

April 4, 1990

Docket No. 50-219

Mr. E. E. Fitzpatrick
Vice President and Director
Oyster Creek Nuclear Generating Station
Post Office Box 388 Forked River, New Jersey 08731

Dear Mr. Fitzpatrick:

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION - NOTICE OF CONSIDERATION
OF ISSUANCE OF AMENDMENT (TAC NO. 76132)

The Commission has requested the Office of the Federal Register to publish the enclosed "Notice of Consideration of Issuance of Amendment to Provisional Operating License and Opportunity for Hearing." This notice relates to your application for amendment dated March 2, 1990, which would revise Technical Specifications to accommodate implementation of a 21 month operating cycle with a 3 month outage, or a 24 month plant refueling cycle. Technical Specification Definition 1.12, Refueling Outage is revised to specify that refueling outage tests or surveillances shall be performed at least once per 24 months. In addition, the request also proposes to specify a 20 month interval, which is the existing refueling outage definition, for those surveillances currently specified on a refueling outage basis that have not yet been evaluated.

Sincerely,

/s/

Alexander W. Dromerick, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure: Notice

cc w/enclosure:
See next page

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Mr. E. E. Fitzpatrick
Oyster Creek Nuclear Generating Station

Oyster Creek Nuclear
Generating Station

cc:

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UNITED STATES NUCLEAR REGULATORY COMMISSIONGPU NUCLEAR CORPORATIONDOCKET NO. 50-219NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO
PROVISIONAL OPERATING LICENSE AND OPPORTUNITY FOR HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Provisional Operating License No. DPR-16, issued to GPU Nuclear Corporation (GPUN, the licensee), for operation of the Oyster Creek Nuclear Generating Station located in Ocean County, New Jersey.

The amendment would revise License Condition 2.C.7 to specify that the core spray sparger and repair assemblies inspections shall be performed at intervals not to exceed 20 months in lieu of each refueling outage. This change requires that the surveillance be performed at the currently defined refueling outage interval of 20 months since justification for extension to 24 months is not provided. This is considered an editorial change.

The amendment would also revise the Technical Specifications as follows:

1. Technical Specification Definition 1.12, REFUELING OUTAGE, is revised to specify that refueling outage tests or surveillances shall be performed at least once per 24 months. The existing provision, which allows refueling outage surveillances to be postponed to the next regularly scheduled outage when refueling outages occur within 8 months of the end of the previous refueling outage, is removed. The asterisked footnote is no longer applicable and is removed. Specific Technical Specifications affected by this definition change are listed below:

Table 4.1.1	Item 13.b	High Radiation in Main Steamline - instrument channel sensor calibration
Table 4.1.1	Item 15	High Radiation on Air Ejector Off-Gas-instrument channel calibration/test
Table 4.1.1	Item 20	High Temperature Main Steamline Tunnel - instrument channel test
Table 4.1.1	Item 27.b	Scram Trip Bypass instrument channel test
Table 4.1.1	Item 28.a	4.16 KV Emergency Bus Undervoltage (Loss of Voltage) instrument channel calibration
Table 4.1.1	Item 28.b	4.16 KV Emergency Bus Undervoltage (Degraded Voltage) instrument channel calibration
Table 4.1.1	Item 29	Drywell High Radiation instrument channel calibration/test
Table 4.1.2	Item 3	Containment Spray Trip System Test
Table 4.1.2	Item 4	Automatic Depressurization Trip System Test
Table 4.1.2	Item 5	MSIV Closure Trip System Test
Table 4.1.2	Item 6	Core Spray Trip System Test
Table 4.1.2	Item 7	Primary Containment Isolation Trip System Test
Table 4.1.2	Item 9	Isolation Condenser Actuation Trip System Test
Table 4.1.2	Item 12	Air Ejector Offgas Line Isolation Trip System Test
4.2.E.3		Standby Liquid Control System Functional Test
4.2.H		Scram Discharge Volume Drain and Vent Valve Operability Test
4.3.D		Reactor Coolant System Visual Examination
4.3.G		Primary Coolant System Pressure Isolation Valve Leak Test
4.4.A.1		Core Spray System Pump Operability Test

4.4.B.2		Automatic Depressurization System Automatic Actuation Test
4.4.C.1		Containment Cooling System Pump Operability Test
4.4.D.1		Emergency Service Water System Pump Operability Test
4.4.E.1		Control Rod Drive Hydraulic System pump operability
4.4.F.1		Fire Protection System pump and isolation valve operability
4.5.E		Type "B" and "C" Local Leak Rate Test (LLRT)
4.5.J.1		Containment Isolation Valve Automatic Closure Test
4.5.J.4.b		Reactor Building to Suppression Chamber Vacuum Breakers test and instrument calibration
4.5.J.5.b		Suppression Chamber - Drywell Vacuum Breakers Test, position indication and alarm test and calibration, and inspection
4.5.0		Instrument Line Flow Check Valve Test
4.5.P.2		Suppression Chamber interior visual inspection
4.7.A.2 & A.3		Diesel Generator start and load test, automatic start and sequence timer test, inspection, fuel supply check and starting batteries test/monitoring
4.7.A.5 & B.3		Station Battery capacity tests and low voltage communicator operability verification
4.8.A.4.a		Isolation Condenser isolation valve visual inspection and external leakage check
4.12.I		Alternate Shutdown Monitoring Instrumentation
Table 4.13-1	Item 5	Containment High Range Radiation Monitor
4.17		Control Room Heating, Ventilating, and Air-Conditioning System
6.15		Core Spray, Containment Spray, Isolation Condenser and Shutdown Cooling System Leak Tests

2. Table 4.1.1 Items 18, 20, 25 and 27.a are revised to specify that calibration and/or test of Condenser Low Vacuum, High Temperature Main Steamline Tunnel, Recirculation Loop Flow, and Scram Discharge Volume (Rod Block) - Water Level High Instrument channels, respectively, shall be performed once per 20 months in lieu of each refueling outage. This change requires that the surveillance be performed at the currently defined refueling outage interval of 20 months since justification for extension to 24 months is not provided. This is considered an editorial change.
3. Table 4.1.1 Items 28.a and 28.b are revised to extend the Loss of Power Instrument Channel Calibrations from once per 18 months to once per 24 months consistent with the 24 month plant operating cycle.
4. Table 4.1.2 Item 13 is revised to specify that the Containment Spray Trip System Test and the Containment Vent and Purge Isolation Trip System Test be performed once per 20 months in lieu of each refueling outage. This change requires that the surveillance be performed at the currently defined refueling outage interval of 20 months since justification for extension to 24 months is not provided. This is considered an editorial change.
5. Sections 4.2.A and 4.2.C.1 are clarified to specify that shutdown margin demonstration and control rod scram time tests be performed at an interval not do exceed 20 months. This change requires that these surveillances be performed at the currently defined refueling outage interval of 20 months since justification for extension to 24 months is not provided. This is considered an editorial change.

6. Section 4.4.B.1 is clarified to specify that the Automatic Depressurization System valve operability test be performed at intervals not to exceed 20 months. This change requires that the surveillance be performed at the currently defined refueling outage interval of 20 months since justification for extension to 24 months is not provided. This is considered an editorial change.
7. Section 4.5.J.4.B(4) is clarified to specify that the drywell to suppression chamber leak rate test be performed at intervals not to exceed 20 months. This change requires that the surveillance be performed at the currently defined refueling outage interval of 20 months as justification for extension to 24 months is not provided. This is considered an editorial change.
8. Section 4.5.K.2 is clarified to specify that the Standby Gas Treatment System differential pressure test for Reactor Building integrity be performed at intervals not to exceed 20 months. This change requires that this surveillance be performed at the currently defined refueling outage interval of 20 months since justification for extension to 24 months is not provided. This is considered an editorial change.
9. Section 4.7.A.3 is revised to extend the diesel generator inspection frequency from at least once per 18 months to at least once per 24 months consistent with the 24 month plant operating cycle.
10. Section 4.7.B.3 is revised to extend the station battery capacity tests from at least once per 18 months to at least once per 24 months consistent with the 24 month plant operating cycle.

11. Section 4.8.A.2 is clarified to specify that the Isolation Condenser System automatic actuation and functional test shall be performed at intervals not to exceed 20 months. This change requires that this surveillance be performed at the currently defined refueling outage interval of 20 months since justification for extension to 24 months is not provided. This is considered an editorial change.
12. Section 6.15.2(3) is revised to specify that the Reactor Water Cleanup System leak test to demonstrate integrity outside containment shall be performed at an interval not to exceed 20 months in lieu of refueling cycle intervals. This change requires that the surveillance be performed at the currently defined refueling outage interval 20 months since justification for extension to 24 months is not provided. This is considered an editorial change.

Prior to issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

By May 9, 1990, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and petitions for leave to intervene shall be filed in

accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, N.W., Washington, D.C. 20555 and at the Local Public Document Room located at Ocean County Library, Reference Department, 101 Washington Street, Toms River, New Jersey 08753. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as

to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to the first pre-hearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendments under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, 2120 L Street, N.W. Washington, D.C., by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 325-6000 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number 3737 and the following message addressed to John F. Stoitz: petitioner's name and telephone number; date petition was mailed; plant name; and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and to Ernest L. Blake, Jr., Esquire, Shaw, Pittman, Potts and Trowbridge, 200 N Street, N.W., Washington, D.C. 20037, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the

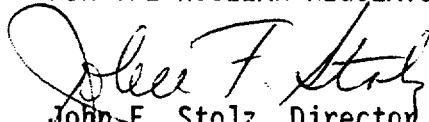
presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

If a request for a hearing is received, the Commission's staff may issue the amendment after it completes its technical review and prior to the completion of any required hearing if it publishes a further notice for public comment of its proposed finding of no significant hazards consideration in accordance with 10 CFR 50.91 and 50.92.

For further details with respect to this action, see the application for amendment dated March 2, 1990, which is available for public inspection at the Commission's Public Document Room, 2120 L Street, N.W., Washington, D.C. 20555, and at the Local Public Document Room, Ocean County Library, Reference Department, 101 Washington Street, Toms River, New Jersey 08753.

Dated at Rockville, Maryland, this 3rd day of April 1990.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation