

November 18, 1988

Docket No. 50-219

Mr. E. E. Fitzpatrick
Vice President and Director
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

Dear Mr. Fitzpatrick:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 69508)

The Commission has issued the enclosed Amendment No. 130 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station, in response to your application dated September 28, 1988.

The amendment allows removing the high pressure scram function when the reactor head is removed or unbolted. This change is necessary to install new analog pressure sensors during the 12R refueling outage.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

original signed by John Stolz for

Alexander W. Dromerick, Project Manager
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 130 to DPR-16
- 2. Safety Evaluation

cc w/enclosures:

See next page

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[Signature]

Mr. E. E. Fitzpatrick
Oyster Creek Nuclear Generating Station

Oyster Creek Nuclear
Generating Station

cc:

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Forked River, New Jersey 08731



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No.130
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear Corporation, et al., (the licensee), dated September 28, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 130, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 18, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 130
PROVISIONAL OPERATING LICENSE NO. DPR-16
DOCKET NO. 50-219

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

Page 3.1-8

Page 3.1-19

Insert

Page 3.1-8

Page 3.1-19

TABLE 3.1.1 PROTECTIVE INSTRUMENTATION REQUIREMENTS

Function	Trip Setting	Reactor Modes in which Function Must Be Operable				Min. No. of Operable or Operating [tripped] Trip Systems	Min. No. of Instrument Channels Per Operable Trip Systems	Action Required*
		Shutdown	Refuel	Startup	Run			
A. Scram								
1. Manual Scram		X	X	X	X	2	1	Insert control rods
2. High Reactor Pressure	**		X(s)	X(11)	X	2	2	
3. High Drywell Pressure	3.5 psig		X(u)	X(u)	X	2	2	
4. Low Reactor Water Level	**		X	X	X	2	2	
5. a. High Water Level in Scram Discharge Volume North Side	29 gal.		X(a)	X(z)	X(z)	2	2	
b. High Water Level in Scram Discharge Volume South Side	29 gal.		X(a)	X(z)	X(z)	2	2	
6. Low Condenser Vacuum	23" hg.		X(b)	X(b)	X	2	2	
7. High Radiation in Main Steam Line Tunnel	10 x normal background		X(s)	X	X	2	2	

TABLE 3.1.1 (CONTD)

- *gg. These functions are not required to be operable when secondary containment is not required to be maintained or when the conditions of Section 3.5.b.1.a, b, c, and d are met, and reactor water level is closely monitored and logged hourly. The Standby Gas Treatment System will be manually initiated if reactor water level drops to the low level trip set point.
- hh. The high flow trip function for "B" Isolation Condenser is bypassed upon initiation of the alternate shutdown panel. This prevents a spurious trip of the isolation condenser in the event of fire induced circuit damage.
- ii. Instrument shall be operable during main condenser air ejector operation except that a channel may be taken out-of-service for the purpose of a check, calibration, test, or maintenance without declaring it to be inoperable.
- jj. With no channel OPERABLE, main condenser offgas may be released to the environment for as long as 72 hours provided the stack radioactive noble gas monitor is OPERABLE. Otherwise, be in at least SHUTDOWN CONDITION within 24 hours.
- kk. Reserved.
- ll. This function not required to be operable with the reactor vessel head removed or unbolted.

* This note is applicable only during the Cycle 10M outage.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 130

TO PROVISIONAL OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND
JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

INTRODUCTION

By letter dated September 28, 1988, GPU Nuclear Corporation (GPUN/licensee) requested an amendment to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station (Oyster Creek). The requested change to Appendix A of the operating license would allow removing the high pressure scram function when the reactor vessel head is removed or unbolted. This change is necessary to install new analog pressure sensors during the 12R refueling outage. The amendment would revise the Oyster Creek Technical Specifications Table 3.1.1 by adding a reference note "ll" in the startup mode for the high reactor pressure scram function, which would state: "This function not required to be operable with the reactor vessel head removed or unbolted." Table 3.1.1 note "kk" is being reserved for another Technical Specification change presently under the licensee's review.

EVALUATION

The licensee's proposed Technical Specification Change Request (TSCR) No. 174 would allow removing high pressure scram function when the reactor vessel head is removed or unbolted. This change is necessary to install new analog pressure sensors during the 12R refueling outage.

When the reactor temperature is less than 212°F and either the reactor vessel head is removed or unbolted, there is no possibility of a major reactor pressure excursion. Since a major reactor pressure excursion is not possible, the instrumentation required to scram the reactor in response to an excursion serves no purpose. Additionally, the Boiling Water Reactor Standard Technical Specifications specifically state that the high pressure scram function is not required when the pressure vessel head is unbolted or removed in the STARTUP mode. This request more closely aligns the Oyster Creek Nuclear Generating Station Technical Specifications with the Boiling Water Reactor Standard Technical Specifications. On this basis, we have determined that above changes to Table 3.1.1 of the Oyster Creek Technical Specifications are acceptable.

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ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Dated: November 18, 1988

Principal Contributor: A. W. Dromerick