

NOV 08 1987

Docket No. 50-219

Mr. P. B. Fiedler
Vice President and Director
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

Dear Mr. Fiedler:

Subject: ISSUANCE OF AMENDMENT (TAC 65640)

The Commission has issued the enclosed Amendment No. 118 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment is a partial response to your application dated June 19, 1987, as supplemented by your letter dated July 14, 1987. Matters related to your request regarding organizational changes to corporate and Oyster Creek site organization of GPUN Nuclear Corporation were addressed in Amendment No. 117.

This amendment involves a change in a surveillance requirement. This amendment changes Technical Specification 4.3-G to include the requirement for leak testing after each Primary Coolant System Pressure Isolation Valve movement.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

/s/

Alexander W. Dromerick, Project Manager
Project Directorate I-4
Division of Reactor Projects I/II

Enclosures:

- 1. Amendment No. 118 to DPR-16
- 2. Safety Evaluation

cc w/enclosures:

See next page

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Mr. P. B. Fiedler
Oyster Creek Nuclear Generating Station

Oyster Creek Nuclear
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 118
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear Corporation, et al. (the licensee) dated June 19, 1987, as supplemented July 14, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(?) of Facility Operating License No. DPR-16 is hereby amended to read as follows:

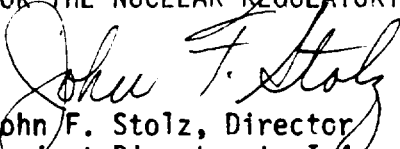
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 118, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This amendment is effective on issuance, to be implemented no later than 30 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 6, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 118
PROVISIONAL OPERATING LICENSE NO. DPR-16
DOCKET NO. 50-219

Replace the following page of Appendix A Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the areas of change.

Remove

Page 4.3-2

Insert

Page 4.3-2

*G. Primary Coolant System Pressure Isolation Valves Specification:

1. Periodic leakage testing (a) on each valve listed in table 4.3.1 shall be accomplished prior to exceeding 600 psig reactor pressure every time the plant is placed in the cold shutdown condition for refueling, each time the plant is placed in a cold shutdown condition for 72 hours if testing has not been accomplished in the preceding 9 months, whenever the valve is moved whether by manual actuation or due to flow conditions, and after returning the valve to service after maintenance, repair or replacement work is performed.

H. Reactor Coolant System Leakage

1. Unidentified leakage rate shall be calculated at least once every 4 hours.
2. Total leakage rate (identified and unidentified) shall be calculated at least once every 8 hours.
3. A channel calibration of the primary containment sump flow integrator and the primary containment equipment drain tank flow integrator shall be conducted at least once per 18 months.

Bases:

Numerous data are available relating integrated flux and the change in Nil-Ductility Transition Temperature (NDTT) in various steels. The base metal has been demonstrated to be relatively insensitive to neutron irradiation (see expected NDT changes in FDSAR Table IV-1-1, and Figures IV-2-0 and IV-2-10). The most conservative data has been used in Specification 3.3. The integrated flux at the vessel wall is calculated from core physics data and will be measured using flux monitors installed inside the vessel. The measurements of the neutron flux at the vessel wall will be used to check and if necessary correct, the calculated data to determine an accurate flux. From this a conservative NDT temperature can be determined. Since no shift will occur until an integrated flux of 10^{17} nvt is reached, the confirmation can be made long before an NDTT shift would occur.

The inspection program will reveal problem areas should they occur, before a leak develops. In addition, extensive visual inspection for leaks will be made on critical systems. Oyster Creek was designed and constructed prior to

(a) To satisfy ALARA requirements, leakage may be measured indirectly (as from the performance of pressure indicators) if accomplished in accordance with approved procedures and supported by computations showing that the method is capable of demonstrating valve compliance with the leakage criteria.

*NRC Order dated April 20, 1981



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 118

TO FACILITY OPERATING LICENSE NO. DPR-16

JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

Introduction

The April 20, 1981, Order for Modification of License Concerning Primary Coolant System Pressure Isolation Valves required the licensee to implement Technical Specifications (TS) to perform periodic leak testing of the Primary Coolant System Pressure Isolation Valves. The TS requirements for test frequency imposed by the April 20, 1981, Order were derived from the Franklin Research Center report TER-C5257-252 (Rev. 1). The test frequency criteria in the report, however, included a recommendation for leak testing, "...each time any check valve may have moved from the fully closed position..." which was not incorporated into the TS requirements imposed by the April 20, 1981, Order. This inconsistency was identified during NRC inspection 50-219/85-36 when the Core Spray System air operated check valves (classified as Primary Coolant System Isolation Valves in the TS) had been exercised twice without subsequent leakage testing. The licensee's TS 4.3-G did not include the requirement for leak testing after each Primary Coolant System Pressure Isolation Valve movement.

In response to the above NRC inspection concern, by letter dated June 19, 1987, GPU Nuclear Corporation submitted Technical Specification Change Request No. 146 to Provisional Operating License No. DPR-16 to amend the facility's TS to require leak rate testing after each primary coolant system pressure isolation valve movement. This SE addresses the additional proposed testing.

Proposed Changes

The proposed change is to add "...whenever a valve is moved whether by manual actuation or due to flow conditions..." to Technical Specification 4.3-G, to leak test the primary coolant system pressure isolation valves.

Review Criteria/Requirements

- (1) Order for Modification of License Concerning Primary Coolant System Pressure Isolation Valves (w/attachments), April 20, 1981.

Evaluation

The additional requirement for leakage testing of the Primary Coolant System Pressure Isolation Valves after each movement of the valves will provide confidence that the valves are properly seated. The change will also make the Oyster Creek Nuclear Generating Station Technical Specifications consistent with the test frequency criteria in TER-C5257-252 (Rev. 1) which formed the basis of the April 20, 1981, Order imposing the original leak test requirements. Based upon the above, the staff finds the licensee's proposed amendment to be acceptable.

Environmental Consideration

This amendment involves a change in a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: November 6, 1987

Principal Contributor: John G. Hunter, III, Division of Reactor Safety, RGI