February 18, 1993

Docket No. 50-219

Mr. John J. Barton
Vice President and Director
GPU Nuclear Corporation
Oyster Creek Nuclear Generating Station
Post Office Box 388
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Dear Mr. Barton:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M83226)

The Commission has issued the enclosed Amendment No. 161 to Facility Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station, in response to your application dated April 20, 1992.

The amendment removes certain fire protection related items from Oyster Creek Nuclear Generating Station Technical Specifications and relocates them in the Fire Protection Program to the Updated Final Safety Analysis Report. This amendment was requested in accordance with the guidance provided in Generic Letters 86-10 and 88-12.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Alexander W. Dromerick, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 161 to DPR-16
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. John J. Barton
GPU Nuclear Corporation

Oyster Creek Nuclear
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cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 161
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by GPU Nuclear Corporation, et al. (the licensee), dated April 20, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-16 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 161 , are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz

John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 18, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 161

FACILITY OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

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3.12-4	----
3.12-5	----
3.12-6	----
3.12-7	----
3.12-8	----
3.12-9	----
3.12-10	----
3.12-11	----
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B. The testing of one system, subsystem, train or other designated component at the beginning of each subinterval.

1.24 SURVEILLANCE REQUIREMENTS

Surveillance requirements are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within the safety limits, and that the limiting conditions of operation will be met. Each surveillance requirement shall be performed within the specified time interval with a maximum allowable extension not to exceed 25% of the surveillance interval.*

Surveillance requirements for systems and components are applicable only during the modes of operation for which the system or components are required to be operable, unless otherwise stated in the specification.

This definition establishes the limit for which the specified time interval for Surveillance Requirements may be extended. It permits an allowable extension of the normal surveillance interval to facilitate surveillance scheduling and consideration of plant operating conditions that may not be suitable for conducting the surveillance, e.g., transient conditions or other ongoing surveillance or maintenance activities. It also provides flexibility to accommodate the length of a fuel cycle for surveillances that are performed at each refueling outage and are specified with a fuel cycle length surveillance interval. It is not intended that this provision be used repeatedly as a convenience to extend surveillance intervals beyond that specified for the surveillance that are not performed during refueling outages. The limitation of this definition is based on engineering judgement and the recognition that the most probable result of any particular surveillance being performed is the verification of conformance with the Surveillance Requirements. This provision is sufficient to ensure that the reliability ensured through surveillance activities is not significantly degraded beyond that obtained from the specified surveillance interval.

1.25 (DELETED)

1.26 FRACTION OF LIMITING POWER DENSITY (FLPD)

The fraction of limiting power density is the ratio of the linear heat generation rate (LHGR) existing at a given location to the design LHGR for that bundle type.

1.27 MAXIMUM FRACTION OF LIMITING POWER DENSITY (MFLPD)

The maximum fraction of limiting power density is the highest value existing in the core of the fraction of limiting power density (FLPD).

* Not applicable to containment leak rate test.

3.12 Alternate Shutdown Monitoring Instrumentation

<u>Applicability:</u> Applies to the operating status of alternate shutdown monitoring instrumentation.

<u>Objective:</u> To assure the operability of the alternate shutdown monitoring instrumentation.

<u>Specification:</u>

A. The alternate shutdown monitoring instruments listed in Table 3.12-1 shall be operable during reactor power operations and when reactor coolant temperature exceeds 212 F.

B. With less than the minimum number of operable channels specified in Table 3.12-1, either restore the inoperable channel to operable status within 30 days, or be in at least hot shutdown within the next 12 hours and in cold shutdown within the following 24 hours.

<u>Basis:</u>

The operability of the alternate shutdown monitoring instrumentation ensures that sufficient capability is available to permit shutdown and maintenance of hot shutdown of the plant from locations outside of the control room. This capability is required in the event control room habitability is lost and is consistent with Appendix R and General Design Criteria 19 of 10 CFR 50.

TABLE 3.12-1 ALTERNATE SHUTDOWN

MONITORING INSTRUMENTATION

Functional Unit	Readout Location	Min. Channels Operable
Reactor Pressure	RSP	1
Reactor Water Level (fuel zone)	RSP	1
Condensate Storage Tank Level	Local	1
Service Water Pump Discharge Pressure	Local	1
Control Rod Drive System Flowmeter	Rx 23' near V-15-30	1
Shutdown Cooling System Flowmeter	Local	1
Isolation Condenser "B" Shell Water Level	RSP	1
Reactor Building Closed Cooling Water Pump Discharge Pressure	Local	1

RSP - Remote Shutdown Panel

4.12 Alternate Shutdown Monitoring Instrumentation

<u>Applicability:</u> Applies to the surveillance requirements of the alternate shutdown monitoring instrumentation.

<u>Objective:</u> To specify the minimum frequency and type of surveillance to be applied to the alternate shutdown monitoring instrumentation.

<u>Specification:</u>

Each of the alternate shutdown monitoring channels shall be demonstrated operable by performance of the channel check and channel calibration operations at the frequencies shown in Table 4.12-1.

<u>Basis:</u>

The operability of the alternate shutdown monitoring instrumentation ensures that sufficient capability is available to permit shutdown and maintenance of hot shutdown of the plant from locations outside of the control room. The type and frequency of surveillances required in Table 4.12-1 are consistent with or more conservative than the BWR Standard Technical Specifications.

TABLE 4.12-1 ALTERNATE SHUTDOWN

MONITORING INSTRUMENTATION

Functional Unit	Channel Check	Channel Calibration
Reactor Pressure	M	Q
Reactor Water Level (fuel zone)	n/a	Q
Condensate Storage Tank Level	M	R
Service Water Pump Discharge Pressure	M	R
Control Rod Drive System Flowmeter	M	R
Shutdown Cooling System Flowmeter	n/a	R
Isolation Condenser "B" Shell Water Level	M	R
Reactor Building Closed Cooling Water Pump Discharge Pressure	M	R

M - Monthly

Q - Quarterly

R - Refueling Outage

d. At least two licensed reactor operators shall be in the control room during all reactor startups, shutdowns, and other periods involving planned control rod manipulations.

e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

f. An individual qualified in radiation protection measures shall be on site when fuel is in the reactor.

g. (deleted)

h. Each on duty shift shall include a Shift Technical Advisor except that the Shift Technical Advisors position need not be filled if the reactor is in the refuel or shutdown mode and the reactor is less than 212 F.

i. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety related functions.

In the event that unforeseen problems require substantial amounts of overtime to be used or during extended periods of shutdown for refueling, major maintenance or major plant modifications, on a temporary basis, the following guidelines shall be followed:

a. An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.

b. An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any seven-day period, all excluding shift turnover time.

c. A break of at least eight hours should be allowed between work period, including shift turnover time.

d. In a, b, and c above, the time required to complete shift turnover is to be counted as break time and is not to be counted as work time.

e. Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Department Managers, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation.

6.4 TRAINING

6.4.1 A retaining program for operators shall be maintained under the direction of the Manager responsible for plant training and shall meet the requirements and recommendation of 10 CFR Part 55. Replacement training programs, the content of which shall meet the requirements of 10 CFR Part 55, shall be conducted under the direction of the Manager responsible for plant training for licensed operators and Senior Reactor Operators.

6.5 REVIEW AND AUDIT

6.5.1 TECHNICAL REVIEW AND CONTROL

The Vice President of each division within GPU Nuclear Corporation shall be responsible for ensuring the preparation, review, and approval of documents required by the activities described in 6.5.1.1 through 6.5.1.5 within his functional area of responsibility as assigned in the GPUN Review and Approval Matrix. Implementing approvals shall be performed at the cognizant manager level or above.

ACTIVITIES

6.5.1.1 Each procedure required by Technical Specification 6.8 and other procedures which affect nuclear safety, and substantive changes thereto, shall be prepared by a designated individual(s)/group knowledgeable in the area affected by the procedure. Each such procedure, and substantive change thereto, shall be reviewed for adequacy by an individual(s)/group other than the preparer, but who may be from the same division as the individual who prepared the procedure or change.

6.5.3 <u>AUDITS</u>

6.5.3.1 Audits of facility activities shall be performed under the cognizance of the Vice President - responsible for Quality Assurance. These audits shall encompass:

a. The conformance of facility operations to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.

b. The performance, training and qualifications of the facility staff at least once per 12 months.

c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.

d. The Facility Emergency Plan and implementing procedures at least once per 12 months.

e. The Facility Security Plan and implementing procedures at least once per 12 months.

f. The Fire Protection Program and implementing procedures at least once per 24 months.

g. The performance of activities required by the Operational Quality Assurance Plan to meet the criteria of Appendix 'B', 10 CFR 50, at least once per 24 months.

h. The radiological environmental monitoring program and the results thereof at least once per 12 months.

i. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months.

j. The PROCESS CONTROL PROGRAM and implementing procedures for radioactive wastes at least once per 24 months.

k. Any other area of facility operation considered appropriate by the IOSRG or the Office of the President-GPUNC.

6.5.3.2 Audits of the following shall be performed under the cognizance of the Vice President - responsible for technical support.

a. An independent fire protection and loss prevention program inspection and audit shall be performed annually utilizing either qualified licensee personnel or an outside fire protection firm.

b. An inspection and audit of the fire protection and loss prevention program, by an outside qualified fire consultant at intervals no greater than 3 years.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented, and maintained that meet or exceed the requirements of the Nuclear Regulatory Commission's Regulatory Guide 1.33 (the applicable revision is identified in the GPU Nuclear Operational Quality Assurance Plan) and as provided in 6.8.2 and 6.8.3 below.

Written procedures shall be adopted and maintained to implement the:

Process Control Plan
Offsite Dose Calculation Manual
Fire Protection Program

6.8.2 Each procedure and administrative policy of 6.8.1 above, and substantive changes thereto, shall be reviewed as described in 6.5.1.1 and approved as described in 6.5.1 prior to implementation and periodically as specified in the Administrative Procedures.

6.8.3 Temporary changes to procedures 6.8.1 above may be made provided:

a. The intent of the original is not altered.

b. The change is approved by two members of GPUNC Management Staff authorized under Section 6.5.1.12 and knowledgeable in the area affected by the procedure. For changes which may affect the operational status of facility systems or equipment, at least one of these individuals shall be a member of facility management or supervision holding a Senior Reactor Operator's License on the facility.

c. The change is documented, subsequently reviewed and approved as described in 6.8.2 within 14 days of implementation.

6.9 REPORTING REQUIREMENTS

In addition to the applicable reporting requirements of 10 CFR, the following identified reports shall be submitted to the Administrator of the NRC Region I office unless otherwise noted.

6.9.1 ROUTINE REPORTS

a. Startup Report. A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specified details required in license conditions based on other commitments shall be included in this report.

6.9.3 UNIQUE REPORTING REQUIREMENTS

Special reports shall be submitted to the appropriate NRC office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification.

a. Materials Radiation Surveillance Specimen Reports (4.3A)

b. Integrated Primary Containment Leakage Tests (4.5)

c. Results of required leak tests performed on sealed sources if the tests reveal the presence of 0.005 microcuries or more of removable contamination.

d. (deleted)

e. Core Spray Sparger Inservice Inspection (Table 4.3.1-9)

   Prior to startup of each cycle, a special report presenting the results of the inservice inspection of the Core Spray Spargers during each refueling outage shall be submitted to the Commission for review.

f. Liquid radwaste batch discharge exceeding Specification 3.6.B.1.

g. Main condenser offgas discharge without treatment per Specification 3.6.D.1.

h. Dose due to radioactive liquid effluent exceeding Specification 3.6.J.1.

i. Air dose due to radioactive noble gas in gaseous effluent exceeding Specification 3.6.L.1.

j. Air dose due to radioiodine and particulates exceeding Specification 3.6.M.1.

k. Annual total dose due to radioactive effluents exceeding Specification 3.6.N.1.

l. Records of results of analyses required by the Radiological Environmental Monitoring Program.

m. Failures and challenges to Relief and Safety Valves

   Failures and challenges to Relief and Safety Valves which do not constitute an LER will be the subject of a special report submitted to the Commission within 60 days of the occurrence. A challenge is defined as any automatic actuation (other than during surveillance or testing) of Safety or Relief Valves.

n. Plans for compliance with standby liquid control Specifications 3.2.C.3(b) and 3.2.C.3(e)(1) or plans to obtain enrichment test results per Specification 4.2.E.5.

o. Inoperable high range radioactive noble gas effluent monitor (3.13.H)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 161

TO FACILITY OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND
JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated April 20, 1992, GPU Nuclear Corporation (the licensee) submitted a request for changes to the Oyster Creek Nuclear Generating Station (OCNGS) Technical Specifications (TS). The requested changes would delete the fire protection TS and their associated bases and definitions from the TS. The deleted requirements have been relocated to the OCNGS Fire Protection Plan. The proposed amendment would augment the Administrative Controls Section of the TS to require: (1) that written procedures be established; implemented and maintained for activities involving implementation of the Fire Protection Program, (2) periodic review of the Fire Protection Program and implementing procedures by a qualified individual/ organization, and (3) submittal of recommended changes to the Fire Protection Program and implementing procedures to the Safety Review and Audit Board. Conforming changes would also be made to the TS. License Condition 2.C(3) has been previously revised to: (1) require the licensee to implement and maintain in effect all provisions of the approved Fire Protection Program as described in the updated Final Safety Analysis Report (UFSAR) as approved in the Fire Protection Safety Evaluation Report dated March 3, 1978, and supplements, and (2) permit the licensee to make changes to the approved Fire Protection Program without prior approval of the NRC, only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire. The proposed changes are in accordance with the guidance provided in NRC Generic Letters 88-12, "Removal of Fire Protection Requirements from Technical Specifications," dated August 2, 1988 and 86-10, "Implementation of Fire Protection Requirements," dated April 24, 1986.

2.0 BACKGROUND

Following the fire at the Browns Ferry Nuclear Power Plant on March 22, 1975, the Commission undertook a number of actions to ensure that improvements were implemented in the Fire Protection Programs for all power reactor facilities. Because of the extensive modification of Fire Protection Programs and the number of open issues resulting from staff evaluations, a number of revisions and alterations occurred in these programs over the years. Consequently, licensees were requested by Generic Letter 86-10 to incorporate the final NRC-approved Fire Protection Program in their Final Safety Analysis Reports (FSARs). In this manner, the Fire Protection Program, including the systems, the administrative and technical controls, the organization, and other plant features associated with fire protection, would have a status consistent with that of other plant features described in the FSAR. In addition, the Commission concluded that a standard license condition, requiring compliance with the provisions of the Fire Protection Program as described in the FSAR, should be used to ensure uniform enforcement of fire protection requirements. Finally, the Commission stated that with the requested actions, licensees may request an amendment to delete the fire protection TS that would now be unnecessary.

The licensees for the Callaway and Wolf Creek plants submitted lead-plant proposals to remove fire protection requirements from their TS. This action was an industry effort to obtain NRC guidance on an acceptable format for license amendment requests to remove fire protection requirements from TS. Additionally, in the licensing review of new plants, the staff had approved requests to remove fire protection requirements from TS issued with the operating license. Thus, on the basis of the lead-plant proposals and the staff's experience with TS for new licenses, Generic Letter 88-12 was issued to provide guidance on removing fire protection requirements from TS.

3.0 DISCUSSION

The licensee has requested an amendment to its operating license which would relocate fire protection Technical Specifications to the Updated Final Safety Analysis Report (UFSAR).

The TS changes proposed by the licensee are as follows:

1. Deletion of TS Section 3.12 for Fire Detection Instrumentation, Fire Suppression Water Systems, Spray and/or Sprinkler Systems, Fire Hose Stations, Halon Systems and Fire Barrier Penetration Fire Seals and associated bases. These TS requirements would be relocated to plant administrative procedures controls and to the Fire Protection Plan.

2. Deletion of the minimum fire brigade staffing requirements TS 6.2.2g.

3. TS 6.5.3.2a currently states, "An independent protection and loss prevention program inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel, or an outside fire protection firm." The licensee requested that the word, "offsite" be deleted from the TS.

4. TS Table 3.12-6 (which would be renamed new Table 3.12-1), "Alternate Shutdown Monitoring Instrumentation," be revised to identify "Rx23, near V-15-30" as the readout location for control rod drive system flow.

5. TS Tables 3.12-6 (which would become Table 3.12-1) and 4.12-1 be revised to delete the operability and surveillance requirement for the condensate transfer pump discharge pressure indicator.

6. TS Table 3.12-6 (which would become Table 3.12-1) incorrectly identifies instrument rack "RK05" as the readout location for shutdown cooling system flow.

4.0 EVALUATION

The license amendment request for OCNGS was reviewed against guidance provided in NRC Generic Letters 86-10 and 88-12. In addition, an assessment was made against the guidance provided in Generic Letter 81-12 which establishes the need to provide TS requirements for alternate shutdown equipment which was not previously contained in plant TS.

The licensee incorporated their fire protection program into Oyster Creek's Final Safety Analysis Report in January 1991. The operability and surveillance requirements for the detection systems, fire suppression systems, fire barriers and fire brigade staffing requirements as defined in the current TS were incorporated into the fire protection program.

In addition, current TS 6.5.3.2a states, "An independent fire protection and loss prevention program inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm." The licensee requested to delete the word "offsite" from the TS. Generic Letter 82-21 states that the annual audits may be performed by qualified utility personnel who are not directly responsible for the site fire protection program or by an outside independent fire protection consultant. The licensee had established procedures to ensure that the independence and effectiveness of the audit team by defining the composition of the audit team. The composition of the audit team will include either qualified utility personnel or an outside fire protection consultant. Based on the procedural controls and the composition of the audit team, this change would not reduce the quality or effectiveness of the annual audit and the guidance provided in Generic Letter 82-21 has been met.

The licensee requested TS Table 3.12-6 (which would be renamed Table 3.12-1), "Alternate Shutdown Monitoring Instrumentation" be revised to identify "Rx23 near V-15-30" as the readout location for control rod drive system flow. The licensee has stated that new flow indicator was installed near Valve V-15-30 as the new readout location. Further, the existing surveillance requirements of TS 4.12.1, which apply to the alternate shutdown monitoring instrumentation, would ensure the new flow indicator is capable of performing its intended function.

The licensee requested TS Tables 3.12-6 (which would become Table 3.12-1) and 4.12-1 be revised to delete the operability and surveillance requirement for the condensate transfer pump discharge pressure indicator. This proposed change would be consistent with the guidance provided in Generic Letter 86-10. The surveillance requirements would be relocated in the Plant Administrative procedures.

TS Table 3.12-6 (which would become Table 3.12-1) incorrectly identifies instrument rack "RK05" as the readout location for shutdown cooling system flow. The correct readout location is located on the 51 foot elevation of the reactor building near the reactor building closed cooling water heat exchangers. Accordingly, TS Table 3.12-6 (3.12-1) would be revised to identify "local" as the readout location for this parameter.

The licensee has stated in the April 20, 1992, submittal that the existing TS for the alternate shutdown monitoring instrumentation would be retained in the existing TS and do not propose the removal of the alternate shutdown instrumentation from the TS. This is consistent with the Generic Letters 81-12 and 88-12 guidance and is therefore acceptable.

Based on the review of the April 20, 1992, request for changes to the license and fire protection portions of the TS for OCNGS, the staff concluded that GPU Nuclear Corporation has followed the guidance provided by the NRC in Generic Letters 86-10 and 88-12 and the requested changes should be approved.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendment. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The amendment also relates to changes in recordkeeping, reporting, or administrative procedures or requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 20511). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

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Date: February 18, 1993