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Michael R. Kansler Senior Vice President & Chief Operating Officer

April 26, 2001 IPN-01-037

U.S. Nuclear Regulatory Commission Mail Stop O-P1-17 Washington, DC 20555-0001 ATTN: Document Control Desk

SUBJECT:

Indian Point 3 Nuclear Power Plant

Docket No. 50-286

Annual 10 CFR 50.46 Report for Year 2000

Emergency Core Cooling System Evaluation Changes

REFERENCES: (1) Westinghouse letter (INT-01-003), S. Ira to R. Barrett, "10 CFR 50.46 Annual Notification and Reporting for 2000," dated March 6, 2001.

> (2) Entergy letter (IPN-01-20) M. R. Kansler to NRC "Annual 10 CFR 50.46 Report for Year 2000 - Emergency Core Cooling System Evaluation Model Changes" dated March 8, 2001.

Dear Sir:

This letter reports changes to the emergency core cooling system peak cladding temperature (PCT) for a small break loss of coolant accident (LOCA). This letter satisfies the reporting requirements of 10 CFR 50.46(a)(3)(ii).

Peak cladding temperature increased by approximately 13 °F as the result of mixture level tracking/region depletion errors identified in NOTRUMP.

This report is based on the small break loss-of-coolant accident (LOCA) peak cladding temperature changes described in the referenced letter. The large break LOCA peak cladding temperature changes were provided in Reference 2. As defined by 10 CFR 50.46(a)(3)(i), this change is not significant. Indian Point 3 continues to comply with 10 CFR 50.46(a)(3)(ii) because the PCT remains below the maximum permissible fuel cladding temperature of 2200°F.

There are no new commitments made by this letter. If you have any questions, please contact Ms. Charlene Faison at 914-272-3378.

Very truly yours,

Michael R. Kansler Senior Vice President and Chief Operating Officer

Cc: Regional Administrator
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