

Mr. Michael B. Roche  
Vice President and Director  
GPU Nuclear, Inc.  
Oyster Creek Nuclear Generating Station  
P.O. Box 388  
Forked River, NJ 08731

October 28, 1999

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION - CORRECTION TO  
AMENDMENT NO. 208 (TAC NOS. MA4145 AND MA6074)

Dear Mr. Roche:

By letter dated June 2, 1999, the Commission issued Amendment 208 to the Oyster Creek Technical Specifications (TSS) which included wording for Note h which reflects the wording prior to Amendment 199. By letter dated July 14, 1999, you requested corrections to pages issued in previous amendments. By letter dated August 21, 1998, GPU Nuclear submitted Technical Specification Change Request (TSCR) 255 which, among other things, modified Note h of Table 3.1.1 of the Oyster Creek TSS. By letter dated October 14, 1998, the Commission issued Amendment 199 which granted the change.

On November 5, 1998, GPU Nuclear submitted TSCR 266 which was concerned with APRM/LPRM Safety Limits and Surveillance Requirements. The changes requested in TSCR 266 included several additions to Table 3.1.1 but did not modify Note h. However, GPU Nuclear used TSCR 266 as an opportunity to reformat and standardize Table 3.1.1 in its entirety. Although the NRC had issued Amendment 199, GPU Nuclear had not yet copied and distributed it at the time it submitted TSCR 266. As a result, the text of Note h included in TSCR 266 did not reflect the revised wording approved by Amendment 199.

Also, a review of the pages of Table 3.1.1 revealed an omission on the part of GPU Nuclear in its November 5, 1998, submittal. Specifically, on Sheet 7 of 13 (page 3.1-15), a reference to Note (aa) was inadvertently left off the submitted page.

A copy of TS pages 3.1-15 and 3.1-17 are enclosed. If you have any questions, please feel free to call me at (301) 415-1261.

Sincerely,

Original signed by:  
Helen N. Pastis, Sr. Project Manager, Section 1  
Project Directorate 1  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-219

Enclosure: As stated

cc w/encl: See next page

\* See previous concurrence

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

October 28, 1999

CHAIRMAN

Mr. Michael B. Roche  
Vice President and Director  
GPU Nuclear, Inc.  
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Sincerely,

A handwritten signature in blue ink, appearing to read "Helen N. Pastis, Sr.", written over a faint circular stamp.

Helen N. Pastis, Sr. Project Manager, Section 1  
Project Directorate 1  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

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cc w/encl: See next page

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**TABLE 3.1.1 PROTECTIVE INSTRUMENTATION REQUIREMENTS**  
Sheet 7 of 13

<u>Function</u>	<u>Trip Setting</u>	<u>Reactor Modes in which Function Must Be OPERABLE</u>				<u>Minimum Number of OPERABLE or OPERATING [tripped] Trip Systems</u>	<u>Minimum Number of Instrument Channels Per OPERABLE Trip System</u>	<u>Action Required*</u>
		<u>Shutdown</u>	<u>Refuel</u>	<u>Startup</u>	<u>Run</u>			
<b>M. Diesel Generator Load Sequence Timers (Cont'd.)</b>								
2. Service Water Pump (aa)	120 sec ± 15% (SK1A) (SK2A) 10 sec. ± 15% (SK7A) (SK8A)	X	X	X	X	2(o)	2(p)(kk)	Consider the pump inoperable and comply within 7 days (See note q)
3. Reactor Building Closed Cooling Water Pump (bb)	166 sec ± 15%	X	X	X	X	2(m)	1(n)(kk)	Consider the pump inoperable and comply within 7 days (See note q)
<b>N. Loss of Power</b>								
a. 4.16 KV Emergency Bus Undervoltage (Loss of Voltage)	**	X(ff)	X(ff)	X(ff)	X(ff)	2	1(kk)	
b. 4.16 KV Emergency Bus Undervoltage (Degraded Voltage)	**	X(ff)	X(ff)	X(ff)	X(ff)	2	3(kk)	See note cc
<b>O. Containment Vent and Purge Isolation</b>								
1. Drywell High Radiation	≤ 74.6 R/hr	X(u)	X(u)	X(u)	X	1	1	Isolate vent & purge pathways or PLACE IN COLD SHUTDOWN CONDITION

TABLE 3.1.1 (CONT'D)

Sheet 9 of 13

Individual electromechanical relief valve control switches shall not be placed in the "Off" position for more than 8 hours (total time for all control switches) in any 30-day period and only one relief valve control switch may be placed in the "Off" position at a time.

i. With two core spray systems OPERABLE:

1. A maximum of two core spray booster pump differential pressure (d/p) switches may be inoperable provided that the switches are in opposing ADS trip systems [i.e., only: either RV-40 A&D or RV-40 B&C]. Place the relay contacts associated with the inoperable d/p switch(es) in the de-energized position, within 24 hours. Restore the inoperable d/p switch(es) within 8 days, or declare ADS inoperable and take the action required by Specification 3.4.B.3;

or,

2. If two inoperable d/p switches are in the same ADS trip system [i.e., RV-40 A&B or RV-40 C&D], place the relay contacts associated with the inoperable d/p switch(es) in the de-energized position, within 24 hours. Restore the inoperable d/p switches within 4 days, or declare ADS inoperable and take the action required by Specification 3.4.B.3.

With only one core spray system OPERABLE:

If one or more d/p switches become inoperable in the OPERABLE core spray system, declare ADS inoperable and take the action required by Specification 3.4.B.3.

j. Not required below 40% of rated reactor thermal power.

k. All four (4) drywell pressure instrument channels may be made inoperable during the integrated primary containment leakage rate test (See Specification 4.5), provided that the plant is in the COLD SHUTDOWN condition and that no work is performed on the reactor or its connected systems which could result in lowering the reactor water level to less than 4'8" above the TOP OF THE ACTIVE FUEL.

l. Bypass in IRM Ranges 8,9, and 10.

m. There is one time delay relay associated with each of two pumps.

n. One time delay relay per pump must be OPERABLE.

OYSTER CREEK NUCLEAR GENERATING STATION - CORRECTION TO AMENDMENT NO. 208  
(TAC NOS. MA4145 AND MA6074)

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