

May 28, 1986

Docket No. 50-219

Mr. P. B. Fiedler  
Vice President and Director  
Oyster Creek Nuclear Generating Station  
Post Office Box 388  
Forked River, New Jersey 08731

Dear Mr. Fiedler:

SUBJECT: STANDBY GAS TREATMENT SYSTEM (TAC 60764)

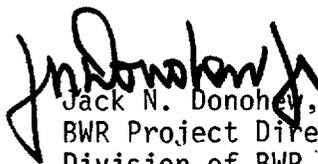
Re: Oyster Creek Nuclear Generating Station

The Commission has issued the enclosed Amendment No.103 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment is in response to your application dated January 30, 1986.

This amendment authorizes changes to Section 3.5 of the Appendix A Technical Specifications (TS) pertaining to the limiting conditions for operation (LCO) when one of the two trains of the Standby Gas Treatment System (SBGTS) is declared inoperable. The changes are to the power operation and the refueling reactor operation modes in TS 3.5.B.3.a.1 and 3.5.B.3.b.1, respectively. The change which is the same for both reactor operation modes would lessen the requirement on when the other SBGTS must be demonstrated operable. The change is from within 2 hours in the previous TS to within 2 hours unless significant painting, fire, or chemical release has taken place in the building within 12 hours and then within 1 hour of the end of the 12-hour period.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notices.

Sincerely,



Jack N. Donohue, Jr., Project Manager  
BWR Project Directorate #1  
Division of BWR Licensing

Enclosures:

1. Amendment No.103 to License No. DPR-16
2. Safety Evaluation

cc w/enclosures:  
See next page

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Mr. P. B. Fiedler  
Oyster Creek Nuclear Generating Station

Oyster Creek Nuclear  
Generating Station

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 103  
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by GPU Nuclear Corporation and Jersey Central Power and Light Company (the licensees) dated January 30, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.103, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Jack N. Donohew, Jr., Project Manager  
BWR Project Directorate #1  
Division of BWR Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: May 28, 1986.

ATTACHMENT TO LICENSE AMENDMENT NO. 103

PROVISIONAL OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain vertical lines indicating the area of change.

REMOVE

3.5-4

INSERT

3.5-4

3. With one standby gas treatment system circuit inoperable:

a. During Power Operation:

1. Demonstrate the operability of the other standby gas treatment system circuit within 2 hours unless significant painting, fire, or chemical release has taken place in the reactor building within the previous 12 hours. In this event, demonstration of operability shall take place within 1 hour of the expiration of the 12 hour period, and

2. Continue to demonstrate the operability of the standby gas treatment system circuit once per 24 hours until the inoperable standby gas treatment circuit is returned to operable status.

3. Restore the inoperable standby gas treatment circuit to operable status within 7 days or be subcritical with reactor coolant temperature less than 212°F within the next 36 hours.

b. During Refueling:

1. Demonstrate the operability of the redundant standby gas treatment system within 2 hours unless significant painting, fire, or chemical release has taken place in the reactor building within the previous 12 hours. In this event, demonstration of operability shall take place within 1 hour of the expiration of the 12 hour period, and

2. Continue to demonstrate the operability of the redundant standby gas treatment system once per 7 days until the inoperable system is returned to operable status.

3. Restore the inoperable standby gas treatment system to operable status within 30 days or cease all spent fuel handling, core alterations or operation that could reduce the shutdown margin (excluding reactor coolant temperature changes).

4. If Specifications 3.5.B.2 and 3.5.B.3 are not met, reactor shutdown shall be initiated and the reactor shall be in the cold shutdown condition within 24 hours and the condition of Specification 3.5.B.1 shall be met.

Bases:

Specifications are placed on the operating status of the containment systems to assure their availability to control the release of any radioactive materials from irradiated fuel in the event of an accident condition. The primary containment system (1) provides a barrier against uncontrolled release of fission products to the environs in the event of a break in the reactor coolant systems.

Whenever the reactor coolant water temperature is above 212°F, failure of the reactor coolant system would cause rapid expulsion of the coolant from the reactor with an associated pressure rise in the primary containment. Primary containment is required, therefore, to contain the thermal energy of the expelled coolant



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 103 TO PROVISIONAL OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND  
JERSEY CENTRAL POWER & LIGHT COMPANY  
OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated January 30, 1986, GPU Nuclear (the licensee) has requested an amendment to Provisional Operating License No. DPR-16 for Oyster Creek Nuclear Generating Station (Oyster Creek). This amendment would authorize changes to Section 3.5 of the Appendix A Technical Specifications (TS) pertaining to the limiting conditions for operation (LCO) when one of the two trains of the Standby Gas Treatment System (SBGTS) is declared inoperable. The changes are to the power operation and the refueling reactor operation modes in TS 3.5.B.3.a.1 and 3.5.B.3.b.1, respectively.

The change which is the same for both reactor operation modes would lessen the requirement on when the other SBGTS must be demonstrated operable. The change is from within 2 hours in the current TS to within 2 hours unless significant painting, fire, or chemical release has taken place in the building within 12 hours and then within 1 hour of the end of the 12-hour period.

2.0 DISCUSSION AND EVALUATION

The licensee has proposed its Technical Specification Change Request (TSCR) 133 to allow a delayed demonstration of the operability of the redundant SBGTS after one of the two SBGTS is declared inoperable. TSCR 133 would delay the demonstration of the operable SBGTS train if significant painting, fire, or chemical release has taken place in the Reactor Building within the previous 12 hours. Current TS surveillance tests which are not being changed by this TSCR provide adequate assurance that the SBGTS will operate upon demand. The proposed TSCR would increase plant safety by increasing system availability and avoiding unnecessary degradation of the system from a significant painting, fire, or chemical release.

The current TS for demonstrating the operability of the SBGTS, upon the loss of one train, do not address circumstances where significant painting, fire, or chemical release has taken place in the Reactor Building within the previous 12 hours. TS 4.5.K.1.a requires certain tests of the SBGTS following significant painting, fire, or chemical release in the Reactor

Building while the SBGTS was in operation. Following the current TS requirements for demonstrating operability could, under these conditions in the Reactor Building, degrade the SBSTS unnecessarily and would not enhance the safety of the plant. In addition, demonstrating operability during these conditions would require subsequent compliance with TS 4.5.K.1.a and declaring the second SBGTS inoperable until the tests of the charcoal were known. This would reduce system availability and may require an unnecessary plant shutdown.

The function of the SBGTS is to treat and exhaust the atmosphere of the reactor building to the stack during containment isolation conditions during a loss-of-coolant accident (LOCA) and a fuel handling accident with a minimum release of radioactive material to the environments. Two separate filter trains are provided, each having 100% capacity. The SBGTS has particulate and charcoal filters which can be damaged by fumes from painting, fires or chemical releases.

The licensee stated Inspection Report 50-219/84-11 identified the need for a change in the current TS based upon Licensee Event Report (LER) 84-7 on the licensee's failure to test an SBGTS within the required time. In this inspection report on LER 84-7, it was stated that on April 2, 1984, Diesel Generator No. 1 (DG-1) was declared inoperable as a result of a failure to fast start during the monthly surveillance. This resulted in the associated SBGTS-1 being declared inoperable, because DG-1 is the emergency power supply for SBGTS-1. TS 3.5.B.3.b.1 then required demonstration of the operability of the redundant SBGTS (in this case SBGTS-2) within 2 hours. At the time of DG-1 failure, torus painting was in progress. The painting was stopped but the SBGTS operability tests were delayed for 10 hours in accordance with station procedures to prevent degradation of the charcoal filters from absorption of paint fumes.

The BWR Standard Technical Specifications applicable to Oyster Creek (BWR-STs, NUREG-0123, Revision 1) do not require the redundant SBGTS to be tested after an SBGTS has been declared inoperable for the power operation and refueling modes. The BWR-STs require the inoperable SBGTS to be operable within 7 days (power operation) or 30 days (refueling) as does the current TS which are not changed by this action. The BWR-STs require testing the SBGTS following painting, fire, or chemical release in any ventilation zone communicating with the SBGTS because the fumes may damage the SBGTS filters to the point it could be inoperable.

Therefore, based on the above, the staff concludes that operation of Oyster Creek with this proposed TSCR 133 is acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational

radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

#### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: J. Donohew

Dated: May 28, 1986.