

October 14, 1998

Mr. Michael B. Roche
Vice President and Director
GPU Nuclear, Inc.
Oyster Creek Nuclear Generating Station
P.O. Box 388
Forked River, NJ 08731

SUBJECT: OYSTER CREEK - ISSUANCE OF AMENDMENT NO. 199
RE: AUTOMATIC DEPRESSURIZATION SYSTEM (TAC NO. MA3413)

Dear Mr. Roche:

The Commission has issued the enclosed Amendment No.199 to Facility Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station, in response to your application dated August 21, 1998.

The amendment removes the requirement for the Automatic Depressurization System (ADS) function of the Electromatic Relief Valves (EMRV) to be operable during Reactor Vessel Pressure Testing. Additionally, Note h of Technical Specification (TS) Table 3.1.1 is reworded to clarify the intent of the note, and will incorporate the correct time limit that was inadvertently changed due to a typographical error introduced in the issuance of Amendment 75.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Ronald B. Eaton, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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DFO

Docket No. 50-219

Enclosures: 1. Amendment No.199 to DPR-16
2. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION: See attached page

DOCUMENT NAME: G:\EATON\ma3413.AMD *SEE PREVIOUS CONCURRENCE

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NEW YORK STATE OFFICE OF NUCLEAR REGULATION

M. Roche
GPU Nuclear, Inc.

cc:

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DATED: October 14, 1998

AMENDMENT NO. 199 TO FACILITY OPERATING LICENSE NO. DPR-16-OYSTER CREEK

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

GPU NUCLEAR, INC.

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 199
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear, Inc. et al., (the licensee), dated August 21, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.199 , are hereby incorporated in the license. GPU Nuclear, Inc. shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Cecil O. Thomas, Director
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: October 14, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 199

FACILITY OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace the following pages of the Appendix A, Technical Specifications, with the attached pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3.1-16

3.4-3

3.4-4

Insert

3.1-16

3.4-3

3.4-4

TABLE 3.1.1 (CONT'D)

- * Action required when minimum conditions for operation are not satisfied. Also permissible to trip inoperable trip system. A channel may be placed in an inoperable status for up to six hours for required surveillance without placing the trip system in the tripped condition provided at least one OPERABLE instrument channel in the same trip system is monitoring that parameter.
- ** See Specification 2.3 for Limiting Safety System Settings.

Notes:

- a. Permissible to bypass, with control rod block, for reactor protection system reset in REFUEL MODE.
- b. Permissible to bypass below 600 psig in REFUEL and STARTUP MODES.
- c. One (1) APRM in each OPERABLE trip system may be bypassed or inoperable provided the requirements of Specification 3.1.C and 3.10.C are satisfied. Two APRM's in the same quadrant shall not be concurrently bypassed except as noted below or permitted by note.

Any one APRM may be removed from service for up to six hours for test or calibration without inserting trips in its trip system only if the remaining OPERABLE APRM's meet the requirements of Specification 3.1.B.1 and no control rods are moved outward during the calibration or test. During this short period, the requirements of Specifications 3.1.B.2, 3.1.C and 3.10.C need not be met.

- d. The IRMs shall be inserted and OPERABLE until the APRMs are OPERABLE and reading at least 2/150 full scale.
- e. Offgas system isolation trip set at $\leq 2,000$ mRem/hr. Air ejector isolation valve closure time delay shall not exceed 15 minutes.
- f. Unless SRM chambers are fully inserted.
- g. Not applicable when IRM on lowest range.
- h. With one or more instrument channel(s) inoperable in one ADS trip system, place the relay contact(s) for the inoperable initiation signal in the tripped condition within 4 days, or declare ADS inoperable and take the action required by Specification 3.4.B.3.

With one or more instrument channel(s) inoperable in both ADS trip systems, restore ADS initiation capability in at least one trip system within 1 hour, or declare ADS inoperable and take the action required by Specification 3.4.B.3.

Individual electromatic relief valve control switches shall not be placed in the "Off" position for more than 8 hours (total time for all control switches) in any 30-day period and only one relief valve control switch may be placed in the "Off" position at a time.

9. If Specifications 3.4.A.7 and 3.4.A.8 cannot be met, the requirements of Specification 3.4.A.6 will be met and work will be initiated to meet minimum operability requirements of 3.4.A.7 and 3.4.A.8.
10. The core spray system is not required to be operable when the following conditions are met:
 - a. The reactor mode switch is locked in the "refuel" or "shutdown" position.
 - b.
 - (1) There is an operable flow path capable of taking suction from the condensate storage tank and transferring water to the reactor vessel, and
 - (2) The fire protection system is operable.
 - c. The reactor coolant system is maintained at less than 212°F and vented (except during reactor vessel pressure testing).
 - d. At least one core spray pump, and system components necessary to deliver rated core spray flow to the reactor vessel, must remain operable to the extent that the pump and any necessary valves can be started or operated from the control room or from local control stations, and the torus is mechanically intact.
 - e.
 - (1) No work shall be performed on the reactor or its connected systems which could result in lowering the reactor water level to less than 4'8" above the top of the active fuel and the condensate storage tank level is greater than thirty (30) feet (360,000 gallons). At least two redundant systems including core spray pumps and system components must remain operable as defined in d. above

OR

- (2) The reactor vessel head, fuel pool gate, and separator-dryer pool gates are removed and the water level is above elevation 117 feet.

NOTE: When filling the reactor cavity from the condensate storage tank and draining the reactor cavity to the condensate storage tank, the 30 foot limit does not apply provided there is sufficient amount of water to complete the flooding operation.

B. Automatic Depressurization System

1. Five electromatic relief valves, which provide the automatic depressurization and pressure relief functions, shall be operable when the reactor water temperature is greater than 212°F and pressurized above 110 psig, except as specified in 3.4.B.2 and during Reactor Vessel Pressure Testing consistent with Specifications 1.39 and 3.3.A.(i).

2. If at any time there are only four operable electromatic relief valves, the reactor may remain in operation for a period not to exceed 3 days provided the motor operated isolation and condensate makeup valves in both isolation condensers are verified daily to be operable.
3. If Specifications 3.4.B.1 and 3.4.B.2 are not met; reactor pressure shall be reduced to 110 psig or less, within 24 hours.
4. The time delay set point for initiation after coincidence of low-low-low reactor water level and high drywell pressure shall be set not to exceed two minutes.

C. Containment Spray System and Emergency Service Water System

1. The containment spray system and the emergency service water system shall be operable at all times with irradiated fuel in the reactor vessel, except as specified in Specifications 3.4.C.3, 3.4.C.4, 3.4.C.6 and 3.4.C.8.
2. The absorption chamber water volume shall not be less than 82,000 ft³ in order for the containment spray and emergency service water system to be considered operable.
3. If one emergency service water system loop becomes inoperable, its associated containment spray system loop shall be considered inoperable. If one containment spray system loop and/or its associated emergency service water system loop becomes inoperable during the run mode, the reactor may remain in operation for a period not to exceed 7 days provided the remaining containment spray system loop and its associated emergency service water system loop each have no inoperable components and are verified daily to be operable.
4. If a pump in the containment spray system or emergency service water system becomes inoperable, the reactor may remain in operation for a period not to exceed 15 days provided the other similar pump is verified daily to be operable. A maximum of two pumps may be inoperable provided the two pumps are not in the same loop. If more than two pumps become inoperable, the limits of Specification 3.4.C.3 shall apply.
5. During the period when one diesel is inoperable, the containment spray loop and emergency service water system loop connected to the operable diesel shall have no inoperable components.
6. If primary containment integrity is not required (see Specification 3.5.A), the containment spray system may be made inoperable.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 199

TO FACILITY OPERATING LICENSE NO. DPR-16

GPU NUCLEAR, INC. AND

JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated August 21, 1998, GPU Nuclear, Inc., (the licensee) submitted a request for changes to the Oyster Creek Nuclear Generating Station (OCNGS) Technical Specifications (TSs). The requested changes would remove the requirement for the Automatic Depressurization System (ADS) function of the Electromatic Relief Valves (EMRVs) to be operable during Reactor Vessel Pressure Testing. Additionally, note h of Table 3.1.1 will be reworded to clarify the intent of the note, and will incorporate the correct time limit that was inadvertently changed due to a typographical error that was introduced in Amendment 75.

2.0 EVALUATION

The purpose of ADS is to depressurize the reactor pressure vessel during a small break loss-of-coolant-accident (LOCA) so that the low pressure emergency core cooling system (ECCS) can inject water into the core. ADS is required when the reactor water temperature is above 212 degrees F and the reactor pressure is above 110 psig. TS 3.3.A.(i) limits the Reactor Cooling System temperature to 250 degrees F during the reactor vessel pressure testing. Therefore, if a small break LOCA occurs during the testing, the resulting pressure would be limited to the saturation temperature at 250 degrees F, which is about 30 psia (approximately 15 psig). The reactor vessel pressure will be rapidly reduced to well below 110 psig without ADS. ADS is not required during the pressure vessel testing.

TS 3.5, Section 3 states that Primary containment integrity is not required during reactor vessel pressure testing. Since the containment may be open during the reactor vessel pressure testing, high containment pressure permissive will not be satisfied for the initiation of ADS. ADS function may be disabled during the vessel pressure testing. However, the present TS 3.4.B.1 requires ADS function during the reactor vessel pressure testing. The inconsistencies between the two TS sections are corrected by the proposed changes where there is no requirement for the ADS function during reactor vessel pressure testing. The proposed TS 3.4.B.1 changes are acceptable.

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The original OCNCS TS in Table 3.1.1, note h, included a requirement to limit bypassing the relief function of the EMRVs to 8 hours in a 30-day period. The statement "Relief Valve controllers shall not be bypassed..." led to some confusion because the EMRV control switches do not have a "Bypass" position. The proposed change replaces the terminology used in reference to the EMRVs with the appropriate terms. The wording change is only a clarification and does not change the intent of the TS. The proposed note h to TS Table 3.1.1 is acceptable. In addition, Amendment 75 inadvertently changed the 8 hours in a 30-day period to 3 hours due to a typographical error. The licensing basis for the plant is 8 hours versus the 3 hours stated in the existing note in the TS. Therefore, the revised note h to TS Table 3.1.1 will incorporate the correct time limit that was inadvertently changed in Amendment 75.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (63 FR 48527). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: G. Thomas

Date: October 14, 1998