± 1.7	13.6 ± 1.8	13.3 ± 1.9	13.0 ± 1.7	54

(a) Field periods normalized to one standard quarter (2190 hours)
(b) Average of the individual measurements in the set ± 1 standard deviation of the set

TABLE H - 2

DIRECT RADIATION LEVELS

Individual Stations at Sequoyah Nuclear Plant

				Environmental Radiation Levels mR / quarter				
Map Location Number	TLD Station Number	Direction, degrees	Approx Distance, miles	1st Qtr Feb - Apr 2000	2nd Qtr May - Jul 2000	3rd Qtr Aug - Oct 2000	4th Qtr Nov - Jan 2000,01	Annual Exposure mR/year
49	N-1	3	.6	15.0	13.8	15.3	15.6	59.7
50	N-2	4	2.1	14.6	14.0	13.6	13.3	55.5
51	N-3	358	5.2	12.4	11.7	11.4	10.8	46.4
52	N-4	355	10.0	13.1	13.6	13.4	12.3	52.4
5	NNE-1	13	1.8	16.7	15.6	15.2	15.8	63.4
53	NNE-2	31	4.5	13.1	13.0	12.1	12.1	50.2
54	NNE-3	32	12.1	13.6	13.0	13.3	12.8	52.6
12	NNE-4	32	17.8	13.1	12.9	13.4	12.3	51.7
55	NE-1	38	2.4	15.1	13.9	13.7	13.7	56.3
4	NE-1A	50	1.5	15.2	14.6	13.9	14.2	57.9
56	NE-2	51	4.1	12.2	11.6	10.6	11.4	45.8
57	ENE-1	73	.4	13.8	13.2	12.6	12.4	52.0
58	ENE-2	66	5.1	13.1	13.6	12.8	12.9	52.4
59	E-1	96	1.2	12.7	12.3	12.0	12.3	49.2
60	E-2	87	5.2	13.0	12.9	12.2	12.2	50.3
65	E-A	91	.3	17.2	16.6	16.2	16.3	66.3
62	ESE-1	110	1.2	13.2	13.1	12.8	13.1	52.2
63	ESE-2	112	4.9	15.1	16.0	16.0	14.9	62.0
13	ESE-3	117	11.3	13.6	14.1	13.6	13.3	54.6
61	ESE-A	110	.3	24.4	18.2	16.5	21.3	80.5
66	SE-1	131	1.4	11.6	10.2	10.2	10.1	42.0
67	SE-2	129	1.9	13.2	12.9	12.1	12.4	50.6
68	SE-4	136	5.2	17.3	16.7	16.2	16.0	66.2
64	SE-A	132	.4	16.1	15.0	13.8	14.2	59.1
69	SSE-1	154	1.6	12.0	11.8	11.3	11.6	46.7
70	SSE-2	158	4.6	15.6	15.4	16.3	15.2	62.5

TABLE H - 2 continued

DIRECT RADIATION LEVELS

Individual Stations at Sequoyah Nuclear Plant

				Environmental Radiation Levels mR / quarter				
Map Location Number	TLD Station Number	Direction, degrees	Approx Distance, miles	1st Qtr Jan - Mar 2000	2nd Qtr Apr - Jun 2000	3rd Qtr Jul - Sep 2000	4th Qtr Oct - Dec 2000, 01	Annual Exposure mR/year
71	S-1	183	1.5	16.7	16.5	16.9	16.2	66.2
72	S-2	185	4.7	12.0	11.1	11.6	11.0	45.7
73	SSW-1	203	.6	15.8	15.7	15.5	15.0	62.2
90	SSW-1B	192	1.5	14.5	13.9	13.8	13.5	55.7
3	SSW-1C	198	2.0	15.7	15.0	14.7	14.7	60.1
74	SSW-2	204	4.0	17.6	17.6	17.4	16.8	69.4
9	SSW-3	203	8.7	15.6	15.4	15.5	14.5	61.1
75	SW-1	228	.9	15.7	16.1	15.8	15.6	63.2
7	SW-2	227	3.8	12.4	12.4	11.6	11.7	48.2
11	SW-3	228	16.7	15.9	16.3	15.7	15.6	63.5
76	WSW-1	241	.9	16.4	16.5	16.6	15.6	65.1
77	WSW-2	238	2.5	11.0	10.4	10.0	10.0	41.4
10	WSW-2A	250	2.6	15.6	15.4	15.1	14.6	60.5
78	WSW-3	248	5.7	12.5	11.9	11.6	12.1	48.1
79	WSW-4	244	7.8	13.5	12.7	12.4	12.8	51.4
80	WSW-5	244	10.1	18.2	18.3	18.4	17.5	72.4
81	W-1	260	.8	12.2	12.0	11.7	11.3	47.1
82	W-2	275	4.3	11.6	11.2	10.7	11.0	44.5
8	W-3	280	5.6	14.8	15.3	14.7	14.6	59.4
83	WNW-1	292	.4	14.8	15.0	14.1	14.2	58.1
84	WNW-2	295	5.3	13.5	13.5	13.6	12.9	53.5
14	WNW-3	299	18.9	11.6	(1)	10.9	11.6	45.4
85	NW-1	315	.4	19.1	18.5	18.4	17.9	73.9
86	NW-2	318	5.2	14.2	13.9	14.1	13.8	56.0
87	NNW-1	344	.6	15.4	15.4	14.6	14.4	59.9
88	NNW-2	342	1.7	13.5	13.3	12.4	12.5	51.7
89	NNW-3	334	5.3	12.6	11.8	11.8	11.7	47.8

note (1) Sum of available quarterly data normalized to 1 year for the annual exposure value

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN AIR FILTER
PCI/M3 - 0.037 BQ/M3

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GROSS BETA						
624	2.00E-03	2.22E-02(419/ 419)	LM-3 HARRISON BAY RD	2.28E-02(53/ 53)	2.26E-02(205/ 205)	
		9.27E-03- 4.91E-02	2.0 MILES SSW	9.27E-03- 4.68E-02	9.00E-03- 5.03E-02	
GAMMA SCAN (GELI)						
167						
BE-7	2.00E-02	9.84E-02(112/ 112)	LM-3 HARRISON BAY RD	1.01E-01(14/ 14)	9.95E-02(55/ 55)	
		5.48E-02- 1.41E-01	2.0 MILES SSW	5.58E-02- 1.39E-01	5.30E-02- 1.42E-01	
BI-214	5.00E-03	2.42E-02(72/ 112)	LM-4 SKULL ISLAND	3.61E-02(7/ 14)	1.87E-02(42/ 55)	
		5.50E-03- 1.45E-01	1.5 MILES NE	6.00E-03- 1.41E-01	5.00E-03- 9.95E-02	
K-40	4.00E-02	112 VALUES < LLD	LM-2 NORTH	14 VALUES < LLD	5.89E-02(2/ 55)	
			0.8 MILES NORTH		4.87E-02- 6.92E-02	
PB-214	5.00E-03	2.23E-02(72/ 112)	LM-4 SKULL ISLAND	3.50E-02(7/ 14)	1.70E-02(42/ 55)	
		5.10E-03- 1.27E-01	1.5 MILES NE	5.30E-03- 1.27E-01	5.40E-03- 8.69E-02	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN CHARCOAL FILTER
PCI/M3 - 0.037 BQ/M3

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GAMMA SCAN (GELI) 624						
BI-214	5.00E-02	7.34E-02(44/ 419)	PM-9 LAKESIDE	8.17E-02(3/ 52)	7.00E-02(25/ 205)	
		5.05E-02- 1.57E-01	2.6 MILES WSW	5.80E-02- 1.16E-01	5.01E-02- 1.33E-01	
K-40	3.00E-01	4.26E-01(8/ 419)	PM-9 LAKESIDE	6.96E-01(1/ 52)	3.07E-01(1/ 205)	
		3.06E-01- 6.96E-01	2.6 MILES WSW	6.96E-01- 6.96E-01	3.07E-01- 3.07E-01	
PB-214	7.00E-02	1.05E-01(26/ 419)	PM-3 DAISY TN	1.28E-01(3/ 52)	1.01E-01(17/ 205)	
		7.08E-02- 1.85E-01	5.6 MILES W	7.66E-02- 1.85E-01	7.04E-02- 1.79E-01	
I-131	SEE NOTE 3					

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).
NOTE: 3. THE ANALYSIS OF CHARCOAL FILTERS WAS PERFORMED BY GAMMA SPECTROSCOPY. NO I-131 WAS DETECTED. THE LLD FOR I-131 BY GAMMA SPECTROSCOPY WAS 0.03 pCi/cubic meter.

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN MILK
PCI/L - 0.037 BQ/L

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
IODINE-131						
120	4.00E-01	64 VALUES < LLD			56 VALUES < LLD	
GAMMA SCAN (GELI)						
120						
BI-214	2.00E+01	2.70E+01(2/ 64)	JOHNSON FARM	2.70E+01(2/ 24)	2.77E+01(4/ 56)	
		2.41E+01- 3.00E+01	3.9 MILES ESE	2.41E+01- 3.00E+01	2.40E+01- 3.06E+01	
K-40	1.00E+02	1.37E+03(64/ 64)	H WALKER FARM	1.39E+03(19/ 19)	1.36E+03(56/ 56)	
		8.12E+02- 1.71E+03	1.2 MILES NW	8.12E+02- 1.71E+03	1.24E+03- 1.51E+03	
PB-214	2.00E+01	2.22E+01(1/ 64)	H WALKER FARM	2.22E+01(1/ 19)	2.13E+01(2/ 56)	
		2.22E+01- 2.22E+01	1.2 MILES NW	2.22E+01- 2.22E+01	2.12E+01- 2.15E+01	
SR 89						
18	3.50E+00	10 VALUES < LLD			8 VALUES < LLD	
SR 90						
18						
	2.00E+00	3.05E+00(1/ 10)	H WALKER FARM	3.05E+00(1/ 3)	8 VALUES < LLD	
		3.05E+00- 3.05E+00	1.2 MILES NW	3.05E+00- 3.05E+00		

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN VEGETATION
PCI/KG - 0.037 BQ/KG (WET WEIGHT)

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
IODINE-131						
26	6.00E+00	13 VALUES < LLD			13 VALUES < LLD	
GAMMA SCAN (GELI)						
26						
BE-7	2.00E+02	1.06E+03(13/ 13)	EDGAR MALONE FARM	1.06E+03(13/ 13)	9.64E+02(13/ 13)	
		3.44E+02- 2.29E+03	2.6 MILES N	3.44E+02- 2.29E+03	2.79E+02- 2.40E+03	
BI-214	5.50E+01	9.98E+01(7/ 13)	EDGAR MALONE FARM	9.98E+01(7/ 13)	9.98E+01(6/ 13)	
		5.70E+01- 1.55E+02	2.6 MILES N	5.70E+01- 1.55E+02	6.79E+01- 2.24E+02	
K-40	4.00E+02	5.39E+03(13/ 13)	EDGAR MALONE FARM	5.39E+03(13/ 13)	5.13E+03(13/ 13)	
		3.78E+03- 6.43E+03	2.6 MILES N	3.78E+03- 6.43E+03	3.48E+03- 6.74E+03	
PB-214	8.00E+01	1.05E+02(3/ 13)	EDGAR MALONE FARM	1.05E+02(3/ 13)	1.21E+02(2/ 13)	
		8.57E+01- 1.28E+02	2.6 MILES N	8.57E+01- 1.28E+02	8.66E+01- 1.55E+02	
SR 89						
8	3.10E+01	4 VALUES < LLD			4 VALUES < LLD	
SR 90						
8						
	1.20E+01	1.85E+01(2/ 4)	EDGAR MALONE FARM	1.85E+01(2/ 4)	1.77E+01(3/ 4)	
		1.30E+01- 2.39E+01	2.6 MILES N	1.30E+01- 2.39E+01	1.48E+01- 2.07E+01	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN SOIL
PCI/GM - 0.037 BQ/G (DRY WEIGHT)

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GAMMA SCAN (GELI)						
12						
AC-228	2.50E-01	1.10E+00(8/ 8)	PM-8 HARRISON TN	1.55E+00(1/ 1)	9.66E-01(4/ 4)	
		6.31E-01- 1.55E+00	8.7 MILES SSW	1.55E+00- 1.55E+00	4.81E-01- 1.43E+00	
BI-212	4.50E-01	1.06E+00(8/ 8)	LM-5 WARE POINT	1.50E+00(1/ 1)	9.55E-01(4/ 4)	
		6.98E-01- 1.50E+00	1.8 MILES NNE	1.50E+00- 1.50E+00	5.52E-01- 1.43E+00	
BI-214	1.50E-01	8.94E-01(8/ 8)	PM-8 HARRISON TN	1.25E+00(1/ 1)	6.71E-01(4/ 4)	
		6.91E-01- 1.25E+00	8.7 MILES SSW	1.25E+00- 1.25E+00	5.52E-01- 8.66E-01	
CS-137	3.00E-02	4.40E-01(7/ 8)	PM-2 COUNTY PARK TN	7.31E-01(1/ 1)	2.40E-01(4/ 4)	
		8.63E-02- 7.31E-01	3.8 MILES SW	7.31E-01- 7.31E-01	4.24E-02- 5.41E-01	
K-40	7.50E-01	5.98E+00(8/ 8)	LM-2 NORTH	1.24E+01(1/ 1)	8.60E+00(4/ 4)	
		2.72E+00- 1.24E+01	0.8 MILES NORTH	1.24E+01- 1.24E+01	2.81E+00- 2.12E+01	
PB-212	1.00E-01	1.02E+00(8/ 8)	PM-8 HARRISON TN	1.50E+00(1/ 1)	9.32E-01(4/ 4)	
		6.21E-01- 1.50E+00	8.7 MILES SSW	1.50E+00- 1.50E+00	4.53E-01- 1.45E+00	
PB-214	1.50E-01	1.00E+00(8/ 8)	PM-8 HARRISON TN	1.38E+00(1/ 1)	7.52E-01(4/ 4)	
		7.81E-01- 1.38E+00	8.7 MILES SSW	1.38E+00- 1.38E+00	6.07E-01- 9.46E-01	
RA-224	7.50E-01	1.32E+00(6/ 8)	PM-8 HARRISON TN	1.96E+00(1/ 1)	1.40E+00(2/ 4)	
		7.81E-01- 1.96E+00	8.7 MILES SSW	1.96E+00- 1.96E+00	1.29E+00- 1.52E+00	
RA-226	1.50E-01	8.94E-01(8/ 8)	PM-8 HARRISON TN	1.25E+00(1/ 1)	6.71E-01(4/ 4)	
		6.91E-01- 1.25E+00	8.7 MILES SSW	1.25E+00- 1.25E+00	5.52E-01- 8.66E-01	
TL-208	6.00E-02	3.43E-01(8/ 8)	PM-8 HARRISON TN	4.73E-01(1/ 1)	2.96E-01(4/ 4)	
		1.94E-01- 4.73E-01	8.7 MILES SSW	4.73E-01- 4.73E-01	1.52E-01- 4.43E-01	
SR 89						
12						
	1.60E+00	8 VALUES < LLD			4 VALUES < LLD	
SR 90						
12						
	4.00E-01	8 VALUES < LLD			4 VALUES < LLD	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN APPLES
PCI/KG - 0.037 BQ/KG (WET WT)

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GAMMA SCAN (GELI)						
K-40	2.50E+02	9.25E+02(1/ 1) 9.25E+02- 9.25E+02	JONES FARM 1.1 MILES WNW	9.25E+02(1/ 1) 9.25E+02- 9.25E+02	8.36E+02(1/ 1) 8.36E+02- 8.36E+02	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN CABBAGE
PCI/KG - 0.037 BQ/KG (WET WT)

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GAMMA SCAN (GELI)						
K-40	2.50E+02	1.41E+03(1/ 1) [2] 1.41E+03- 1.41E+03	1 MILES NW	1.41E+03(1/ 1) 1.41E+03- 1.41E+03	1.34E+03(1/ 1) 1.34E+03- 1.34E+03	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN CORN
PCI/KG - 0.037 BQ/KG (WET WT)

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GAMMA SCAN (GELI) 2						
K-40	2.50E+02	2.13E+03(1/ 1) 2.13E+03- 2.13E+03	1 MILES NW	2.13E+03(1/ 1) 2.13E+03- 2.13E+03	2.00E+03(1/ 1) 2.00E+03- 2.00E+03	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN GREEN BEANS
PCI/KG - 0.037 BQ/KG (WET WT)

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GAMMA SCAN (GELI)						
K-40	2.50E+02	1.49E+03(1/ 1) [2] 1.49E+03- 1.49E+03	1 MILES NW	1.49E+03(1/ 1) 1.49E+03- 1.49E+03	1.62E+03(1/ 1) 1.62E+03- 1.62E+03	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN POTATOES
PCI/KG - 0.037 BQ/KG (WET WT)

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GAMMA SCAN (GELI)						
K-40	2.50E+02	2.81E+03(1/ 1) [2] 2.81E+03- 2.81E+03	1 MILES NW	2.81E+03(1/ 1) 2.81E+03- 2.81E+03	3.60E+03(1/ 1) 3.60E+03- 3.60E+03	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN TOMATOES
PCI/KG - 0.037 BQ/KG (WET WT)

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GAMMA SCAN (GELI) 2						
K-40	2.50E+02	1.77E+03(1/ 1) 1.77E+03- 1.77E+03	1 MILES NW	1.77E+03(1/ 1) 1.77E+03- 1.77E+03	2.16E+03(1/ 1) 2.16E+03- 2.16E+03	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN SURFACE WATER(Total)
PCI/L - 0.037 BQ/L

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT DOCKET NO.: 50-327,328

LOCATION OF FACILITY: HAMILTON TENNESSEE REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GROSS BETA						
26	1.90E+00	2.94E+00(12/ 13) 2.32E+00- 3.38E+00	TRM 483.4	2.94E+00(12/ 13) 2.32E+00- 3.38E+00	3.35E+00(12/ 13) 2.61E+00- 4.35E+00	
GAMMA SCAN (GELI)						
26	5.00E+00	13 VALUES < LLD			13 VALUES < LLD	
SR 89						
8	5.00E+00	4 VALUES < LLD			4 VALUES < LLD	
SR 90						
8	2.00E+00	4 VALUES < LLD			4 VALUES < LLD	
TRITIUM						
8	3.00E+02	4 VALUES < LLD			4 VALUES < LLD	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN PUBLIC WATER(Total)
PCI/L - 0.037 BQ/L

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GROSS BETA						
50	1.90E+00	2.84E+00(24/ 37)	CF INDUSTRIES	3.22E+00(9/ 11)	3.35E+00(12/ 13)	
		2.01E+00- 3.88E+00	TRM 473.0	2.49E+00- 3.88E+00	2.61E+00- 4.35E+00	
GAMMA SCAN (GELI)						
50						
BI-214	2.00E+01	3.87E+01(4/ 37)	CHATTANOOGA	4.74E+01(2/ 13)	13 VALUES < LLD	
		2.86E+01- 5.52E+01	TRM 465.3	3.97E+01- 5.52E+01		
PB-214	2.00E+01	2.24E+01(3/ 37)	E.I. DUPONT	2.51E+01(1/ 13)	13 VALUES < LLD	
		2.03E+01- 2.51E+01	TRM 470.5	2.51E+01- 2.51E+01		
SR 89						
16	5.00E+00	12 VALUES < LLD			4 VALUES < LLD	
SR 90						
16	2.00E+00	12 VALUES < LLD			4 VALUES < LLD	
TRITIUM						
16	3.00E+02	12 VALUES < LLD			4 VALUES < LLD	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN WELL WATER(Total)
PCI/L - 0.037 BQ/L

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GROSS BETA						
8	1.90E+00	2.79E+00(4/ 4)	SQN WELL #6	2.79E+00(4/ 4)	7.68E+00(4/ 4)	
		2.45E+00- 3.17E+00	ONSITE NNE	2.45E+00- 3.17E+00	3.75E+00- 1.19E+01	
GAMMA SCAN (GELI)						
8						
BI-214	2.00E+01	2.30E+01(1/ 4)	SQN WELL #6	2.30E+01(1/ 4)	2.96E+02(4/ 4)	
		2.30E+01- 2.30E+01	ONSITE NNE	2.30E+01- 2.30E+01	1.52E+02- 4.24E+02	
PB-214	2.00E+01	4 VALUES < LLD	SQN WELL #6	4 VALUES < LLD	3.10E+02(4/ 4)	
			ONSITE NNE		1.42E+02- 4.54E+02	
SR 89						
8	5.00E+00	4 VALUES < LLD			4 VALUES < LLD	
SR 90						
8	2.00E+00	4 VALUES < LLD			4 VALUES < LLD	
TRITIUM						
8	3.00E+02	6.93E+02(4/ 4)	SQN WELL #6	6.93E+02(4/ 4)	4 VALUES < LLD	
		5.45E+02- 8.78E+02	ONSITE NNE	5.45E+02- 8.78E+02		

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN CHANNEL CATFISH FLESH
PCI/GM - 0.037 BQ/G (DRY WEIGHT)

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GAMMA SCAN (GELI) 4						
BI-214	1.00E-01	1.25E-01(1/ 2)	CHICKAMAUGA RES	1.25E-01(1/ 2)	1.79E-01(1/ 2)	
		1.25E-01- 1.25E-01	TRM 471-530	1.25E-01- 1.25E-01	1.79E-01- 1.79E-01	
CS-137	3.00E-02	2 VALUES < LLD	CHICKAMAUGA RES	2 VALUES < LLD	4.96E-02(2/ 2)	
			TRM 471-530		4.59E-02- 5.32E-02	
K-40	4.00E-01	9.55E+00(2/ 2)	CHICKAMAUGA RES	9.55E+00(2/ 2)	1.43E+01(2/ 2)	
		9.06E+00- 1.00E+01	TRM 471-530	9.06E+00- 1.00E+01	1.07E+01- 1.79E+01	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN CRAPPIE FLESH
PCI/GM - 0.037 BQ/G (DRY WEIGHT)

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GAMMA SCAN (GELI) 4						
BI-214	1.00E-01	2.37E-01(1/ 2) 2.37E-01- 2.37E-01	CHICKAMAUGA RES TRM 471-530	2.37E-01(1/ 2) 2.37E-01- 2.37E-01	1.44E-01(1/ 2) 1.44E-01- 1.44E-01	
CS-137	3.00E-02	2 VALUES < LLD	CHICKAMAUGA RES TRM 471-530	2 VALUES < LLD	5.58E-02(2/ 2) 5.56E-02- 5.60E-02	
K-40	4.00E-01	1.62E+01(2/ 2) 1.46E+01- 1.77E+01	CHICKAMAUGA RES TRM 471-530	1.62E+01(2/ 2) 1.46E+01- 1.77E+01	1.38E+01(2/ 2) 1.35E+01- 1.40E+01	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN SMALLMOUTH BUFFALO FLESH
PCI/GM - 0.037 BQ/G (DRY WEIGHT)

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GAMMA SCAN (GELI) 4						
BI-214	1.00E-01	1.59E-01(1/ 2) 1.59E-01- 1.59E-01	CHICKAMAUGA RES TRM 471-530	1.59E-01(1/ 2) 1.59E-01- 1.59E-01	2.24E-01(1/ 2) 2.24E-01- 2.24E-01	
CS-137	3.00E-02	2 VALUES < LLD	CHICKAMAUGA RES TRM 471-530	2 VALUES < LLD	4.24E-02(1/ 2) 4.24E-02- 4.24E-02	
K-40	4.00E-01	8.53E+00(2/ 2) 7.98E+00- 9.08E+00	CHICKAMAUGA RES TRM 471-530	8.53E+00(2/ 2) 7.98E+00- 9.08E+00	1.02E+01(2/ 2) 9.29E+00- 1.11E+01	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN SEDIMENT
PCI/GM - 0.037 BQ/G (DRY WEIGHT)

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GAMMA SCAN (GELI) 6						
AC-228	2.50E-01	1.66E+00(4/ 4)	TRM 480.8	1.76E+00(2/ 2)	1.36E+00(2/ 2)	
		1.42E+00- 1.81E+00		1.70E+00- 1.81E+00	1.18E+00- 1.55E+00	
BE-7	2.50E-01	3.59E-01(2/ 4)	TRM 480.8	3.59E-01(2/ 2)	4.95E-01(1/ 2)	
		3.17E-01- 4.01E-01		3.17E-01- 4.01E-01	4.95E-01- 4.95E-01	
BI-212	4.50E-01	1.61E+00(4/ 4)	TRM 480.8	1.67E+00(2/ 2)	1.46E+00(2/ 2)	
		1.41E+00- 1.69E+00		1.65E+00- 1.69E+00	1.21E+00- 1.71E+00	
BI-214	1.50E-01	1.23E+00(4/ 4)	TRM 480.8	1.29E+00(2/ 2)	1.15E+00(2/ 2)	
		1.01E+00- 1.32E+00		1.26E+00- 1.32E+00	9.57E-01- 1.35E+00	
CO-60	3.00E-02	1.40E-01(2/ 4)	TRM 480.8	1.40E-01(2/ 2)	2 VALUES < LLD	
		1.16E-01- 1.64E-01		1.16E-01- 1.64E-01		
CS-137	3.00E-02	6.43E-01(4/ 4)	TRM 480.8	7.94E-01(2/ 2)	5.61E-01(2/ 2)	
		6.89E-02- 9.15E-01		7.57E-01- 8.32E-01	4.86E-01- 6.36E-01	
K-40	7.50E-01	1.53E+01(4/ 4)	TRM 480.8	1.60E+01(2/ 2)	1.58E+01(2/ 2)	
		1.25E+01- 1.67E+01		1.59E+01- 1.61E+01	1.51E+01- 1.66E+01	
PB-212	1.00E-01	1.60E+00(4/ 4)	TRM 480.8	1.68E+00(2/ 2)	1.40E+00(2/ 2)	
		1.35E+00- 1.70E+00		1.66E+00- 1.70E+00	1.19E+00- 1.61E+00	
PB-214	1.50E-01	1.37E+00(4/ 4)	TRM 480.8	1.45E+00(2/ 2)	1.24E+00(2/ 2)	
		1.16E+00- 1.48E+00		1.42E+00- 1.48E+00	1.05E+00- 1.42E+00	
RA-224	7.50E-01	1.72E+00(3/ 4)	TRM 480.8	1.78E+00(2/ 2)	1.72E+00(1/ 2)	
		1.62E+00- 1.84E+00		1.71E+00- 1.84E+00	1.72E+00- 1.72E+00	
RA-226	1.50E-01	1.23E+00(4/ 4)	TRM 480.8	1.29E+00(2/ 2)	1.15E+00(2/ 2)	
		1.01E+00- 1.32E+00		1.26E+00- 1.32E+00	9.57E-01- 1.35E+00	
TL-208	6.00E-02	5.14E-01(4/ 4)	TRM 480.8	5.25E-01(2/ 2)	4.40E-01(2/ 2)	
		4.52E-01- 5.53E-01		5.22E-01- 5.27E-01	4.00E-01- 4.81E-01	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN SHORELINE SEDIMENT
PCI/GM - 0.037 BQ/G (DRY WEIGHT)

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST ANNUAL MEAN NAME DISTANCE AND DIRECTION	MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GAMMA SCAN (GELI) 6						
AC-228	2.50E-01	1.42E+00(4/ 4)	TRM 479.0	1.84E+00(2/ 2)	1.48E+00(2/ 2)	
		7.30E-01- 2.00E+00	TRM 479.0	1.69E+00- 2.00E+00	1.31E+00- 1.65E+00	
BE-7	2.50E-01	3.81E-01(1/ 4)	TRM 480.0	3.81E-01(1/ 2)	2 VALUES < LLD	
		3.81E-01- 3.81E-01	TRM 480.0	3.81E-01- 3.81E-01		
BI-212	4.50E-01	1.46E+00(4/ 4)	TRM 479.0	1.87E+00(2/ 2)	1.73E+00(2/ 2)	
		7.34E-01- 2.12E+00	TRM 479.0	1.62E+00- 2.12E+00	1.39E+00- 2.07E+00	
BI-214	1.50E-01	8.01E-01(4/ 4)	TRM 479.0	1.10E+00(2/ 2)	1.04E+00(2/ 2)	
		3.65E-01- 1.17E+00	TRM 479.0	1.03E+00- 1.17E+00	9.65E-01- 1.11E+00	
CS-137	3.00E-02	6.63E-02(2/ 4)	TRM 480.0	6.63E-02(2/ 2)	7.72E-02(1/ 2)	
		5.18E-02- 8.07E-02	TRM 480.0	5.18E-02- 8.07E-02	7.72E-02- 7.72E-02	
K-40	7.50E-01	1.56E+01(4/ 4)	TRM 480.0	2.15E+01(2/ 2)	7.25E+00(2/ 2)	
		9.60E+00- 3.34E+01	TRM 480.0	9.67E+00- 3.34E+01	5.96E+00- 8.55E+00	
PB-212	1.00E-01	1.40E+00(4/ 4)	TRM 479.0	1.86E+00(2/ 2)	1.66E+00(2/ 2)	
		5.72E-01- 2.05E+00	TRM 479.0	1.67E+00- 2.05E+00	1.37E+00- 1.94E+00	
PB-214	1.50E-01	8.99E-01(4/ 4)	TRM 479.0	1.25E+00(2/ 2)	1.14E+00(2/ 2)	
		4.28E-01- 1.28E+00	TRM 479.0	1.22E+00- 1.28E+00	1.02E+00- 1.25E+00	
RA-224	7.50E-01	1.75E+00(2/ 4)	TRM 479.0	2.23E+00(1/ 2)	1.57E+00(1/ 2)	
		1.26E+00- 2.23E+00	TRM 479.0	2.23E+00- 2.23E+00	1.57E+00- 1.57E+00	
RA-226	1.50E-01	8.01E-01(4/ 4)	TRM 479.0	1.10E+00(2/ 2)	1.04E+00(2/ 2)	
		3.65E-01- 1.17E+00	TRM 479.0	1.03E+00- 1.17E+00	9.65E-01- 1.11E+00	
TL-208	6.00E-02	4.51E-01(4/ 4)	TRM 479.0	5.87E-01(2/ 2)	5.23E-01(2/ 2)	
		2.34E-01- 6.42E-01	TRM 479.0	5.31E-01- 6.42E-01	4.30E-01- 6.16E-01	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

TENNESSEE VALLEY AUTHORITY
ENVIRONMENTAL RADIOLOGICAL MONITORING AND INSTRUMENTATION
WESTERN AREA RADIOLOGICAL LABORATORY

RADIOACTIVITY IN CLAM FLESH
PCI/GM - 0.037 BQ/G (DRY WEIGHT)

NAME OF FACILITY: SEQUOYAH NUCLEAR PLANT
LOCATION OF FACILITY: HAMILTON TENNESSEE

DOCKET NO.: 50-327,328
REPORTING PERIOD: 2000

TYPE AND TOTAL NUMBER OF ANALYSIS PERFORMED	LOWER LIMIT OF DETECTION (LLD) SEE NOTE 1	ALL INDICATOR LOCATIONS MEAN (F) RANGE SEE NOTE 2	LOCATION WITH HIGHEST NAME DISTANCE AND DIRECTION	ANNUAL MEAN MEAN (F) RANGE SEE NOTE 2	CONTROL LOCATIONS MEAN (F) RANGE SEE NOTE 2	NUMBER OF NONROUTINE REPORTED MEASUREMENTS
GAMMA SCAN (GELI) 4						
BI-214	5.00E-01	2.15E+00(2/ 2) 8.40E-01- 3.45E+00	SQN Downstream Stati n	2.15E+00(2/ 2) 8.40E-01- 3.45E+00	1.34E+00(2/ 2) 6.86E-01- 2.00E+00	
PB-214	1.00E-01	2.35E+00(2/ 2) 9.56E-01- 3.75E+00	SQN Downstream Stati n	2.35E+00(2/ 2) 9.56E-01- 3.75E+00	1.32E+00(2/ 2) 7.28E-01- 1.90E+00	

NOTE: 1. NOMINAL LOWER LIMIT OF DETECTION (LLD) AS DESCRIBED IN TABLE E-1 .
NOTE: 2. MEAN AND RANGE BASED UPON DETECTABLE MEASUREMENTS ONLY. FRACTION OF DETECTABLE MEASUREMENTS AT SPECIFIED LOCATIONS IS INDICATED IN PARENTHESES (F).

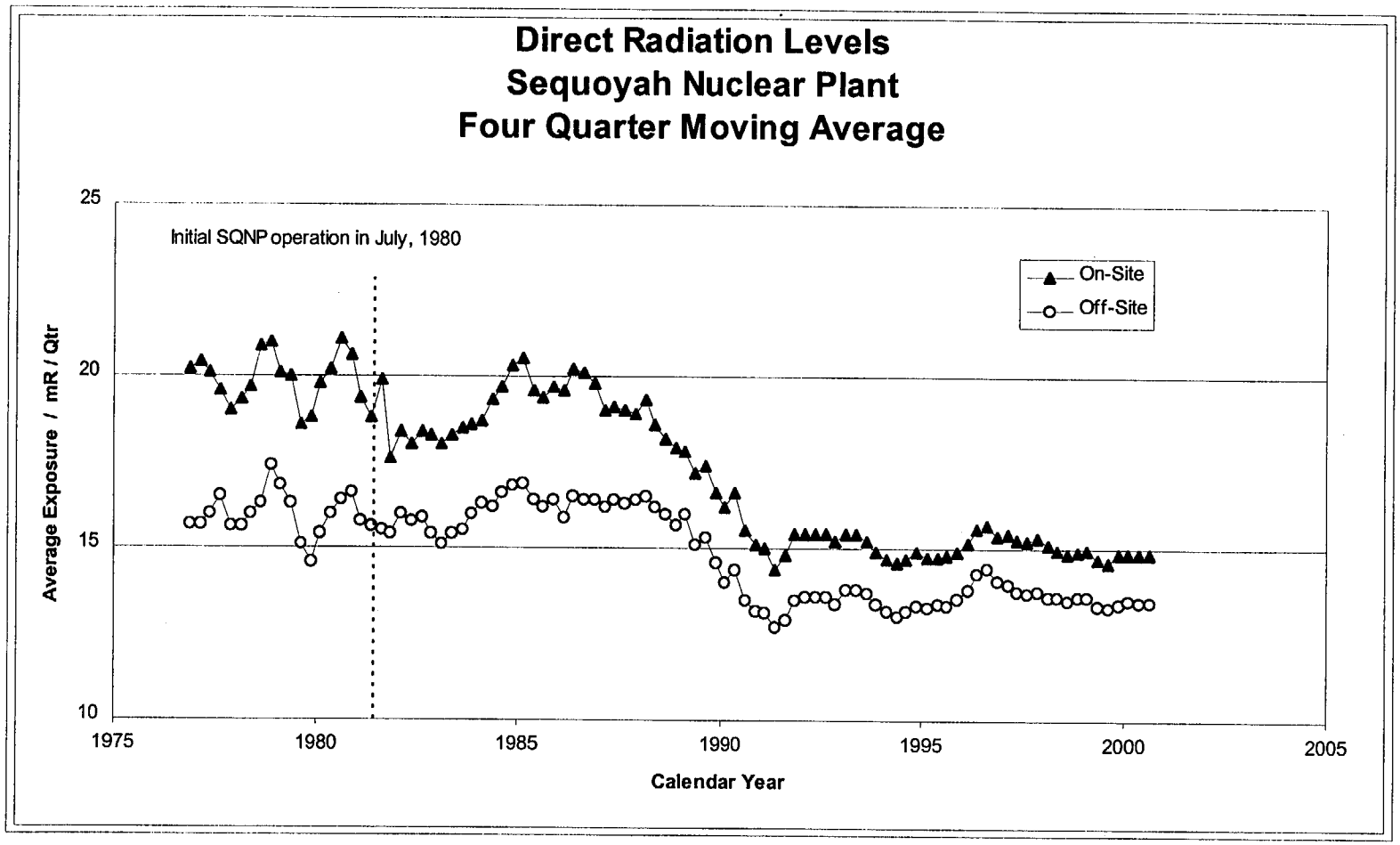

Direct Radiation Levels
Sequoyah Nuclear Plant
Four Quarter Moving Average
Initial SQNP operation in July, 1980
On-Site
Off-Site
Average Exposure / mR / Qtr
25
20
15
10
1975
1980
1985
1990
1995
2000
2005
Calendar Year

Figure H-1

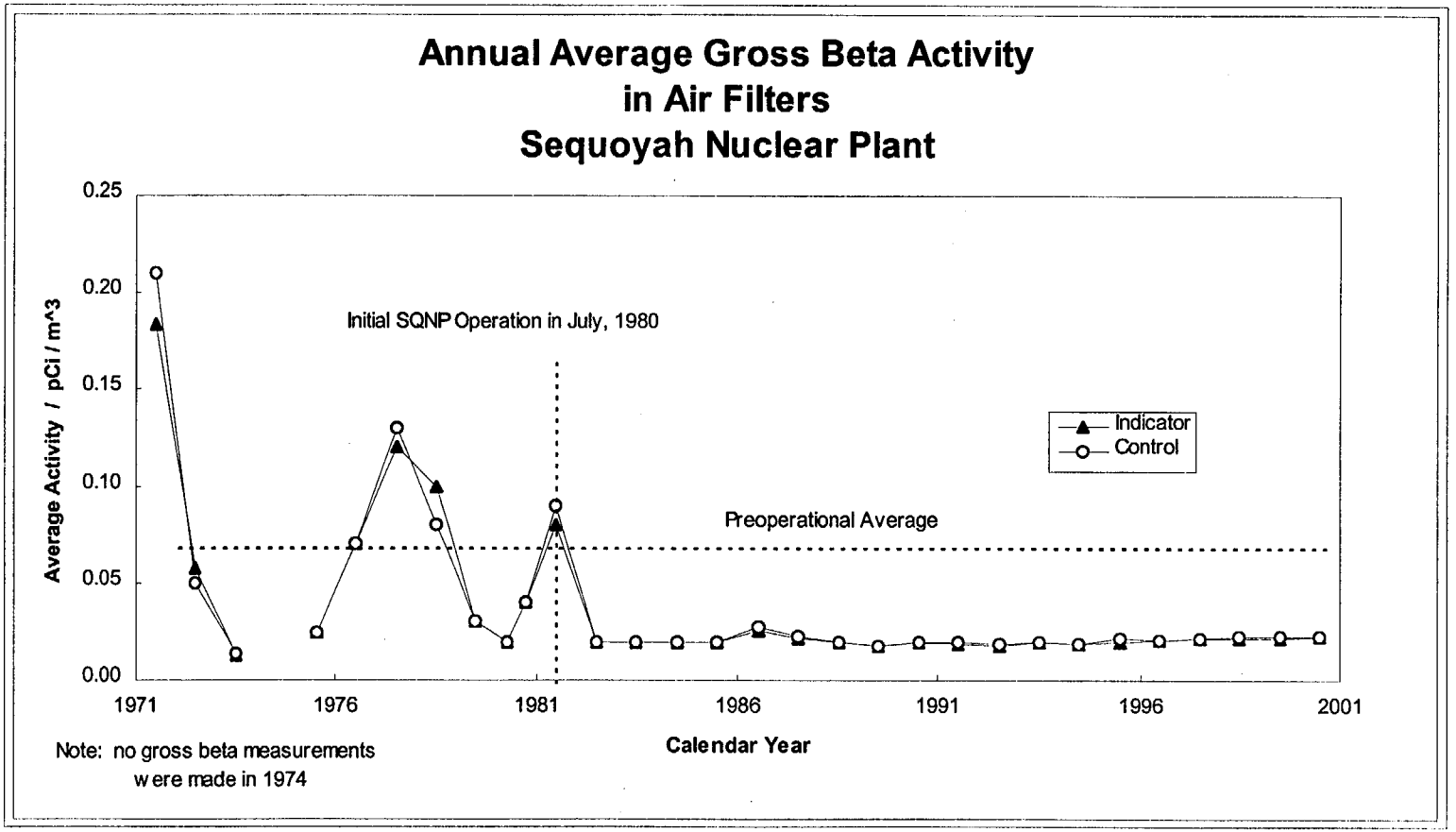

Annual Average Gross Beta Activity
in Air Filters
Sequoyah Nuclear Plant
Average Activity / pCi / m^3
0.25
0.20
0.15
0.10
0.05
0.00
1971
1976
1981
1986
1991
1996
2001
Calendar Year
Initial SQNP Operation in July, 1980
Preoperational Average
Indicator
Control
Note: no gross beta measurements
were made in 1974

Figure H-2

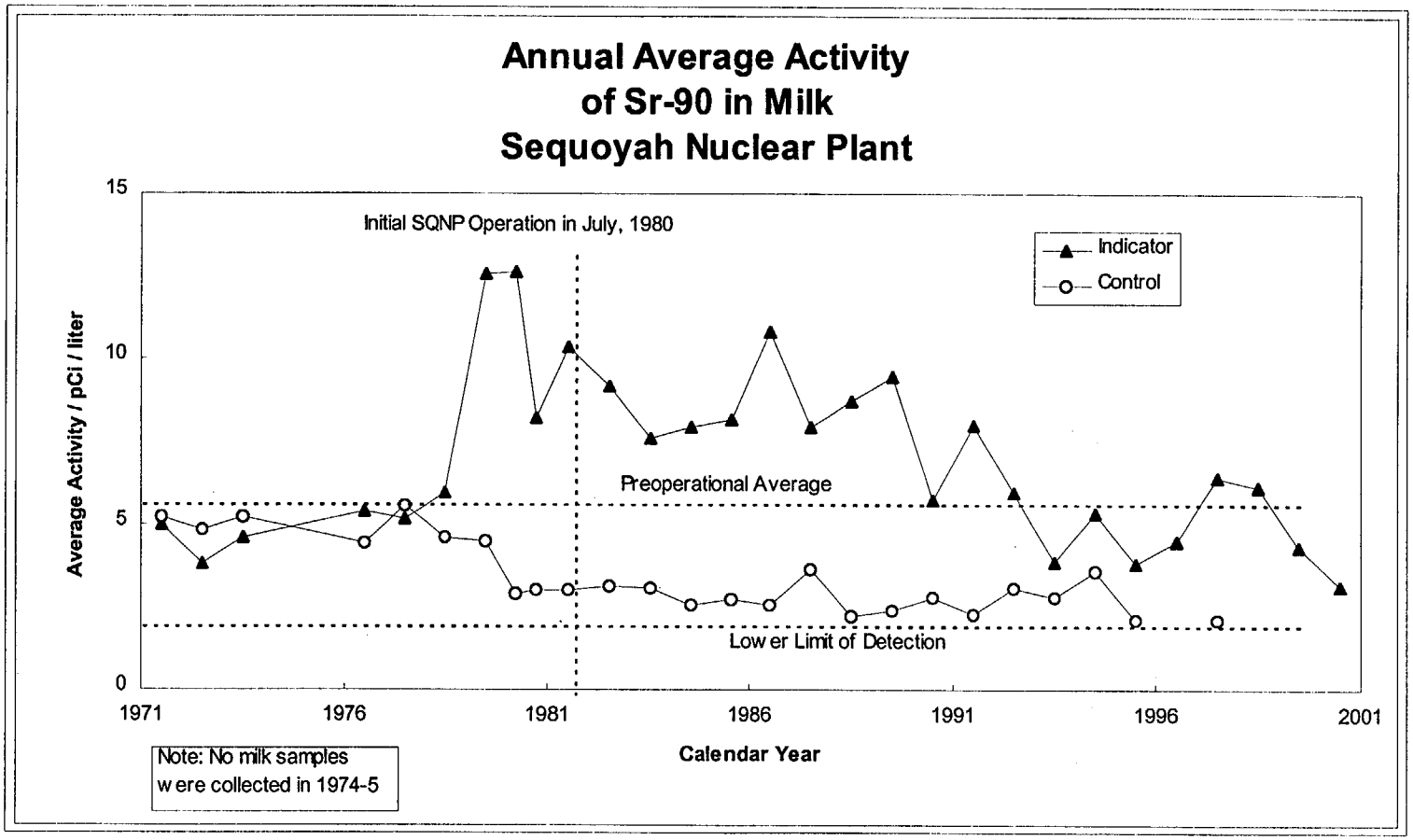

Annual Average Activity
of Sr-90 in Milk
Sequoyah Nuclear Plant
Initial SQNP Operation in July, 1980
Indicator
Control
Preoperational Average
Lower Limit of Detection
Average Activity / pCi / liter
15
10
5
0
1971
1976
1981
1986
1991
1996
2001
Calendar Year
Note: No milk samples
were collected in 1974-5

Figure H-3

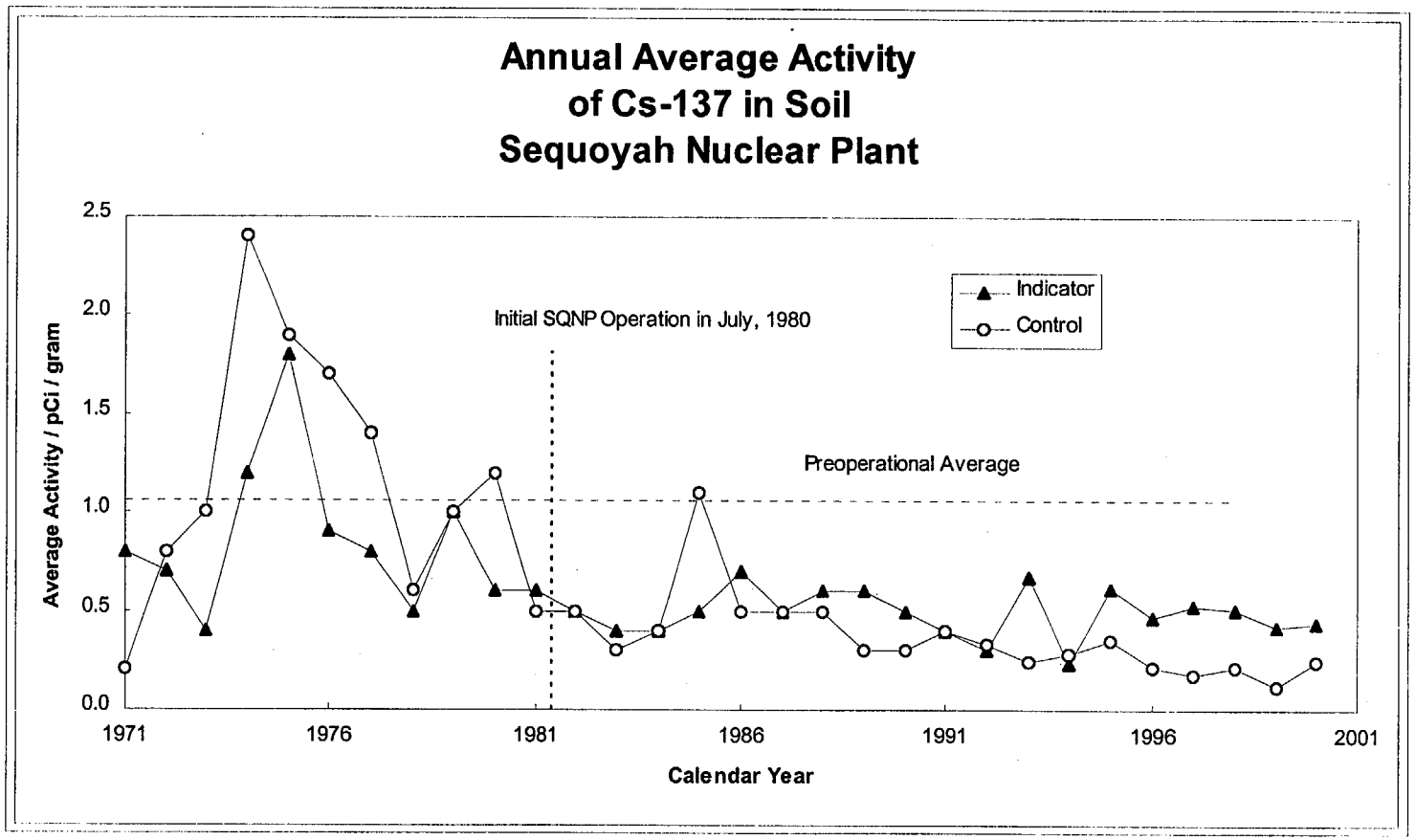

Annual Average Activity
of Cs-137 in Soil
Sequoyah Nuclear Plant
Average Activity / pCi / gram
2.5
2.0
1.5
1.0
0.5
0.0
1971
1976
1981
1986
1991
1996
2001
Calendar Year
Initial SQNP Operation in July, 1980
Preoperational Average
Indicator
Control

Figure H-4

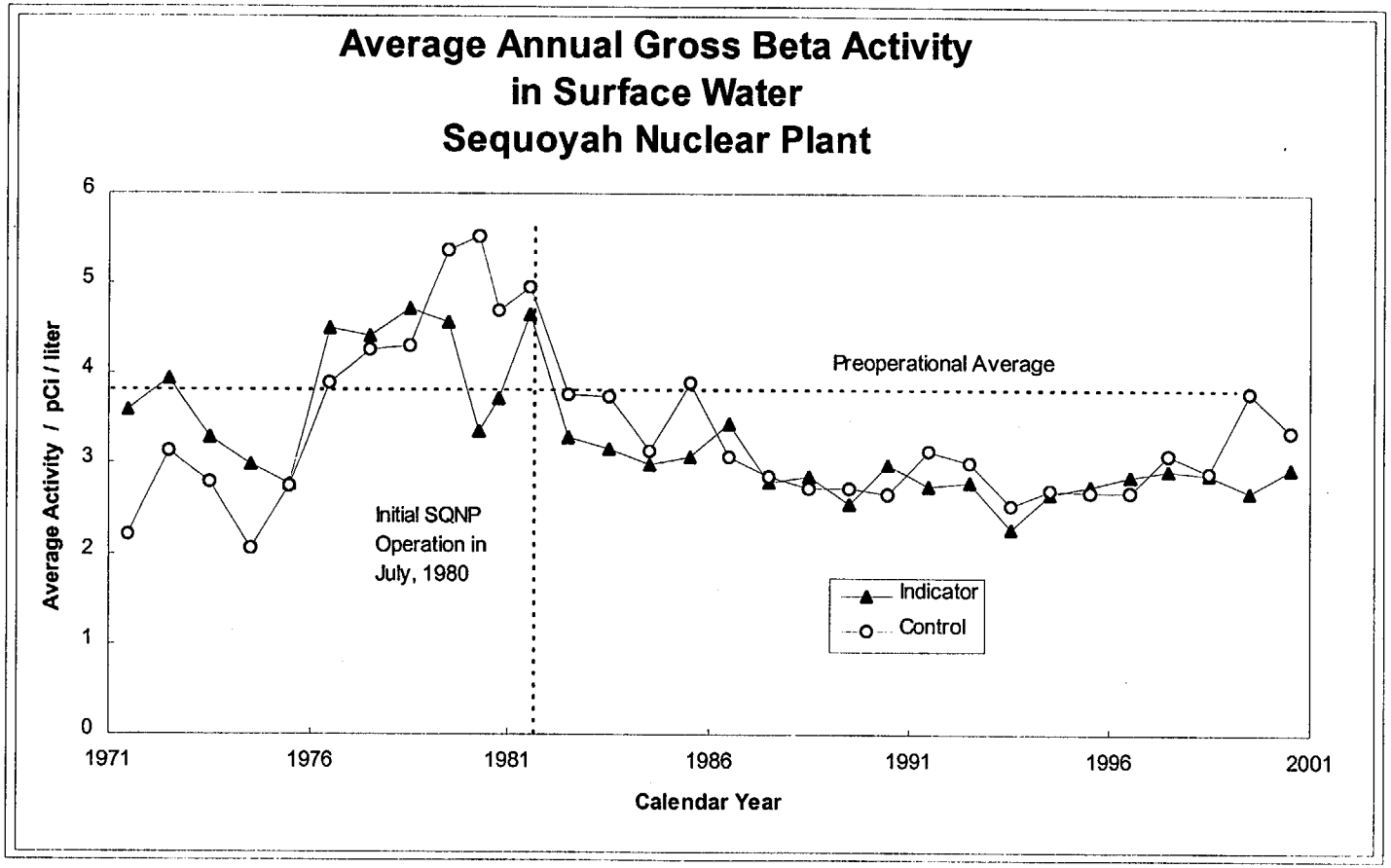

Average Annual Gross Beta Activity
in Surface Water
Sequoyah Nuclear Plant
Average Activity / pCi / liter
0
1
2
3
4
5
6
1971
1976
1981
1986
1991
1996
2001
Calendar Year
Preoperational Average
Initial SQNP
Operation in
July, 1980
Indicator
Control

Figure H-5

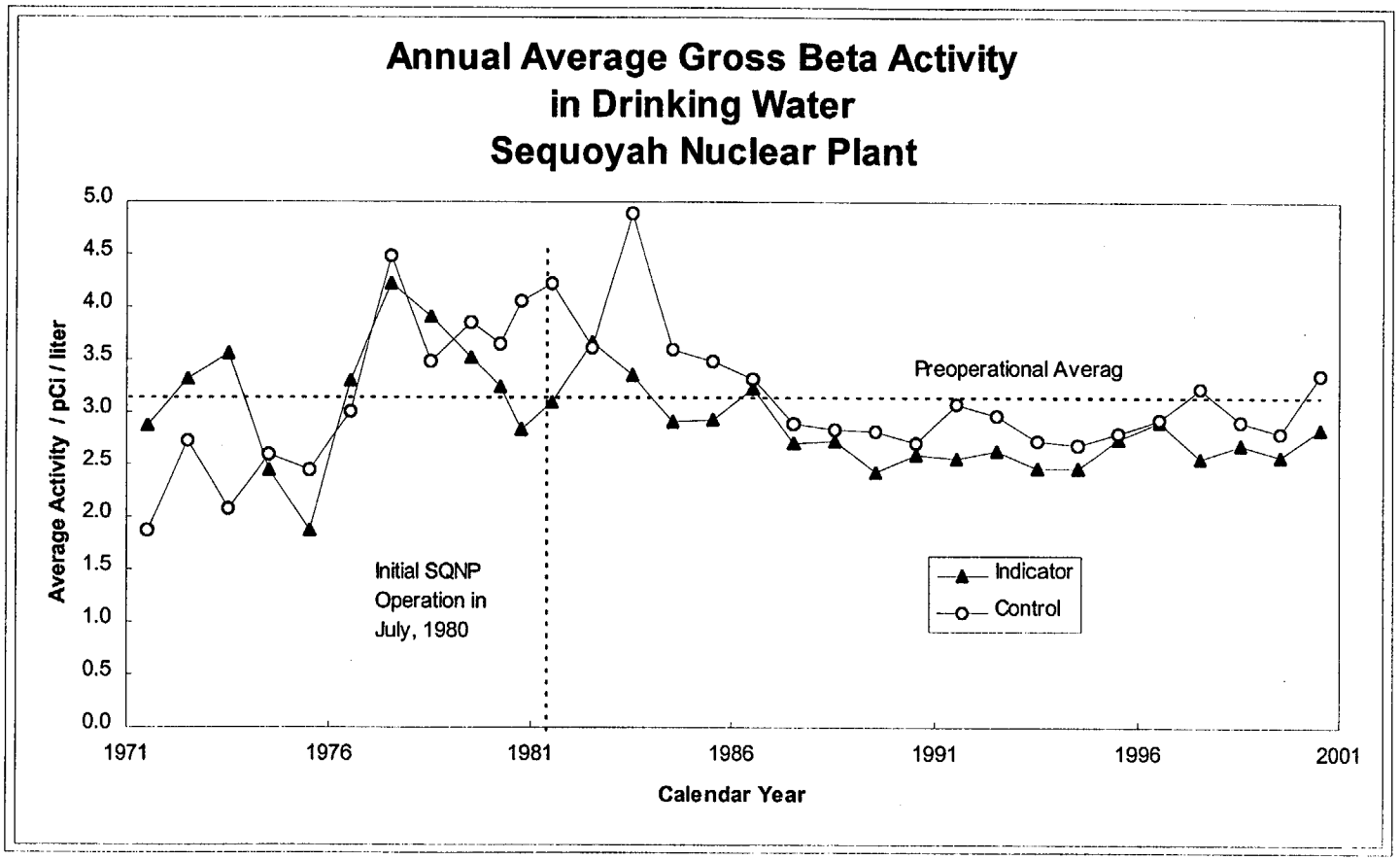

Annual Average Gross Beta Activity
in Drinking Water
Sequoyah Nuclear Plant
Average Activity / pCi / liter
5.0
4.5
4.0
3.5
3.0
2.5
2.0
1.5
1.0
0.5
0.0
1971
1976
1981
1986
1991
1996
2001
Calendar Year
Preoperational Averag
Initial SQNP
Operation in
July, 1980
Indicator
Control

Figure H-6

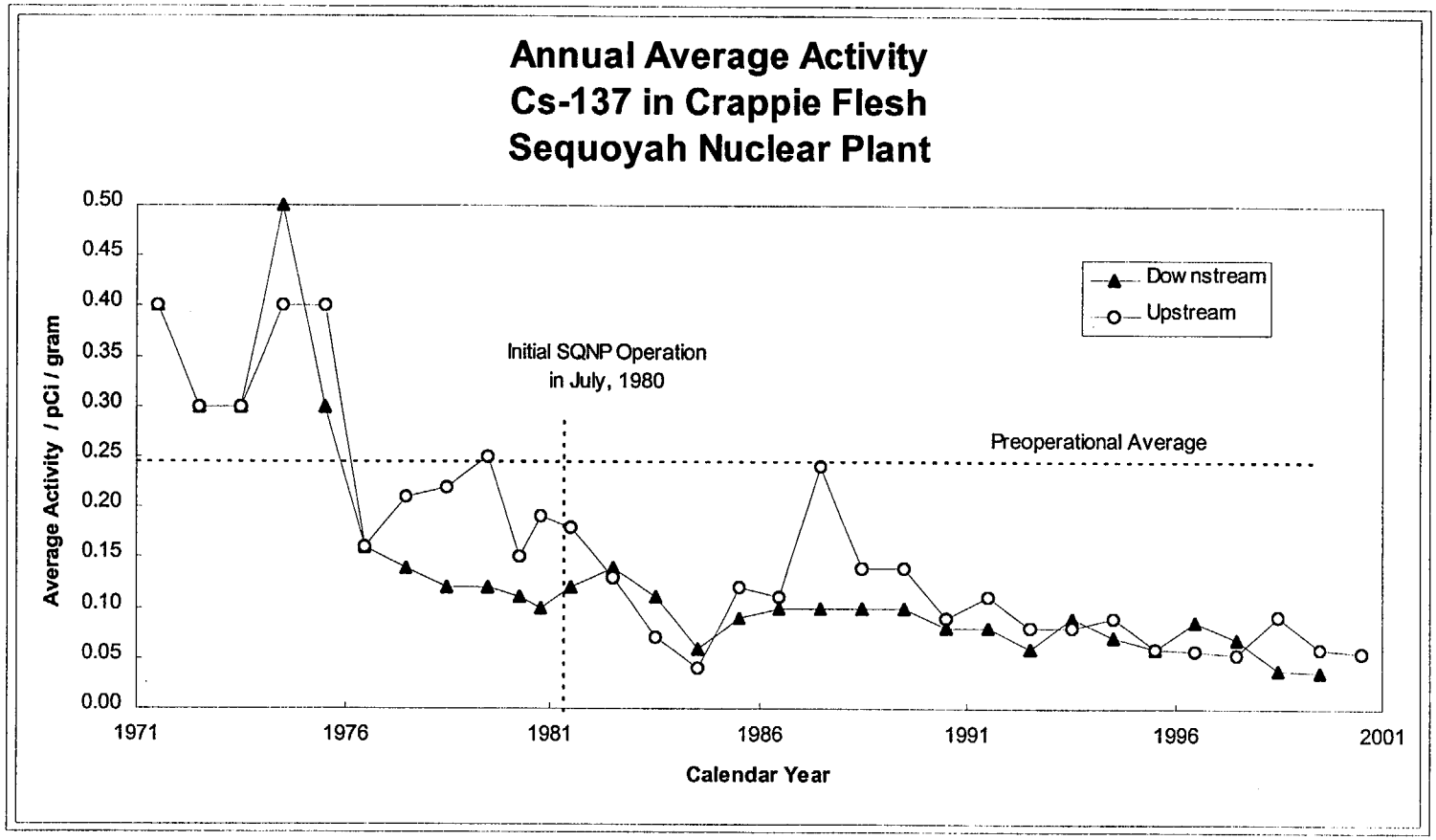

Annual Average Activity
Cs-137 in Crappie Flesh
Sequoyah Nuclear Plant
Average Activity / pCi / gram
0.50
0.45
0.40
0.35
0.30
0.25
0.20
0.15
0.10
0.05
0.00
1971
1976
1981
1986
1991
1996
2001
Calendar Year
Downstream
Upstream
Initial SQNP Operation
in July, 1980
Preoperational Average

Figure H-7

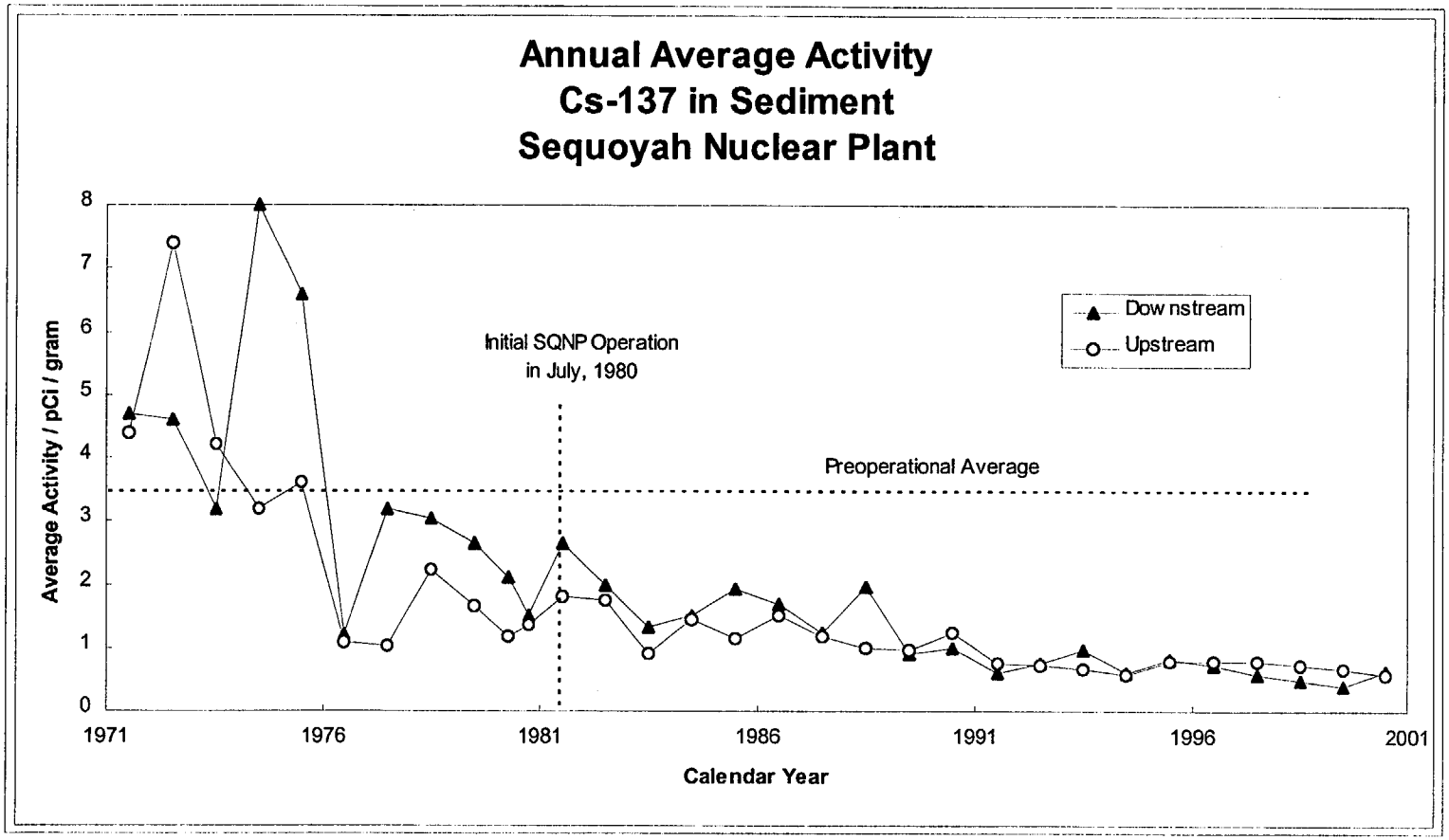
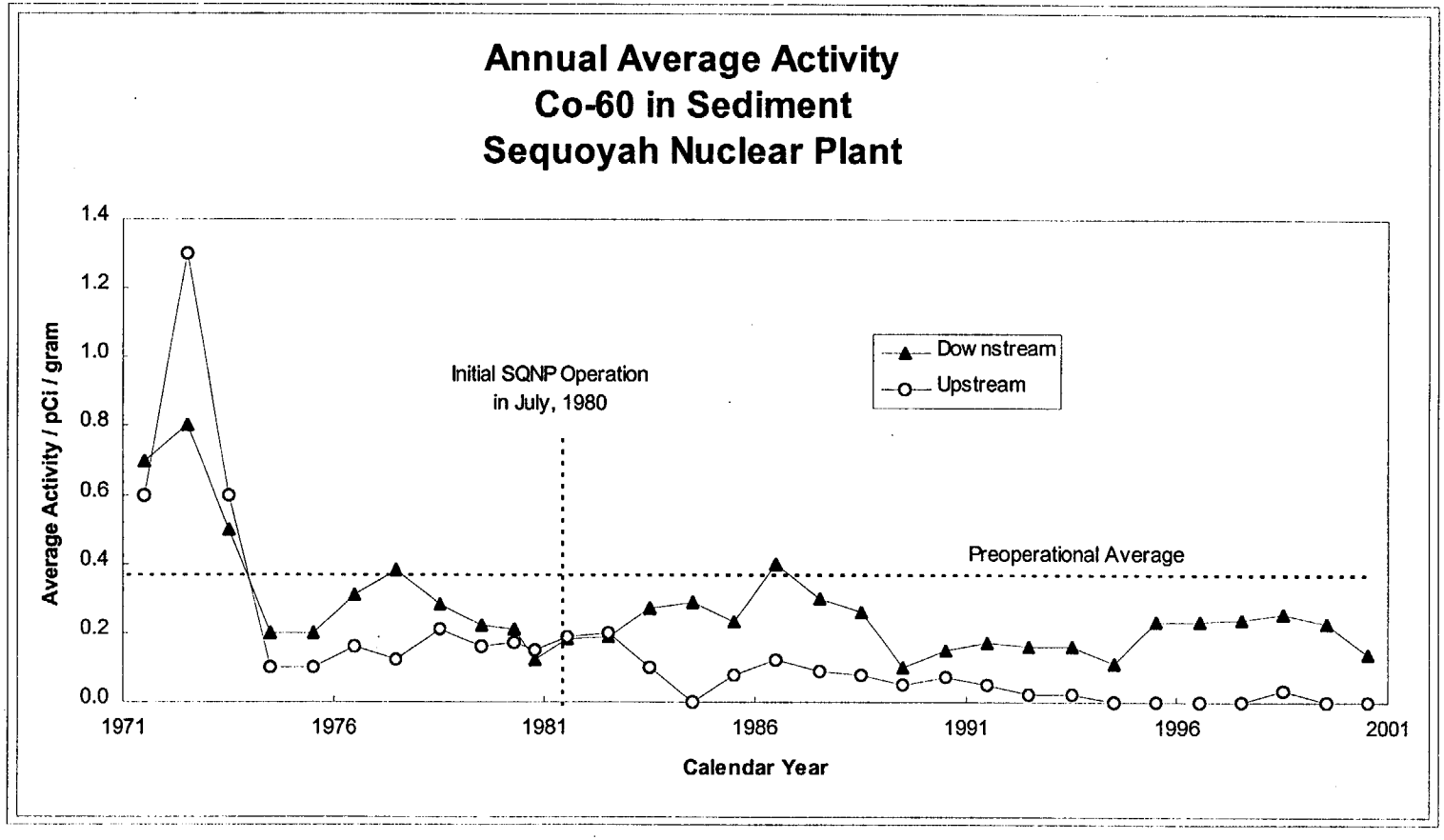

Annual Average Activity
Cs-137 in Sediment
Sequoyah Nuclear Plant
Average Activity / pCi / gram
0
1
2
3
4
5
6
7
8
1971
1976
1981
1986
1991
1996
2001
Calendar Year
Initial SQNP Operation
in July, 1980
Preoperational Average
Downstream
Upstream

Figure H-8


Annual Average Activity
Co-60 in Sediment
Sequoyah Nuclear Plant
Average Activity / pCi / gram
1.4
1.2
1.0
0.8
0.6
0.4
0.2
0.0
1971
1976
1981
1986
1991
1996
2001
Calendar Year
Initial SQNP Operation
in July, 1980
Preoperational Average
Downstream
Upstream

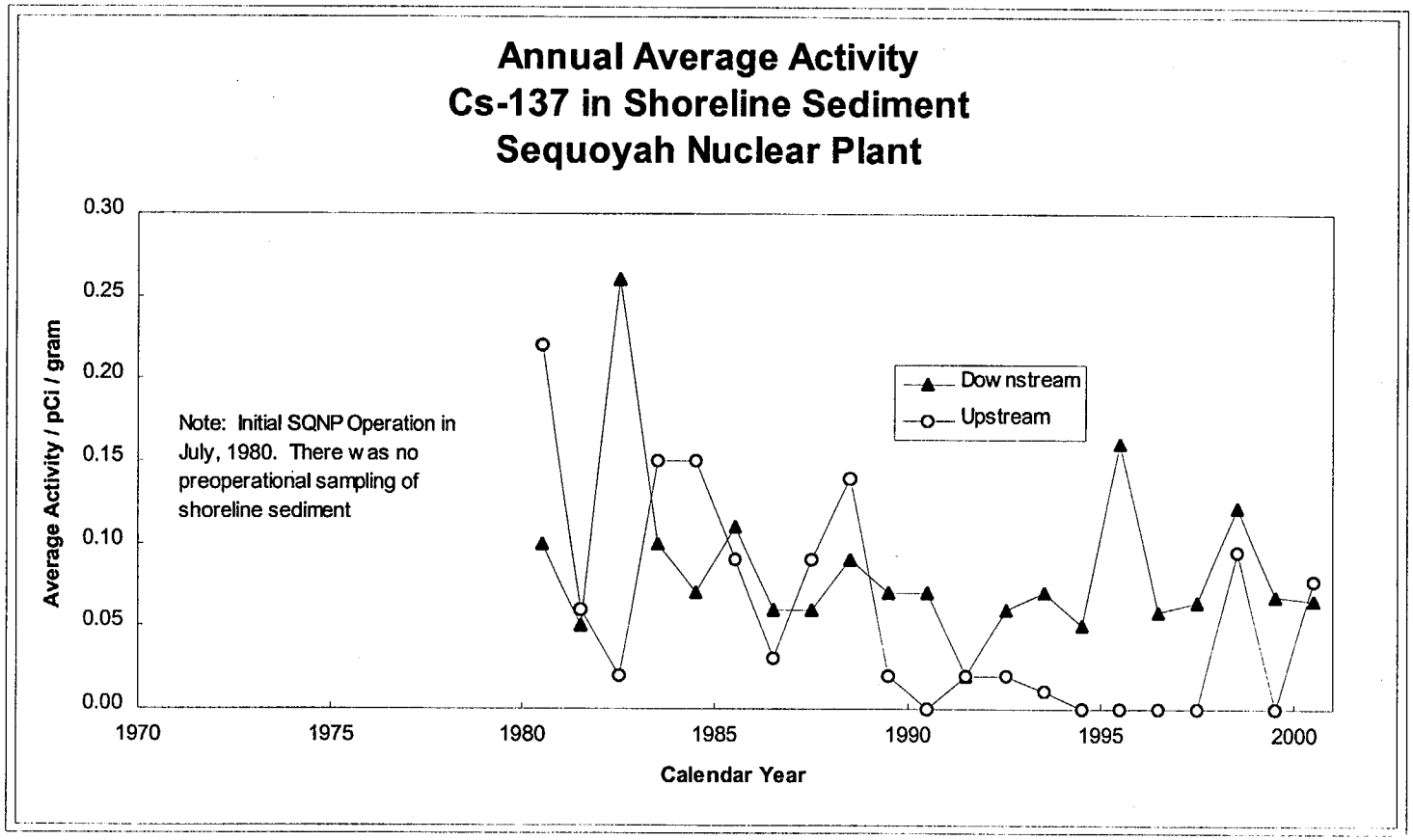

Annual Average Activity
Cs-137 in Shoreline Sediment
Sequoyah Nuclear Plant
Average Activity / pCi / gram
0.30
0.25
0.20
0.15
0.10
0.05
0.00
1970
1975
1980
1985
1990
1995
2000
Calendar Year
Note: Initial SQNP Operation in July, 1980. There was no preoperational sampling of shoreline sediment
Downstream
Upstream

Figure H-10