

August 10, 1988

Docket No. 50-219

Mr. E. E. Fitzpatrick
Vice President and Director
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

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Dear Mr. Fitzpatrick:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 68372)

The Commission has issued the enclosed Amendment No. 125 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station, in response to your application dated May 26, 1988.

The amendment changes the Technical Specification by revising various areas in Section 6 (Administrative Controls) for clarity and consistency with GE Standard Technical Specification (NUREG-0123, Revision 3) insofar as the safety review process for new and revised procedures, modifications to unit structures, systems and components and for proposed tests and experiments is concerned. The amendment also adds a definition to Section 1 of the Technical Specifications for substantive changes to these activities.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

original signed by

Alexander W. Dromerick, Project Manager
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 125 to DPR-16
2. Safety Evaluation

cc w/enclosures:

See next page

LA:PDI-4

PM:PDI-4

PD:PDI-4

OGC

*SNorris

*ADromerick:lm

*JStolz

*APH - concur w/changes

07/25/88

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07/27/88

*Previously concurred

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Mr. E. E. Fitzpatrick
Oyster Creek Nuclear Generating Station

Oyster Creek Nuclear
Generating Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 125
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear Corporation, et al., (the licensee), dated May 26, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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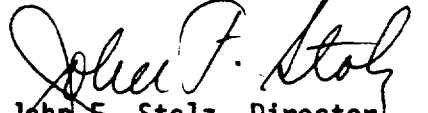
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 125, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 10, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 125

PROVISIONAL OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

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i
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Page 6-15

Insert

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1.39 REACTOR VESSEL PRESSURE TESTING

System pressure testing required by ASME Code Section XI, Article IWA-5000, including system leakage and hydrostatic tests, with reactor vessel completely water solid, core not critical and section 3.2.A satisfied.

1.40 SUBSTANTIVE CHANGES

SUBSTANTIVE CHANGES are those which affect the activities associated with a document or the document's meaning or intent. Examples of non-substantive changes are: (1) correcting spelling, (2) adding (but not deleting) sign-off spaces, (3) blocking in notes, cautions, etc, (4) changes in corporate and personnel titles which do not reassign responsibilities and which are not referenced in the Appendix A Technical Specifications, and (5) changes in nomenclature or editorial changes which clearly do not change function, meaning or intent.

Shift Technical Advisor

Requirements: Bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.3.2 Each member of the radiation protection organization for which there is a comparable position described in ANSI N18.1-1971 shall meet or exceed the minimum qualifications specified therein, or in the case of radiation protection technicians, they shall have at least one year's continuous experience in applied radiation protection work in a nuclear facility dealing with radiological problems similar to those encountered in nuclear power stations and shall have been certified by the Radiological Controls Director, as qualified to perform assigned functions. This certification must be based on an NRC approved, documented program consisting of classroom training with appropriate examinations and documented positive findings by responsible supervision that the individual has demonstrated his ability to perform each specified procedure and assigned function with an understanding of its basis and purpose.

6.4 TRAINING

6.4.1 A retraining program for operators shall be maintained under the direction of the Manager Plant Training Oyster Creek and shall meet the requirements and recommendation of Appendix A of 10CFR Part 55. Replacement training programs, the content of which shall meet the requirements of 10CFR Part 55, shall be conducted under the direction of the Manager Plant Training Oyster Creek for licensed operators and Senior Reactor Operators.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Manager Plant Training Oyster Creek.

6.5 REVIEW AND AUDIT

6.5.1 TECHNICAL REVIEW AND CONTROL

The Vice President of each division within GPU Nuclear Corporation as indicated in Figure 6.2.1, shall be responsible for ensuring the preparation, review, and approval of documents required by the activities described in 6.5.1.1 through 6.5.1.5 within his functional area of responsibility as assigned in the GPUN Review and Approval Matrix. Implementing approvals shall be performed at the cognizant manager level or above.

ACTIVITIES

6.5.1.1 Each procedure required by Technical Specification 6.8 and other procedures which affect nuclear safety, and substantive changes thereto, shall be prepared by a designated individual(s)/group knowledgeable in the area affected by the procedure. Each such procedure, and substantive change thereto, shall be reviewed for adequacy by an individual(s)/group other than the preparer, but who may be from the same division as the individual who prepared the procedure or change.

6.5.1.2 Proposed changes to the Appendix "A" Technical Specifications shall be reviewed by a knowledgeable individual(s)/group other than the individual(s)/group who prepared the change.

6.5.1.3 Proposed modifications, that affect nuclear safety, to facility structures, systems and components shall be designed by an individual/organization knowledgeable in the areas affected by the proposed modification. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification but may be from the same division as the individual who designed the modification.

6.5.1.4 Proposed tests and experiments that affect nuclear safety shall be reviewed by a knowledgeable individual(s)/group other than the preparer but who may be from the same division as the individual who prepared the tests and experiments.

6.5.1.5 Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence, shall be reviewed by a knowledgeable individual(s)/group other than the individual/group which performed the investigation.

6.5.1.6 Events requiring 24-hour written notification to the Commission shall be reviewed by an individual/group other than the individual/group which prepared the report.

6.5.1.7 Special reviews, investigations or analyses and reports thereon as requested by the Vice President & Director Oyster Creek shall be performed by a knowledgeable individual(s)/group.

6.5.1.8 The Security Plan and implementing procedures shall be reviewed by a knowledgeable individual(s)/group other than the individual(s)/group which prepared them.

6.5.1.9 The Emergency Plan and implementing procedures shall be reviewed by a knowledgeable individual(s)/group other than the individual(s)/group which prepared them.

6.5.1.10 Review of every unplanned onsite release of radioactive material to the environs including the preparation and forwarding of reports covering evaluation shall be performed by a knowledgeable individual(s)/group. Recommendations and disposition of the corrective action to prevent recurrence shall be sent to the Vice President & Director Oyster Creek.

6.5.1.11 Major changes to radwaste systems shall be reviewed by a knowledgeable individual(s)/group other than the individual(s)/group which prepared them.

6.5.1.12 Individuals responsible for reviews performed in accordance with 6.5.1.1 through 6.5.1.4 shall include a determination of whether or not additional cross-disciplinary review is necessary. If deemed necessary, such review shall be performed by the appropriate personnel. Individuals responsible for reviews considered under 6.5.1.1 through 6.5.1.5 shall render determinations in writing with regard to whether or not 6.5.1.1 through 6.5.1.5 constitute an unreviewed safety question.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented, and maintained that meet or exceed the requirements of the Nuclear Regulatory Commission's Regulatory Guide 1.33 (the applicable revision is identified in the GPU Nuclear Operational Quality Assurance Plan) and as provided in 6.8.2 and 6.8.3 below.

Written procedures shall be adopted and maintained to implement the:

Process Control Plan
Offsite Dose Calculation Manual

6.8.2 Each procedure and administrative policy of 6.8.1 above, and substantive changes thereto, shall be reviewed as described in 6.5.1.1 and approved as described in 6.5.1 prior to implementation and periodically as specified in the Administrative Procedures.

6.8.3 Temporary changes to procedures 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of GPUNC Management Staff authorized under Section 6.5.1.12 and knowledgeable in the area affected by the procedure. For changes which may affect the operational status of facility systems or equipment, at least one of these individuals shall be a member of facility management or supervision holding a Senior Reactor Operator's License on the facility.
- c. The change is documented, subsequently reviewed and approved as described in 6.8.2 within 14 days of implementation.

6.9 REPORTING REQUIREMENTS

In addition to the applicable reporting requirements of 10CFR, the following identified reports shall be submitted to the Director of the appropriate Regional Office of Inspection and Enforcement unless otherwise noted.

6.9.1 ROUTINE REPORTS

- a. Startup Report. A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specified details required in license conditions based on other commitments shall be included in this report.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO.125

TO PROVISIONAL OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND
JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated May 26, 1988, GPU Nuclear Corporation (GPUN) requested a revision to the Technical Specifications (TSs) for Oyster Creek Nuclear Generating Station. The TS change would revise various areas in Section 6 (Administrative Controls) for clarity and consistency with GE Standard Technical Specifications (NUREG-0123, Revision 3) insofar as the safety review process for new and revised procedures, modifications to unit structures, systems and components and for proposed test and experiments is concerned. The change would also add a definition to Section 1 of the TSs for substantive changes to these activities.

2.0 EVALUATION

Licensees are allowed by 10 CFR 50.59 to make changes in their facility (structures, systems and components) and procedures as described in the safety analysis report and to conduct tests and experiments not described in the safety analysis report without prior Commission approval as long as the change, test or experiment does not involve a TS change or an unreviewed safety question. This regulation also provides criteria for judging whether or not an unreviewed safety question might be involved in a proposed change, test or experiment. In order to assure appropriate provisions for compliance with 10 CFR 50.59, Section 6.5 of the TSs for most nuclear power plants lists specific requirements for technical review and audit of certain procedures and procedure changes (including those pertaining to tests and experiments) and of facility modifications. GPUN procedure 1000-ADM-1291.01 implements the safety review requirements of section 6.5 of the Oyster Creek TSs. On September 1, 1986, GPUN implemented a significant revision of this procedure which established a two-step process for review of procedure changes, facility modifications, tests and experiments. The first step of the process is to determine applicability of 10 CFR 50.59 to the contemplated action. Step two of the process is to provide a written safety evaluation of why the contemplated action does or does not involve a change to the TSs or an unreviewed safety question.

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The existence of the words "important to safety" in the current TSs for Oyster Creek have caused some confusion with respect to review of procedure changes and modifications. This confusion apparently stems in part from the industry wide issue with the NRC's use and definition of the terms "important to safety" and "safety related" as discussed in detail in NRC Generic Letter No. 84-01 dated January 5, 1984. To eliminate this confusion and to upgrade the Oyster Creek TSs terminology to be consistent with NUREG 0123, "Standard Technical Specifications for General Electric Boiling Water Reactors (BWR/s) Rev. 3," GPUN has proposed the term "affecting nuclear safety" as a substitute for "important to safety" in several places throughout Section 6.5.

Furthermore, GPUN has requested placing a definition for the word "substantive" in the TS (Section 1.40) and adding the word to Sections 6.5.1.1 and 6.8.2 to make it clear that minor changes not affecting the function, meaning, or intent of a document need not undergo the formal review process intended for substantive changes. GPUN's concern is that if the formal review process is literally applied to all changes, including corrections of typographical errors and editorial improvements, the number of such reviews will become overwhelming and the substantive changes may not get the proper level of detail in their review. Philosophically the staff agrees with this distinction as long as the individuals exercising these judgements are adequately trained and objective. The staff met with GPUN on April 15, 1988 to discuss, in detail, implementation of the safety review process and GPUN Procedure 1000-ADM-1291.01 at Oyster Creek and TMI-1. The staff concluded that with a minor modification to the procedure, the method used by GPUN will provide acceptable results.

The specific TS changes addressed by this Safety Evaluation are as follows:

1.40 Provides addition of a definition of substantive changes to documents.

6.5.1.1 Replaces "important to safety" with "which affect nuclear safety" as applied to which procedures require preparation and review by a designated individual or group. Replaces "important to safety" with "substantive" in specifying to which procedures changes this section applies.

6.5.1.3 Replaces "important to safety" with "that affect nuclear safety" in conjunction with modifications to unit structures, systems and components and clarifies that those words apply to the proposed modification and not to the structures, system or component themselves.

6.5.1.4 Replaces "important to safety" with "that affect nuclear safety" in conjunction with tests and experiments.

6.8.2 Replaces "important to safety" with "substantive" in conjunction with review of procedures required by Section 6.8.1.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment changes administrative procedures and requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Dated: August 10, 1988

Principal Contributor:

Alexander W. Dromerick