

May 23, 1989

Docket No. 50-219

Mr. E. E. Fitzpatrick  
Vice President and Director  
Oyster Creek Nuclear Generating Station  
Post Office Box 388  
Forked River, New Jersey 08731

DISTRIBUTION

Docket File EJordan  
NRC & Local PDRs BGrimes  
Plant File TMeek(4) WJones  
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ADromerick JKudrick  
OGC  
DHagan

Dear Mr. Fitzpatrick:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 65100)

The Commission has issued the enclosed Amendment No.133 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station, in response to your application dated March 17, 1987.

The amendment revises Section 3.3, Reactor Coolant, of the Appendix A Technical Specifications regarding the requirements of Generic Letter 84-11. Specifically, the amendment limits the reactor coolant leakage system to a 2 gpm increase in unidentified leakage rate within any 24 hour period while operating at steady state power. In your letter of February 4, 1987 you justified that no further technical specifications were needed in this area.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

original signed by

Alexander W. Dromerick, Project Manager  
Project Directorate I-4  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 133 to DPR-16
- 2. Safety Evaluation

cc w/enclosures:  
See next page

[TAC NO. 65100]

LA:PDI-4  
SNorris  
5/15/89

PM:PDI-4  
ADromerick:cb  
5/15/89

PS:PDI-4  
JStolz  
5/15/89

OGC  
5/17/89

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Mr. E. E. Fitzpatrick  
Oyster Creek Nuclear Generating Station

Oyster Creek Nuclear  
Generating Station

cc:

Ernest L. Blake, Jr.  
Shaw, Pittman, Potts and Trowbridge  
2300 N Street, NW  
Washington, D.C. 20037

Resident Inspector  
c/o U.S. NRC  
Post Office Box 445  
Forked River, New Jersey 08731

J.B. Liberman, Esquire  
Bishop, Liberman, Cook, et al.  
1155 Avenue of the Americas  
New York, New York 10036

Commissioner  
New Jersey Department of Energy  
101 Commerce Street  
Newark, New Jersey 07102

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, Pennsylvania 19406

Jennifer Moon, Acting Chief  
New Jersey Department of Environmental  
Protection  
Bureau of Nuclear Engineering  
CN 415  
Trenton, New Jersey 08625

BWR Licensing Manager  
GPU Nuclear Corporation  
1 Upper Pond Road  
Parsippany, New Jersey 07054

Deputy Attorney General  
State of New Jersey  
Department of Law and Public Safety  
36 West State Street - CN 112  
Trenton, New Jersey 08625

Mayor  
Lacey Township  
818 West Lacey Road  
Forked River, New Jersey 08731

Licensing Manager  
Oyster Creek Nuclear Generating Station  
Mail Stop: Site Emergency Bldg.  
P. O. Box 388  
Forked River, New Jersey 08731



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 133  
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by GPU Nuclear Corporation, et al., (the licensee), dated March 17, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 133, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director  
Project Directorate I  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: May 23, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 133  
PROVISIONAL OPERATING LICENSE NO. DPR-16  
DOCKET NO. 50-219

Replace the following page of the Appendix A Technical Specifications with the enclosed page as indicated. The revised page is identified by amendment number and contains vertical lines indicating the areas of change.

Remove

Page 3.3.2

Insert

Page 3.3.2

D. Reactor Coolant System Leakage

1. Reactor coolant system leakage shall be limited to:
  - a. 5 gpm unidentified leakage
  - b. 25 gpm total (identified and unidentified)
  - c. 2 gpm increase in unidentified leakage rate within any 24 hour period while operating at steady state power
2. With the reactor coolant system leakage greater than the limits in 3.3.D.1.a or b above, reduce the leakage rate to within the acceptable limits within 8 hours, or place the reactor in the shutdown condition within the next 12 hours and be in the cold shutdown condition within the following 24 hours.
3. With any reactor coolant leakage greater than the limit in 3.3.D.1.c above, identify the source of leakage within 4 hours, or be in the shutdown condition within the next 12 hours and be in the cold shutdown condition within the following 24 hours.
4. For determination of unidentified leakage, the primary containment sump flow monitoring system shall be operable except as specified below:
  - a. With the primary containment sump flow integrator inoperable:
    1. Restore it to operable status within 7 days.
    2. Calculate the unidentified leakage rate utilizing an acceptable alternate means as specified in plant procedures.
  - b. If Specification 3.3.D.4a cannot be met, place the reactor in the shutdown condition within the next 12 hours.
5. For determination of identified leakage, the primary containment equipment drain tank monitoring system shall be operable except as specified below:
  - a. With the primary containment equipment drain tank monitoring system inoperable:
    1. Restore it to operable status within 7 days.
    2. Calculate the identified leakage rate utilizing an acceptable alternate means as specified in plant procedures.
  - b. If Specification 3.3.D.5.a cannot be met, place the reactor in the shutdown condition within the next 12 hours.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 133

TO PROVISIONAL OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND  
JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

1.0 INTRODUCTION

In a letter dated September 5, 1986, the NRC requested GPU Nuclear Corporation (GPUN/licensee) to revise the Oyster Creek Technical Specifications (TS) concerning the reactor coolant system leakage. The request was made to comply with items B, C, and E of Attachment 1 to Generic Letter (GL) 84-11. In response to this request, the licensee submitted letters dated February 4, 1987 and March 17, 1987. Briefly, GPUN indicated that item B would be met. The leakage would be monitored to assure that the maximum increase in unidentified leakage would not exceed 2 gpm within any 24 hour period. This would be incorporated into the TS. However, the remaining two issues were believed by GPUN to be adequately addressed by the existing TS. Therefore, no further TS changes were recommended by the licensee. Rather, justification was provided to show why additional TS changes were not necessary.

Item C of GL 84-11 suggested that the maximum outage time for inoperable leakage measurement instruments associated with each sump be limited by TS to 24 hours. GPUN provided operating procedures put into effect when the instrument would be inoperable that they believed would justify inoperability times of 7 days rather than the 24 hours. Seven days is the current TS LCO.

Item E of GL 84-11 called for visual examinations for leakage of the reactor coolant piping be performed during each outage. The licensee showed that the existing TS 4.3d currently calls for a visual examination for leaks with the reactor coolant system at pressure during each scheduled refueling outage or after major repairs have been made to the reactor coolant system. The difference between GL 84-11 and the current TS being the interpretation of "outage". GL 84-11, if taken literally, would indicate that any time the drywell is deinerted during a shutdown, an examination would be required. The existing TS and the GPUN position is that the intent of GL 84-11 is met if "outage" is interpreted to mean "refueling outage". Therefore, no revisions to the TS are necessary.

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## 2.0 EVALUATION

The licensee's proposed TS to limit the leakage increase within a 24 hour period to 2 gpm meets the literal meaning of item B of Attachment 1 to GL 84-11. Therefore, the staff finds the proposed TS change acceptable.

The intent of item C of GL 84-11 is to limit the outage time of critical leakage instrumentation to 24 hours. The GPUN submittal showed that whenever a critical instrument is inoperable, alternate procedures would be activated. These procedures would require leak rate measurements to be performed every 4 hours. Calculations would then be performed to determine the leak rate which would be recorded every 4 hours. This procedure would be followed for up to 7 days. If the instrument was not repaired within 7 days, the reactor would be taken to a cold shutdown in an orderly manner.

The staff having reviewed the above procedures, has concluded that the alternate procedures adequately compensate for the increased time for continued power operation with inoperable instrumentation. The conclusion is based on the belief that equivalent measurements will be obtained at 4 hour intervals. In other words, the measurement interval is changed from near continuous to every 4 hours. This appears to be an acceptable modification to allow the inoperable time to increase from the recommended 24 hours to 7 days.

The final GL 84-11 item addressed by the licensee submittals is the frequency that visual examinations for leaks shall be made with the reactor coolant system at pressure. GPUN has interpreted the required frequency to mean each refueling outage. If taken literally, the staff's requirement would mean that each shutdown resulting in a deinerted drywell would require a visual examination.

The staff believes that the licensee's interpretation meets the intent of the generic letter. There are instances when the drywell would be deinerted for only a short period of time; such as during a quick repair. The letter was not intended to require a unique inspection for these types of maintenance shutdowns. Rather, the intent was to institute a visual inspection program at regular intervals so that slowly growing cracks would be detected. Therefore, the staff finds the licensee's interpretation acceptable and additional TS changes unnecessary.

Based on the rationale discussed above, the staff finds the licensee's proposed TS changes acceptable. Also, the justification provided by the licensee to preclude further TS changes is acceptable.

## 3.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types,

of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The staff has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Dated: May 23, 1989

Principal Contributor: J. Kudrick