

December 27, 1984

Amendment 78 to
DPR-16

Docket No. 50-219
LS05-84-12-021

Mr. P. B. Fiedler
Vice President and Director
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

Dear Mr. Fiedler:

SUBJECT: ADMINISTRATIVE CONTROLS

Re: Oyster Creek Nuclear Generating Station

DISTRIBUTION

Docket	PFMcKee
NRC PDR	ACRS (10)
Local PDR	SEPR
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CJamerson	ERutcher
PM	WJones
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The Commission has issued the enclosed Amendment No. 78 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment consists of changes to the Technical Specifications in response to your application dated June 8, 1984.

The amendment authorizes changes to the Appendix A Technical Specifications, Section 6.0, Administrative controls.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on November 21, 1984 (49 FR 45953). No request for hearing and no comments were received.

A copy of our related Safety Evaluation is also enclosed. A notice of issuance pertaining to this action will appear in the Commission's next Monthly Notice publication in the Federal Register.

Sincerely,

Original signed by

John A. Zwolinski, Chief
Operating Reactors Branch #5
Division of Licensing

Enclosures:

- Amendment No. 78 to License No. DPR-16
- Safety Evaluation

cc w/enclosures:
See next page

DL:ORB#5
CJamerson:jb
12/4/84

DL:ORB#5
JDShohew
12/4/84

DL:ORB#5
JZwolinski
12/3/84

*copy - conditioned on
no issuance until
operation of
notice period.*
OELD
J. GRAY GD
12/7/84

DL:ORB#5
DCrutchfield
12/19/84

*Make sure you
get all the
changes
JES
12/10*

SED
11 Ex(51)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION
AND
JERSEY CENTRAL POWER AND LIGHT COMPANY
OYSTER CREEK NUCLEAR GENERATING STATION
AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 78
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear Corporation and Jersey Central Power and Light Company (the licensees) dated June 8, 1984 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter J;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations.
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

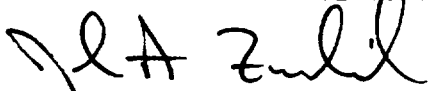
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 78, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John A. Zwolinski, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance:

ATTACHMENT TO LICENSE AMENDMENT NO. 78

PROVISIONAL OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by the captioned amendment number and contain vertical lines indicating the area of change.

Remove Pages

6-4

6-5

6-6

6-7

6-15

6-22

6-23

Replace Pages

6-4

6-5

6-6

6-7

6-15

6-22

6-23

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1

The members of the facility staff shall meet or exceed the following qualifications:

Vice President & Director/Deputy Director

Requirements: Ten years total power plant experience of which three years must be nuclear power plant experience. A maximum of four years of academic training may fulfill four of the remaining seven years of required experience. Both must be capable of obtaining or possess a Senior Reactor Operator's License.

Plant Operations Director

Requirements: Eight years total power plant experience of which three years must be nuclear power plant experience. A maximum of two years of academic or related technical training may fulfill two years of the remaining five years of required experience. The Plant Operations Director must be capable of obtaining or possess a Senior Reactor Operator's License.

Plant Engineering Director

Requirements: Eight years of responsible positions related to power generation, of which three years shall be nuclear power plant experience. A maximum of four of the remaining five years of experience may be fulfilled by satisfactory completion of academic or related technical training.

Manager-Plant Administration

Requirements: Eight years total power plant experience of which four years must have been in nuclear power plant experience. The Manager should possess a four year college degree or equivalent in Business Administration or an Engineering discipline.

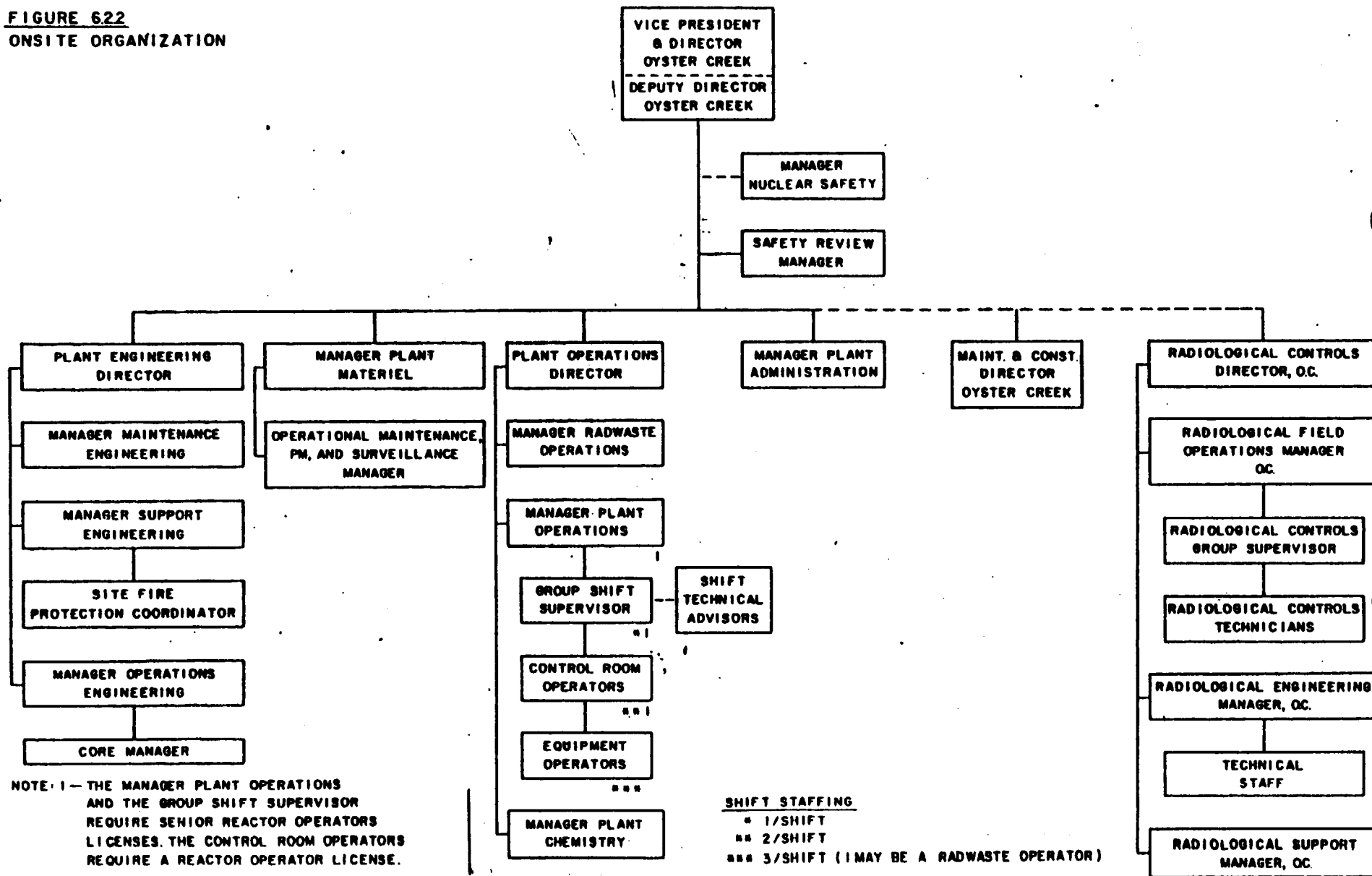
Manager-Plant Operations

Requirements: Eight years total power plant experience of which three years must be nuclear power plant experience. A maximum of two years of academic or related technical training may fulfill two of the remaining five years of required experience. The Manager Plant Operations must possess a Senior Reactor Operator's License.

Manager-Plant Chemistry

Requirements: Five years experience in chemistry of which a minimum of one year shall be in radiochemistry at an operating nuclear power plant. A maximum of four years of this five year experience may be fulfilled by related technical or academic training.

FIGURE 6.22
ONSITE ORGANIZATION



6-4

Amendment No. 58, 59, 68, 69, 78

Safety Review Manager

Requirements: Eight years total power plant experience of which three years must be nuclear power plant experience. A maximum of two years of academic or related technical training may fulfill two of the remaining five years of required experience.

Manager-Plant Materiel

Requirements: Seven years of total power plant experience of which one year must be nuclear power plant experience. Two years of academic or related technical training may fulfill two of the remaining six years of required experience.

Area Supervisor-Instrument and Computer Maintenance

Requirements: Five years of experience in instrumentation and control, of which a minimum of one year shall be in nuclear instrumentation and control at an operating nuclear power plant. A maximum of four years of this five year experience may be fulfilled by related technical or academic training.

Managers-Plant Engineering

The engineers in charge of technical support shall have a Bachelor's Degree in Engineering or the Physical Sciences and have three years of professional level experience in nuclear services, nuclear plant operation, or nuclear engineering, and the necessary overall nuclear background to determine when to call consultants and contractors for dealing with complex problems beyond the scope of owner-organization expertise.

Core-Manager

At the time of initial core loading or appointment to the position, whichever is later, the responsible person shall have a Bachelor's Degree in Engineering or the Physical Sciences and four years experience or a graduate degree and three years experience. Two of these years shall be nuclear power plant experience. The experience shall be in such areas as reactor physics, core measurements, core heat transfer, and core physics testing programs. Successful completion of a reactor engineering training program (such as the 12 week concentrated programs offered by NSS Vendors) may be equivalent to one year's nuclear power plant experience.

Radiological Controls Director (Reports Offsite)

Requirements: Bachelor's degree or the equivalent in a science or engineering subject, including some formal training in radiation protection. Five years of professional experience in applied radiation protection. (Master's degree equivalent to one year experience and Doctor's degree equivalent to two years experience where coursework related to radiation protection is involved.) Three years of this

professional experience should be in applied radiation protection work in a nuclear facility dealing with radiological problems similar to those encountered in nuclear power stations.

M&C Director, O.C.

Requirements: Seven years of total power plant experience of which one year must be nuclear power plant experience. Two years of academic or related technical training may fulfill two of the remaining six years of required experience.

Shift Technical Advisor

Requirements: Bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.3.2

Each member of the radiation protection organization for which there is a comparable position described in ANSI N18.1-1971 shall meet or exceed the minimum qualifications specified therein, or in the case of radiation protection technicians, they shall have at least one year's continuous experience in applied radiation protection work in a nuclear facility dealing with radiological problems similar to those encountered in nuclear power stations. and shall have been certified by the Radiological Controls Director, as qualified to perform assigned functions. This certification must be based on an NRC approved, documented program consisting of classroom training with appropriate examinations and documented positive findings by responsible supervision that the individual has demonstrated his ability to perform each specified procedure and assigned function with an understanding of its basis and purpose.

6.4 TRAINING

6.4.1

A retraining program for operators shall be maintained under the direction of the Manager Plant Training Oyster Creek and shall meet the requirements and recommendation of Appendix A of 10CFR Part 55. Replacement training programs, the content of which shall meet the requirements of 10CFR Part 55, shall be conducted under the direction of the Manager Plant Training Oyster creek for licensed operators and Senior Reactor Operators.

6.4.2

A training program for the Fire Brigade shall be maintained under the direction of the Manager Plant Training Oyster Creek.

6.7 SAFETY LIMIT VIOLATION

6.7.1

The following actions shall be taken in the event a Safety Limit is violated:

- a. If any Safety Limit is exceeded, the reactor shall be shut down immediately until the Commission authorizes the resumption of operation.
- b. The Safety Limit violation shall be reported to the Commission and the Vice President and Director Oyster Creek.
- c. A Safety Limit Violation Report shall be prepared. The report shall be submitted to the Vice President and Director Oyster Creek. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission within 10 days of the violation.

6.8 PROCEDURES

6.8.1

Written procedures shall be established, implemented, and maintained that meet or exceed the requirements of the Nuclear Regulatory Commission's Regulatory Guide 1.33 (the applicable revision is identified in the GPU Nuclear Operational Quality Assurance Plan) and as provided in 6.8.2 and 6.8.3 below.

6.8.2

Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed as described in 6.5.1.1 and approved as described in 6.5.1 prior to implementation and periodically as specified in the Administrative Procedures.

6.8.3

Temporary changes to procedures 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of GPUNC Management Staff authorized under Section 6.5.1.12 and knowledgeable in the area affected by the procedure. For changes which may affect the operational status of facility systems or equipment, at least one of these individuals shall be a member of facility management or supervision holding a Senior Reactor Operator's License on the facility.
- c. The change is documented, subsequently reviewed and approved as described in 6.8.2 within 14 days of implementation.

6.10.2

The following records shall be retained for the duration of the Facility Operating License:

- a. Record and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.
- d. Records of radiation exposure for all individuals entering radiation control areas.
- e. records of gaseous and liquid radioactive material released to the environs.
- f. Records of transient or operational cycles for those facility components designed for a limited number of transients or cycles.
- g. Records of training and qualification for current members of the plant staff.
- h. Records of inservice inspections performed pursuant to these technical specifications.
- i. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- j. Records of reviews by the Independent Onsite Safety Review Group.
- k. Records for Environmental Qualification which are covered under the provisions for paragraph 6.14.

6.10.3

Quality Assurance Records shall be retained as specified by the Quality Assurance Plan.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 (Deleted)

20 estimates of the likely resultant exposure to individuals and to population groups, and assumptions upon which estimates are based shall be provided.

(c). If statistically significant variations of offsite environmental concentrations with time are observed, correlation of these results with effluent release shall be provided.

(d). Results of required leak tests performed on sealed sources if the tests reveal the presence of 0.005 microcuries or more of removable contamination.

d. Inoperable Fire Protection Equipment (3.12)

e. Core Spray Sparger Inservice Inspection (Table 4.3.1-9)

Prior to startup of each cycle, a special report presenting the results of the inservice inspection of the Core Spray Spargers during each refueling outage shall be submitted to the Commission for review.

f. Failures and challenges to Relief and Safety Valves

Failures and challenges to Relief and Safety Valves which do not constitute an LER will be the subject of a special report submitted to the Commission within 60 days of the occurrence. A challenge is defined as any automatic actuation (other than during surveillance or testing) of Safety or Relief Valves.

6.10 RECORD RETENTION

6.10.1

The following records shall be retained for at least five years:

a. Records and logs of facility operation covering time interval at each power level.

b. Records and logs of principle maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.

c. Reportable occurrence reports.

d. Records of surveillance activities, inspections and calibrations required by these technical specifications.

e. Records of reactor tests and experiments.

f. Records of changes made to operating procedures.

g. Records of radioactive shipments

h. Records of sealed source leak tests and results.

i. Records of annual physical inventory of all source material of record.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 78 TO PROVISIONAL OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND
JERSEY CENTRAL POWER AND LIGHT COMPANY
OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated June 8, 1984, GPU Nuclear Corporation (GPU) (the licensee) requested an amendment to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment would authorize changes to the Technical Specifications (TS), Section 6.0, Administrative Controls.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on November 21, 1984 (49 FR 45953). No request for hearing or public comments were received.

2.0 DISCUSSION AND EVALUATION

The proposed amendment would authorize changes to Appendix A of the TS in four areas. The first area involves changes to the Plant Engineering Organization to allow it to be more responsive to operations and maintenance needs, and to enhance the administrative capabilities of the organization. The second area involves upgrading the position of Manager, Radiological Controls to the Director level. The third area involves updating the requirements for written procedures in effect for Oyster Creek and the fourth area involves the addition of the NUREG-0737 (Clarification of TMI Action Plan Requirements) requirement for a special report to be submitted to the NRC following the failure of, or challenge to, relief and safety valves.

The Plant Engineering organization will be restructured as depicted in revised Figure 6.2.2 and as described in revised Section 6.3.1. The responsibilities of the Manager-Plant Engineering will be split into three separate areas: Operations Engineering, Maintenance Engineering and Support Engineering. Each of the three areas will be headed by a Manager

with an appropriate Technical/Administrative background and capabilities as specified in the Oyster Creek TS. This structure provides increased time on the managerial level for both technical and administrative support to the Supervisory/Engineering personnel. Since the technical, educational, administrative and experience requirements for these three managerial positions have not been changed, the staff finds this proposed administrative change acceptable.

The Fire Protection Supervisor title will be revised to Site Fire Protection Coordinator since this position no longer directly supervises the activities of fire protection personnel. This position reports to the Manager-Support Engineering, but continues direct access to the Plant Engineering Director providing an additional level of technical support to this function. Since the technical responsibilities of the position remain the same, the staff finds this proposed administrative change acceptable.

The title of Manager-Core Engineering will be revised to Core Manager. This position reports directly to the Manager-Operations Engineering and relieves this position of some administrative responsibilities which allows more time for technical duties. An additional level of expertise is also provided to the Plant Engineering Director. The description of the Core Manager has been placed in a new place on TS page 6-6 because the position now ranks at a different level in the facility staff organization. Since the technical responsibilities of this position remain as specified in Section 6.3, the staff finds this proposed administrative change acceptable.

The position of Chemistry Manager no longer falls under the cognizance of the Plant Engineering organization. This position now reports to the Plant Operations Director and is subsequently deleted from Plant Engineering; this title has also been changed to Manager-Plant Chemistry. The description of the Manager-Plant Chemistry has been moved from the bottom of TS page 6-6 to the bottom of TS page 6-5 because the position now ranks at a higher level in the facility staff organization. Since the qualifications of this position remain the same and the position's reporting requirements have not been downgraded, the staff finds this administrative change acceptable.

Another change to the onsite organization at Oyster Creek is that the position of Manager/Deputy Radiological Controls has been upgraded to the Director level. The title of the position will be revised from Manager/Deputy Radiological Controls to Radiological Controls Director. This revision in titles will also be made in TS Section 6.3.2 which has a reference to the Manager/Deputy Radiological Controls. The TS requirements and reporting requirements remain the same for this position; therefore the staff finds these administrative changes acceptable.

Under current TS requirements of Section 6.8, written procedures shall be established, implemented, and maintained that meet or exceed the requirements of Sections 5.2 and 5.3 of ANSI N18.7-1976 and Appendix A of Regulatory Guide 1.33-1972. However, due to updated requirements for written procedures, Oyster Creek has proposed committing to Regulatory Guide 1.33-1978. The licensee has proposed to list the Regulatory 1.33 in TS Section 6.8.1 without the date of the approved revision. Instead, the licensee has proposed the following: "the applicable revision is identified in GPU Nuclear Operational Quality Assurance Plan." The applicable revision date is 1978. Paragraph 10 CFR 50.54(a)(3) states that changes to a quality assurance program description which reduce commitments accepted by the NRC, in this case the applicable revision date for Regulatory Guide 1.33, must be submitted to NRC and receive NRC approval before implementation. In addition, the TS should refer to the "Nuclear Regulatory Commission's Regulatory Guide 1.33" and not to the "Nuclear Regulatory Guide 1.33." The licensee agreed to this non-substantive change in a meeting at the plant site on November 28, 1984. The staff finds these administrative changes to be acceptable.

The final change is to add the NUREG-0737 requirements for special reports that are to be submitted to the NRC after failure of, or challenge to, relief and safety valves. The model TS for these reporting requirements were described in the NRC's Generic Letter No. 82-16. However, subsequent issuance of the new "LER rule" (10 CFR 50.73) supersedes the model reporting requirement of most events that may be attributable to or associated with failures of relief and safety valves. Therefore, Oyster Creek has proposed reporting failures and challenges to relief and safety valves, which do not constitute an LER, within 60 days of the occurrence. This proposed administrative change is consistent with the intent of the NUREG-0737 requirement and is considered acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

5.0 ACKNOWLEDGEMENT

This evaluation was prepared by R. Urban (Region I).

Dated: December 27, 1984.

December 27, 1984

cc

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