

**Virginia Electric and Power Company
North Anna Power Station
P. O. Box 402
Mineral, Virginia 23117**

April 24, 2001

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D. C. 20555

Serial No.: 01-241
NAPS: JHL
Docket No.: 50-339
License No.: NPF-7

Dear Sirs:

Pursuant to 10CFR50.73, Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to North Anna Unit 2.

Report No. 50-339/2001-002-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to the Management Safety Review Committee for its review.

Very truly yours,



D. A. Heacock
Site Vice President

Commitments contained in this letter: None

Enclosure

cc: U. S. Nuclear Regulatory Commission
Region II
Sam Nunn Atlanta Federal Center
61 Forsyth Street, SW, Suite 23T85
Atlanta, Georgia 30303-8931

Mr. M. J. Morgan
NRC Senior Resident Inspector
North Anna Power Station

IE22

LICENSEE EVENT REPORT (LER)

(See reverse for required number of
digits/characters for each block)

Estimated burden per response to comply with this mandatory information collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records Management Branch (T-6 E6), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to bj1@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202 (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

FACILITY NAME (1) NORTH ANNA POWER STATION UNIT 2	DOCKET NUMBER (2) 05000 339	PAGE (3) 1 OF 3
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TITLE (4)

MANUAL REACTOR TRIP DUE TO CONTROL BANK "B" GROUP 2 STEP COUNTER INOPERABLE

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MO	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO	MO	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
03	11	2001	2001	02	00	04	24	2001	FACILITY NAME	DOCKET NUMBER
										05000
										05000

OPERATING MODE (9)	3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply) (11)					
POWER LEVEL (10) 0	0	20.2201(b)		20.2203(a)(3)(ii)		50.73(a)(2)(ii)(B)	50.73(a)(2)(ix)(A)
		20.2201(d)		20.2203(a)(4)		50.73(a)(2)(iii)	50.73(a)(2)(x)
		20.2203(a)(1)		50.36(c)(1)(i)(A)	x	50.73(a)(2)(iv)(A)	73.71(a)(4)
		20.2203(a)(2)(i)		50.36(c)(1)(ii)(A)		50.73(a)(2)(v)(A)	73.71(a)(5)
		20.2203(a)(2)(ii)		50.36(c)(2)		50.73(a)(2)(v)(B)	OTHER
		20.2203(a)(2)(iii)		50.46(a)(3)(ii)		50.73(a)(2)(v)(C)	Specify in Abstract below or in
		20.2203(a)(2)(iv)		50.73(a)(2)(i)(A)		50.73(a)(2)(v)(D)	NRC Form 366A
		20.2203(a)(2)(v)		50.73(a)(2)(i)(B)		50.73(a)(2)(vii)	
		20.2203(a)(2)(vi)		50.73(a)(2)(i)(C)		50.73(a)(2)(viii)(A)	
		20.2203(a)(3)(i)		50.73(a)(2)(ii)(A)		50.73(a)(2)(viii)(B)	

LICENSEE CONTACT FOR THIS LER (12)

NAME David A. Heacock	TELEPHONE NUMBER (Include Area Code) (540) 894-2101
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX
X	AA	CTR	W120	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE).	X	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
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ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On March 11, 2001, at 0109 hours, with Unit 2 at 0 percent power (Mode 3) and shutting down for refueling, the control bank "B" group 2 step counter demand position indicator stopped moving at 53 steps. The Limiting Condition for Operation (LCO) for Technical Specification 3.1.3.3 requires the rod group step counter demand position indicator to be within plus or minus 2 steps of the demand position from the logic cabinet for each shutdown or control rod group not fully inserted. The control bank "B" group 2 step counter demand position indicator was greater than 2 steps from the demand position from the logic cabinet. The action requirements of TS 3.1.3.3 require the reactor trip breakers to be opened within 15 minutes if the LCO requirements cannot be met. At 0119 hours, a manual reactor trip was initiated by opening the reactor trip breakers. At 0428 hours, an 8 hour report was made to the NRC Operations Center in accordance with 10 CFR 50.72(b)(3)(iv)(A). The cause for the control bank "B" group 2 step counter demand position indicator not moving was determined to be a defect in the down counter gear. This event is reportable per 10 CFR 50.73 (a)(2)(iv)(A) for any event or condition that resulted in the manual actuation of the reactor protection system. No significant safety consequences resulted from this event since the reactor was already subcritical, with the control banks fully inserted, when the reactor trip breakers were opened and the shutdown banks inserted into the core. Therefore, the health and safety of the public were not affected at any time during this event.

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NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

1. DESCRIPTION OF EVENT

On March 11, 2001, at approximately 0109 hours, with Unit 2 at 0 percent power (Mode 3) and shutting down for refueling, the control bank "B" group 2 step counter demand position indicator (EIS System AA, Component CTR) stopped moving at 53 steps while rods were being driven into the core. All other indications of rod motion were normal. The Limiting Condition for Operation (LCO) for Technical Specification 3.1.3.3 requires the rod group step counter demand position indicator to be within plus or minus 2 steps of the demand position from the logic cabinet for each shutdown or control rod group not fully inserted. The control bank "B" group 2 step counter demand position indicator was greater than 2 steps from the demand position from the logic cabinet. The action requirements of TS 3.1.3.3 require the reactor trip breakers (EIS - BKR) to be opened within 15 minutes if the LCO requirements cannot be met. Operations personnel continued to insert the control banks into the core. At 0119 hours, a manual reactor trip was initiated by opening the reactor trip breakers, inserting the shutdown banks into the core. At 0428 hours, an 8 hour report was made to the NRC Operations Center in accordance with 10 CFR 50.72(b)(3)(iv)(A). This event is reportable per 10 CFR 50.73 (a)(2)(iv)(A) for any event or condition that resulted in the manual actuation of the reactor protection system.

2. SIGNIFICANT SAFETY CONSEQUENCES AND IMPLICATIONS

No significant safety consequences resulted from this event since the reactor was already subcritical, with all control banks fully inserted, when the reactor trip breakers were opened and the shutdown banks inserted into the core. Therefore, the health and safety of the public were not affected at any time during this event.

3. CAUSE

The cause of the control bank "B" group 2 step counter demand position indicator not moving was determined to be a defect in the down counter gear such that when the ones column wheel reaches the count of three, the down count coil hammer just misses the gear and is unable to advance the ones column wheel any further.

4. IMMEDIATE CORRECTIVE ACTIONS

Upon discovery of the condition, the "B" control bank group step counter for Group 2 was declared inoperable and the action requirements of TS 3.1.3.3 were entered to open the reactor trip breakers within 15 minutes.

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NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

5. ADDITIONAL CORRECTIVE ACTIONS

A manual reactor trip was initiated by opening the reactor trip breakers, inserting the shutdown banks into the core.

6. ACTIONS TO PREVENT RECURRENCE

The control bank "B" rod group 2 step counter and relay driver were replaced during the refueling outage.

7. SIMILAR EVENTS

Unit 2 LER 1998-001-00, dated April 30, 1998, documents that with Unit 2 at 0 percent power (Mode 3) while inserting control rods in preparation for a refueling outage, the "B" control bank group step counter for Group 2 deviated from the actual rod position by more than the allowable Technical Specification limit of two steps. The cause of this event was attributed to a malfunction of the mechanical drive for the step counter.

8. ADDITIONAL INFORMATION

Unit 1 was operating in Mode 1, at 100 percent power, and was not affected by this event.