June 28, 1988

Docket No. 50-219

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Mr. E. E. Fitzpatrick
Vice President and Director
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

Dear Mr. Fitzpatrick:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 67964)

The Commission has issued the enclosed Amendment No.123 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station, in response to your application dated April 29, 1988, as supplemented May 11, 1988.

The amendment revises Technical Specifications 3.13.B.2 and 3.13.B.3 by adding a note which would permit a one time change, for current operating Cycle 11 only, which allows continued power operation if both of the primary and backup safety valve position indicators become inoperable on no more than two safety valves.

The amendment also makes an administrative change to Technical Specification Sections 3.13.A, 3.13.B and 3.13.C to capitalize Technical Specifications definitions where they appear in these sections.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

original signed by

Alexander W. Dromerick, Project Manager
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No.123 to DPR-16
2. Safety Evaluation

cc w/enclosures:
See next page

LA:PDI-4	PM:PDI-4	D:PDI-4	OGC SXRB
SNorris	ADromerick:bd	JStolz	WHodges
06/14/88	06/14/88	06/15/88	06/16/88 6/15/88

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 123
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by GPU Nuclear Corporation, et al., (the licensee), dated April 29, 1988, as supplemented May 11, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.123 , are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 28, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 123

PROVISIONAL OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove	Insert
Page 3.13-1	3.13-1
Page 3.13-2	3.13-2

3.13 ACCIDENT MONITORING INSTRUMENTATION

Applicability: Applies to the operating status of accident monitoring instrumentation.

Objective: To assure operability of accident monitoring instrumentation.

Specification:

A. Relief Valve Position Indicators

1. The accident monitoring instrumentation channels shown in Table 3.13.1 shall be OPERABLE when the mode switch is in the Startup or Run positions.

2. With the number of OPERABLE accident monitoring instrumentation channels less than the Total Number of Channels shown in Table 3.13.1, either restore the inoperable channels to OPERABLE status within 7 days, or place the reactor in the SHUTDOWN CONDITION within the next 24 hours.

3. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels Operable requirements of Table 3.13.1, either restore the inoperable channel(s) to the OPERABLE status within 48 hours, or place the reactor in the SHUTDOWN CONDITION within the next 24 hours.

B. Safety Valve Position Indicators

1. During POWER OPERATION, both primary* and backup** safety valve monitoring instruments are required to be OPERABLE except as provided in 3.13.B.2 and 3.13.B.3.

2. If either the primary* or backup** accident monitoring instruments on a valve become inoperable, the primary* accident monitoring instrument on an adjacent valve must be OPERABLE, and its set point appropriately reduced. (see NOTE below)

3. If both the primary* and backup** accident monitoring instruments on a valve become inoperable and the primary* accident monitoring instrument on an adjacent valve is OPERABLE, either restore the inoperable channel(s) to an OPERABLE status within 7 days, or place the reactor in the SHUTDOWN CONDITION within the next 24 hours. (see NOTE below)

*Acoustic Monitor
**Thermocouple

NOTE: During Cycle 11 only, the provisions of Specification 3.13.B.2 do not apply to backup** accident monitoring instruments. In addition, continued POWER OPERATION is permitted if both the primary* and backup** accident monitoring instruments become inoperable on a maximum of two (2) safety valves. The 7 day action statement in Specification 3.13.B.3 will commence if both instruments on a third valve become inoperable.

4. If the requirements of Section 3.13.B.2 or 3.13.B.3 cannot be met within 48 hours, place the reactor in the SHUTDOWN CONDITION within the next 24 hours.

C. In the event that any of these monitoring channels become inoperable, they shall be made OPERABLE prior to startup following the next COLD SHUTDOWN.

D. Wide Range Torus Water Level Monitor

1. Two wide range torus water level monitor channels shall be continuously indicated in the control room during Power Operation.

2. With the number of operable accident monitoring channels less than the total Number of Channels shown in Table 3.13.1, restore the inoperable channel(s) to Operable status within 7 days or place the reactor in the shutdown condition within the next 24 hours.

3. With the number of operable accident monitoring instrumentation channels less than the Minimum Channels operable requirements of Table 3.13.1, restore the inoperable channel(s) to operable status within 48 hours or place the reactor in the shutdown condition within the next 24 hours.

E. Wide Range Drywell Pressure Monitor

1. Two Wide Range Drywell Pressure monitor channels shall be continuously indicated in the control room during Power Operation.

2. With the number of operable accident monitoring channels less than the total Number of Channels shown in Table 3.13.1, restore the inoperable channel(s) to Operable status within 7 days or place the reactor in the shutdown condition within the next 24 hours.

3. With the number of operable accident monitoring instrumentation channels less than the Minimum Channels operable requirements of 3.13.1, restore the inoperable channel(s) to operable status within 48 hours or place the reactor in the shutdown condition within the next 24 hours.

F. Drywell H_2 Monitor

1. Two drywell hydrogen monitor channels shall be capable of continuously indicating in the control room during power operation.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 123

TO PROVISIONAL OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND
JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

INTRODUCTION

By letters dated April 29, 1988, and May 11, 1988, GPU Nuclear Corporation (GPUN or the licensee) requested an Amendment to the Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station (Oyster Creek or OCNGS).

The proposed changes affect Technical Specification pages 3.13-1 and 3.13-2. Specifically, the revised change would add a note associated with Specifications 3.13.B.2 and 3.13.B.3. The note would permit a one time change, for current operating Cycle 11 only, which allows continued power operation if both of the primary and backup safety valve position indicators become inoperable on no more than two safety valves. In addition, the requirement to reduce the setpoint of the acoustic monitor on an adjacent safety valve would be changed so that it compensates for the inoperability of an acoustic monitor only. The note would also state that the 7 day action statement in Specification 3.13.B.3 would commence should both primary and backup devices on a third safety valve become inoperable.

Administrative changes to capitalize the Technical Specification definitions where used in specifications in Sections 3.13.A, 3.13.B and 3.13.C are also proposed. There are no changes to safety valve position indicator surveillance requirements associated with this proposal.

EVALUATION

With regard to the above proposed amendment, the licensee has stated that these changes would still ensure that position indication coverage is retained for all safety valves. This would be accomplished by a combination of operable acoustic monitors and reduced setpoints on adjacent valves' acoustic monitors.

In the proposal the licensee also indicates that in the symptom-based emergency operating procedures implemented at Oyster Creek, the operators response is governed by changes in plant parameters (symptoms) and not by what caused those parameters to change. Consequently, operator response to a stuck open safety valve is not affected or dependent on the operability of the valve acoustic monitors or thermocouples. Thus, operability of the safety valve position indication is not necessary for transient or accident mitigation.

The staff has considered the licensee's statements and has also noted the following.

1. It is expected that leakage occurring prior to both primary and backup monitor failure would be detected.

2. The proposed Technical Specification continues to specify compensating adjustments in acoustic monitors when primary monitors fail.

3. The most likely initiator of safety valve leakage is challenge to the valves - spontaneous leakage is considered unlikely.

4. The likelihood of a scenario which would challenge safety valves during the remainder of Cycle 11 is remote.

5. The likelihood of both Items 3 and 4 occurring (and further, resulting in valve leakage) coincident with inoperability of both primary and backup monitor on that same valve is particularly remote.

The staff concludes from the above discussion that the proposed one-time change does not pose a significant risk to the health and safety of the public, and is, therefore, acceptable. The staff also concludes that changes to capitalize the Technical Specification definitions when use in specification in Section 3.13.A, 3.13.B and 3.13.C are administrative changes and are acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Dated: June 28, 1988

Principal Contributors

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