



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

March 11, 1985

Docket No. 50-219

MEMORANDUM FOR: John A. Zwolinski, Chief
Operating Reactors Branch No. 5, DL

FROM: Jack N. Donohew, Project Manager
Operating Reactors Branch No. 5, DL

SUBJECT: MEETING WITH GPU NUCLEAR ON FEBRUARY 28, 1985
ON EXPANDED SAFETY SYSTEM FACILITY

Re: Oyster Creek Nuclear Generating Station

On Thursday, February 28, 1985, a meeting was held at NRC, Bethesda, Maryland, with GPU Nuclear (licensee) to discuss the licensee's proposed Expanded Safety System Facility (ESSF). Enclosure 1 is the list of the individuals attending the meeting. Enclosure 2 is the agenda for the meeting.

The licensee requested the meeting to inform NRR of certain improvements that are being studied by the licensee that may be undertaken at the Oyster Creek Station site. The improvements would be housed in a new structure not now existing at the site that is referred to as the Expanded Safety System Facility (ESSF). Enclosure 3 is a draft letter and an enclosure describing the facility, the scope of the project, including the improvements, the design criteria, the licensing basis and the preliminary schedule.

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This concept of the ESSF has been considered by the licensee for a long time. It was stated that past owner initiated and NRC required changes to the Oyster Creek station have resulted in reducing the reserve capacity of the existing safety-grade diesels. The licensee has concluded that additional safety-grade diesel power, electrical distribution capability and additional safety-grade building space is needed at the site. The licensee states that the ESSF is to meet this need and meet the NRC requirements on control room habitability for a safety grade control room ventilation system.

The licensee stated that the existing emergency diesels are almost fully loaded. Further, the upgrading of the control room ventilation system to meet NRC control room habitability requirements is an essential load to be added to the new diesels. However, the need for additional safety-grade diesel generator capacity at the site is not being caused solely by this requirement.

The licensee presented drawings showing the location of the ESSF on site and the arrangement of equipment in the building. The building is free standing, resting on piles down to the undisturbed soil and to be designed to the NRC approved Systematic Evaluation Program (SEP) criteria. The building will house two new diesel generators on the first floor, building ventilation equipment on the second floor and the control room ventilation equipment on the third floor. The building is spacious by existing station standards.

The licensee stated that the ESSF is an improvement which will only help the station, and the new emergency diesels will not have Appendix K safety loads on them.

The licensee stated that the equipment to be installed in the building will be primarily existing equipment from cancelled nuclear plants. The licensee requested staff input regarding the criteria to be used in designing the building and in the use of surplus plant equipment for Oyster Creek because of the substantial resources which are involved in the ESSF (approximately \$50 million). The licensee stated that the basis intended to be used is the original station design basis and, where this has been superseded by SEP criteria, the NRC approved SEP criteria. The licensee expressed the desire to receive a staff response regarding this initiative including our concurrence regarding the use of SEP criteria. The staff response would be developed in response to a letter to the staff very similar to the draft letter in Enclosure 3.

The licensee stated its intention to periodically submit information on the ESSF project, to keep the staff up-to-date; thus hoping to negate any surprises from the staff. The licensee had a meeting with Region I on the ESSF but did not show the Region I staff the building drawings because they did not exist at the time of the meeting.

The licensee stated that commitments made to meet the NRC requirements on control room habitability are for the Cycle 12 outage when the ESSF would be completed. However, the licensee stated that the staff is not being asked to accept the ESSF as part of the resolution of control room habitability. The

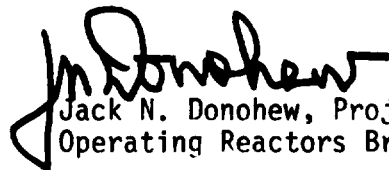
licensee stated that the proper people for discussing control room habitability were not present at this meeting and the licensee agreed to a meeting at the site for the staff to see the existing control room ventilation system and to have the proper people at this meeting to resolve the outstanding issues remaining in control room habitability. This meeting at the site will be held on March 19, 1985.

The present meeting was summarized by J. Zwolinski of the staff to inform D. Crutchfield of what had been discussed. The summary is as follows:

- The licensee is planning to build a new facility at the Oyster Creek site.
- This facility will house safety-grade diesel generators, electrical distribution systems and control room ventilation equipment.
- The ventilation equipment is to meet the NRC requirements on control room habitability.
- The issue of control room habitability is a separate issue from the new facility and will be the subject of a future visit by the staff to the site.
- The equipment for the ESSF will be purchased from cancelled nuclear plants.
- The licensee requests the staff to agree by letter to the original plant design and, where superseded by SEP criteria, the NRC approved SEP criteria as an acceptable design basis for the facility and the surplus equipment.
- This facility and equipment would be a definite improvement to the Oyster Creek Station and, if the formal letter from the licensee was in agreement with what was discussed at the meeting, the staff would respond within the requested time.

- ° The preliminary assessment was that the staff was in agreement with the design basis for the facility and equipment proposed by the licensee.

The matter of the licensee's preliminary judgement that the facility could be constructed under 10 CFR 50.59 was not discussed in any detail because this is between the licensee and Region I; however, the staff in conjunction with Region I can address this matter in its letter to the licensee.


 Jack N. Donohew, Project Manager
 Operating Reactors Branch No. 5, DL

Enclosures:

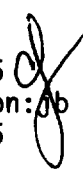
1. Attendance List
2. Meeting Agenda
3. Draft Letter

- cc: D. Crutchfield
 L. Hulman
 T. Quay
 K. Dempsey
 R. Scholl
 T. Cheng
 M. Laggart (GPU Nuclear)

DISTRIBUTION

Docket	NRC PDR
ORB Reading	Local PDR
CJamerson	OELD
JDonohew	EJordan
JZwolinski	BGrimes
ACRS (10)	NRC Participants

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 CJamerson:jb
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DL:ORB#5
 JDonohew
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 DL:ORB#5
 JZwolinski
 3/11/85

March 11, 1985

cc

G. F. Trowbridge, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N.W.
Washington, D.C. 20036

J.B. Liberman, Esquire
Bishop, Liberman, Cook, et al.
1155 Avenue of the Americas
New York, New York 10036

Dr. Thomas E. Murley
Regional Administrator
Nuclear Regulatory Commission
Region I Office
631 Park Avenue
King of Prussia, Pennsylvania 19406

FWR Licensing Manager
GPU Nuclear
100 Interpace Parkway
Parsippany, New Jersey 07054

Deputy Attorney General
State of New Jersey
Department of Law and Public Safety
36 West State Street - CN 112
Trenton, New Jersey 08625

Mayor
Lacey Township
818 West Lacey Road
Forked River, New Jersey 08731

U.S. Environmental Protection Agency
Region II Office
ATTN: Regional Radiation Representative
26 Federal Plaza
New York, New York 10007

D. G. Holland
Licensing Manager
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

Resident Inspector
c/o U.S. NRC
Post Office Box 445
Forked River, New Jersey 08731

Commissioner
New Jersey Department of Energy
101 Commerce Street
Newark, New Jersey 07102

Eugene Fisher, Assistant Director
Division of Environmental Quality
Department of Environmental
Protection
380 Scotch Road
Trenton, New Jersey 08628

P. B. Fiedler
Vice President & Director
Oyster Creek Nuclear Generating
Station
Post Office Box 388
Forked River, New Jersey 08731

ENCLOSURE 1

MEETING WITH GPU NUCLEAR ON EXPANDED SAFETY SYSTEM FACILITY

THURSDAY, FEBRUARY 28, 1985

<u>Name</u>	<u>Company</u>
J. Donohew	NRC/DL/ORB#5
J. Thorpe	GPUN
P. F. Wells	GPUN
B. Churchill	Shaw Pittman (GPUN)
R. F. Wilson	GPUN
R. W. Keaten	GPUN
R. Scholl	NRC/DL/ADSA
J. A. Zwolinski	NRR/DL/ADSA/ORB#5
L. G. Hulman	NRR/DSI/AEB
P. C. Carr	Lic. Bechtel Power
T. R. Quay	NRR/DSI/AEB
K. C. Dempsey	NRR/DSI/AEB
T. Cheng	NRC/DL/SEP8
L. Reiter	NRC/DE/GSB
D. Crutchfield	NRC/DL/ADSA

AGENDA FOR MEETING ON THE
OYSTER CREEK EXPANDED SAFETY SYSTEM FACILITY

- I. Approved SEP Design Criteria for Oyster Creek
- II. Facility Design Criteria
- III. Basis for Licensee's Action
 - ° 50.59 Determination
 - ° Facility Design Criteria
- IV. Construction Schedule
- V. Staff Comments
- VI. Meeting with D. Crutchfield
- VII. Discussion of Control Room Habitability