

Mr. E. E. Fitzpatrick
Vice President and Director
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

Dear Mr. Fitzpatrick:

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION - ISSUANCE OF AMENDMENT
(TAC NO. 67819)

The Commission has issued the enclosed Amendment No. 135 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station, in response to your application dated March 31, 1988, supplemented November 15, 1988 and August 23, 1989.

The amendment revises the Technical Specifications to delete Technical Specification Safety Limit 2.1.E which required at least two recirculation loops to be fully open except when the reactor vessel head is off and vessel water level is above the main steam nozzle. This amendment also incorporates this limitation in Technical Specification 3.3.F and that limitation is revised to require that at least one recirculation loop, instead of two be fully open during applicable plant conditions. The proposed change would also require that during power operations if at least four recirculation loops cannot be maintained inservice, then hot shutdown or refuel conditions must be reached in 12 hours. Presently, the requirement is that the unit be placed in cold shutdown conditions within 24 hours.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance has been forwarded to the Office of the Federal Register for publication.

Sincerely,

/s/

Alexander W. Dromerick, Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 135 to DPR-16
- 2. Safety Evaluation
- 3. Notice

cc w/enclosures:
See next page

*See previous concurrence

OFC	:LA:PDI-4*	:PM:PDI-4*	:PD:PDI-4*	:OGC*	:	:	:
NAME	:SNorris	:ADromrick	:lm:JStolz	:S.Turk - 12/13/89	:	:	:
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DATED: December 30, 1989

AMENDMENT NO. 135 TO FACILITY OPERATING LICENSE NO. DPR-16

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George Thomas (8E23)

ACRS (10)

GPA/PA

ARM/LFMB

cc: Licensee/Applicant Service List



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

December 30, 1989

Docket No. 50-219

Mr. E. E. Fitzpatrick
Vice President and Director
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

Dear Mr. Fitzpatrick:

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION - ISSUANCE OF AMENDMENT
(TAC NO. 67819)

The Commission has issued the enclosed Amendment No. 13⁵ to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station, in response to your application dated March 31, 1988, supplemented November 15, 1988 and August 23, 1989.

The amendment revises the Technical Specifications to delete Technical Specification Safety Limit 2.1.E which required at least two recirculation loops to be fully open except when the reactor vessel head is off and vessel water level is above the main steam nozzle. This amendment also incorporates this limitation in Technical Specification 3.3.F and that limitation is revised to require that at least one recirculation loop, instead of two be fully open during applicable plant conditions. The proposed change would also require that during power operations if at least four recirculation loops cannot be maintained inservice, then hot shutdown or refuel conditions must be reached in 12 hours. Presently, the requirement is that the unit be placed in cold shutdown conditions within 24 hours.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance has been forwarded to the Office of the Federal Register for publication.

Sincerely,

A handwritten signature in cursive script that reads "Alexander W. Dromerick".

Alexander W. Dromerick, Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No135 to DPR-16
2. Safety Evaluation
3. Notice

cc w/enclosures:
See next page

Mr. E. E. Fitzpatrick
Oyster Creek Nuclear Generating Station

Oyster Creek Nuclear
Generating Station

cc:

Ernest L. Blake, Jr.
Shaw, Pittman, Potts and Trowbridge
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Washington, D.C. 20037

Resident Inspector
c/o U.S. NRC
Post Office Box 445
Forked River, New Jersey 08731

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Bishop, Liberman, Cook, et al.
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Licensing Manager
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Mail Stop: Site Emergency Bldg.
P. O. Box 388
Forked River, New Jersey 08731

Mr. E. E. Fitzpatrick
Vice President and Director
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 135
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear Corporation, et al., (the licensee), dated March 31, 1988 as as supplemented in letters dated November 15, 1988 and August 23, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

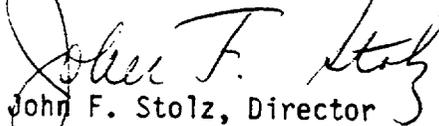
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.135, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 30, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 135
PROVISIONAL OPERATING LICENSE NO. DPR-16
DOCKET NO. 50-219

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
2.1-1	2.1-1
2.1-3	2.1-3
3.3-3	3.3-3
-	3.3-3a
3.3-8	3.3-8
-	3.38a

SECTION 2

SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

2.1 SAFETY LIMIT - FUEL CLADDING INTEGRITY

Applicability: Applies to the interrelated variables associated with fuel thermal behavior.

Objective: To establish limits on the important thermal hydraulic variables to assure the integrity of the fuel cladding.

Specifications:

- A. When the reactor pressure is greater than or equal to 800 psia and the core flow is greater than or equal to 10% of rated, the existence of a minimum critical power ratio (MCPR) less than 1.07 shall constitute violation of the fuel cladding integrity safety limit.
- B. When the reactor pressure is less than 800 psia or the core flow is less than 10% of rated, the core thermal power shall not exceed 25% of rated thermal power.
- C. In the event that reactor parameters exceed the limiting safety system settings in specification 2.3 and a reactor scram is not initiated by the associated protective instrumentation, the reactor shall be brought to, and remain in, the cold shutdown condition until an analysis is performed to determine whether the safety limit established in specification 2.1.A and 2.1.B was exceeded.
- D. During all modes of reactor operation with irradiated fuel in the reactor vessel, the water level shall not be less than 4'8" above the top of active fuel.

Bases:

The fuel cladding integrity safety limit is set such that no fuel damage is calculated to occur if the limit is not violated. Since the parameters which result in fuel damage are not directly observable during reactor operation the thermal and hydraulic conditions resulting in a departure from nucleate boiling have been used to mark the beginning of the region where fuel damage could occur. Although it is recognized that a departure from nucleate boiling would not necessarily result in damage to BWR fuel rods, the critical power at which boiling transition is calculated to occur has been adopted as a convenient limit. However, the uncertainties in monitoring the core operating state and in the procedure used to calculate the

cooling capability could lead to elevated cladding temperatures and clad perforation. With a water level above the top of the active fuel, adequate cooling is maintained and the decay heat can easily be accommodated. It should be noted that during power generation there is no clearly defined water level inside the shroud and what actually exists is a mixture level. This mixture begins within the active fuel region and extends up through the moisture separators. For the purpose of this specification water level is defined to include mixture level during power operations.

The lowest point at which the water level can presently be monitored is 4'8" above the top of active fuel. Although the lowest reactor water level limit which ensures adequate core cooling is the top of the active fuel, the safety limit has been conservatively established at 4'8" above the top of active fuel.

REFERENCES

- (1) NEDO-24195, General Electric Reload Fuel Application for Oyster Creek.

E. Reactor Coolant Quality

1. The reactor coolant quality during power operation with steaming rates to the turbine-condenser of less than 100,000 pounds per hour shall be limited to:

conductivity 2 uS/cm (S = mhos at 25°C(77°F))
chloride ion 0.1 ppm

2. When the conductivity and chloride concentration limits given in 3.3.E.1 are exceeded, an orderly shutdown shall be initiated immediately, and the reactor coolant temperature shall be reduced to less than 212°F within 24 hours.
3. The reactor coolant quality during power operation with steaming rates to the turbine-condenser of greater than or equal to 100,000 pounds per hour shall be limited to:

conductivity 10 uS/cm (S = mhos at 25°C(77°F))
chloride ion 0.5 ppm

4. When the maximum conductivity or chloride concentration limits given in 3.3.E.3 are exceeded, an orderly shutdown shall be initiated immediately, and the reactor coolant temperature shall be reduced to less than 212°F within 24 hours.
5. During power operation with steaming rates on the turbine-condenser of greater than or equal to 100,000 pounds per hour, the time limit above 1.0 uS/cm at 25°C (77°F) and 0.2 ppm chloride shall not exceed 72 hours for any single incident.
6. When the time limits for 3.3.E.5 are exceeded, an orderly shutdown shall be initiated within 4 hours.

F. Recirculation Loop Operability

1. During POWER OPERATION, all five recirculation loops shall be OPERATING except as specified in Specification 3.3.F.2.
2. POWER OPERATION with one idle recirculation loop is permitted provided that the idle loop is not isolated from the reactor vessel.
3. If Specifications 3.3.F.1 and 3.3.F.2 are not met, an orderly shutdown shall be initiated immediately until all operable control rods are fully inserted and the reactor is in either the REFUEL MODE or SHUTDOWN CONDITION within 12 hours.
4. With reactor coolant temperature greater than 212°F and irradiated fuel in the reactor vessel, at least one recirculation loop discharge valve and its associated suction valve shall be in the full open position.
5. If Specification 3.3.F.4 is not met, immediately open one recirculation loop discharge valve and its associated suction valve.

6. With reactor coolant temperature less than 212°F and irradiated fuel in the reactor vessel, at least one recirculation loop discharge valve and its associated suction valve shall be in the full open position unless the reactor vessel is flooded to a level above 185 inches TAF or unless the steam separator and dryer are removed.

pH, chloride, and other chemical parameters are made to determine the cause of the unusual conductivity and instigate proper corrective action. These can be done before limiting conditions, with respect to variables affecting the boundaries of the reactor coolant, are exceeded. Several techniques are available to correct off-standard reactor water quality conditions including removal of impurities from reactor water by the cleanup system, reducing input of impurities causing off-standard conditions by reducing power and reducing the reactor coolant temperature to less than 212°F. The major benefit of reducing the reactor coolant temperature to less than 212°F is to reduce the temperature dependent corrosion rates and thereby provide time for the cleanup system to re-establish proper water quality.

Specifications 3.3.F.1 and 3.3.F.2 require a minimum of four OPERATING recirculation loops during reactor POWER OPERATION. Core parameters have not been established for POWER OPERATION with less than four OPERATING loops. Therefore, Specification 3.3.F.3 requires reactor POWER OPERATION to be terminated and the reactor placed in the REFUEL MODE or SHUTDOWN CONDITION within 12 hours. During four loop POWER OPERATION the idle loop is required to have its discharge valve closed and its discharge bypass and suction valves open. This minimizes the occurrence of a severe cold water addition transient during startup of an idle loop. In addition, with the discharge bypass and suction valves in an idle loop open the coolant inventory in the loop is available during LOCA blowdown.

Specifications 3.3.F.4 and 3.3.F.6 assure that an adequate flow path exists from the annular space, between the pressure vessel wall and the core shroud, to the core region. This provides sufficient hydraulic communication between these areas, thus assuring that reactor water level instrument readings are indicative of the water level in the core region. For the bounding loss of feedwater transient⁽²⁾, a single fully open recirculation loop transfers coolant from the annulus to the core region at approximately five times the boiloff rate with no forced circulation⁽³⁾. With the reactor vessel flooded to a level above 185 inches TAF or when the steam separator and dryer are removed, the core region is in hydraulic communication with the annulus above the core region and all recirculation loops can therefore be isolated. When the steam separator and dryer are removed, safety limit 2.1.D ensures water level is maintained above the core shroud.

References

- (1) FDSAR, Volume I, Section IV-2
- (2) Letter to NRC dated May 19, 1979, "Transient of May 2, 1979"
- (3) General Electric Co. Letter G-EN-9-55, "Revised Natural Circulation Flow Calculation", dated May 29, 1979
- (4) Licensing Application Amendment 16, Design Requirements Section
- (5) (Deleted)
- (6) FDSAR, Volume I, Section IV-2.3.3 and Volume II, Appendix H
- (7) FDSAR, Volume I, Table IV-2-1
- (8) Licensing Application Amendment 34, Question 14

- (9) Licensing Application Amendment 28, Item III-B-2
- (10) Licensing Application Amendment 32, Question 15
- (11) (Deleted)
- (12) (Deleted)
- (13) Licensing Application Amendment 16, Page 1
- (14) GPUN TDR 725 Rev. 0: Testing and Evaluation of Irradiated Reactor Vessel Materials Surveillance Program Specimens.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 135

TO PROVISIONAL OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND
JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated March 31, 1988 (Ref. 1) GPU Nuclear Corporation (the licensee) requested changes to the Oyster Creek Technical Specifications. GPU provided additional information and clarifications in Ref. 2 & 2a. The change request proposes to delete a safety limit of the facility that requires at least two reactor recirculation loops to have both the recirculation pump suction and discharge valves open during all plant conditions except when the reactor head is off and the reactor is flooded to a level above the main steam nozzles. The purpose of the safety limit was to ensure adequate fluid communication between the downcomer and core regions in the reactor vessel so that water level sensed in the downcomer is indicative of water level in the core. This safety limit was imposed after the event on May 2, 1979 when all five recirculation loop valves were simultaneously closed effectively isolating the downcomer region from the core region. Later on September 11, 1987, a violation of the safety limit occurred when the plant was in cold shutdown and when fewer than two sets of recirculation loop valves were not fully open for a short period of time.

The licensee is proposing to delete the safety limit and requests to add a limiting condition for operation (LCO) to require at least one set of recirculation loop valves to be open during conditions other than normal operation.

The licensee is also proposing to allow all five reactor recirculation loops to be isolated if reactor coolant temperature is less than 212°F and the RPV is flooded to a level 185 inches above Top of Active Fuel (TAF). The licensee also provided a basis for Technical Specification 3.3.F.

In a letter dated June 15, 1988 (Ref. 3), the State of New Jersey submitted their comments regarding the proposed changes. They requested NRC to consider two alternatives. The first alternative they suggested is to require implementation of the recirculation loop electrical interlock modification (TMI-2 action item II.K.3.19). The second alternative they suggested is to keep the requirement that one recirculation loop be maintained in communication with the reactor as a safety limit.

2.0 EVALUATION

The current safety limit requirement of maintaining two open recirculation loops was originally proposed after the May 1979 event in order to be conservative. Since the requirement pertains to position requirement of equipment, specifically the reactor recirculation loop valves, rather than to any process variable, the requirement may be included as a LCO of the Technical Specifications. This is in conformance with the definitions of safety limits and LCO given in 10 CFR 50.36(c) and present staff practice. The process variable (reactor water level) is still given as a safety limit in 2.1.D of the plant Technical Specifications.

In the proposed LCO, the number of recirculation loops required to be open during conditions other than power operation is changed from two to one. Normal power operation is not allowed with less than four recirculation loops. In Ref. 4, the staff, in its evaluation of TMI-2 action item II.K.3.19, concluded that one open recirculation loop is sufficient to assure communication between the core and downcomer regions. Licensee calculations have verified that during natural circulation, a single fully open recirculation loop transfers coolant from the downcomer to the core region at approximately five times the boiloff rate (Ref. 5).

The conditions which require less than four recirculation loops are most likely during shutdown conditions when reactor water level is maintained at several feet above top of the active fuel. The recirculation loop closure alarm annunciates in the control room when the fourth recirculation loop is isolated.

This alarm will reflash when the fifth loop isolates. This alarm alerts the operator to open at least one loop. During shutdown conditions at Oyster Creek, if no action is taken upon isolating all five recirculation loops, it would take hours before boiloff of water would lower vessel level to the top of active fuel from the normal water level band. Moreover, fuel zone level instrumentation would remain available for operation. If the RPV level is 185 inches above TAF, there is sufficient hydraulic communication between the core and the downcomer region with all recirculation loop valves closed. Thus, the proposed changes to T/S 3.3.F.6 to isolate all five loops when the reactor coolant temperature is less than 212°F (cold shutdown) and when the RPV water level is 185 inches above TAF is acceptable.

The proposed T/S change 3.3.F.3 also requires the plant to be placed in a hot shutdown condition within 12 hours instead of present 24 hours for cold shutdown if less than four recirculation loops are operating during power operation. This proposed change is in conformance with standard Technical Specification guidance and hence it is acceptable.

Also by this amendment, the above technical considerations are reflected in the bases for Section 3.3 and we find this acceptable.

In Ref. 3, the state of New Jersey, requested the staff to consider implementation of the recirculation loop interlock modification. The staff in Ref. 4 already stated that the interlock modification is not necessary to satisfy TMI-2 action item II.K.3.19. The staff concluded that alarms plus adequate training should be sufficient to maintain one open loop. The staff is not aware of any new facts to change the position taken in Ref. 4. The state of New Jersey also proposed to keep the requirement that one loop be maintained in communication with the reactor as a safety limit. The safety limit is not required as discussed above.

The proposed request to delete safety limit 2.1.E, dealing with the position requirement of reactor recirculation loop valves, and to replace it with a LCO in T/S section 3.3.F.4 and other changes are acceptable.

The proposed T/S changes are acceptable as discussed in Section 2.0 of this report.

3.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 51.32, and 51.35, an environmental assessment and finding of no significant impact have been prepared and published in the Federal Register on December 29, 1989 (54 FR 53789). Accordingly, based upon the environmental assessment, we have determined that the issuance of the amendment will not have a significant effect on the quality of the human environment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security nor to the health and safety of the public.

5.0 REFERENCES

1. Letter dated March 31, 1988 from Peter B. Fiedler, Vice President and Director, Oyster Creek to USNRC.
2. Letter dated August 23, 1989 from E.E. Fitzpatrick, Vice President and Director, Oyster Creek to USNRC.

- 2.a Letter dated November 15, 1988 from M.W. Laggart, Manager, BWR Licensing, Oyster Creek to USNRC.
3. Letter dated June 15, 1988 from David M. Scott, Chief, Bureau of Nuclear Engineering, Department of Environmental Protection, Division of Environmental Quality, State of New Jersey to USNRC.
4. Letter dated April 16, 1988 from John A. Zwolinski, Director, BWR Project Directorate #1, Division of BWR Licensing, Office of NRR, NRC to P.B. Fiedler, Vice President and Director, Oyster Creek Nuclear Generating Station.
5. Letter dated May 12, 1979 from Ivan R. Finfrock, Jr., Vice President, Oyster Creek to USNRC, Appendix 1, GE Calculation "Natural Circulation Flow."
6. Letter dated November 15, 1988 from M.W. Laggart, Manager, BWR Licensing, Oyster Creek to USNRC.
7. Letter dated May 30, 1979 from Victor Stello Jr., Director, Division of Operating Reactors, Office of NRR, NRC to I. R. Finfrock, Jr., Vice President-Generation, Jersey Central Power & Light Company.

Dated: December 30, 1989

Principal Contributor: George Thomas

GPU NUCLEAR CORPORATIONANDJERSEY CENTRAL POWER & LIGHT COMPANYDOCKET NO. 50-219NOTICE OF ISSUANCE OF AMENDMENT TO
PROVISIONAL OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment No. 135 to Provisional Operating License No. DPR-16 issued to GPU Nuclear Corporation (the licensee), which revised the Technical Specifications for operation of the Oyster Creek Nuclear Generating Station located in Ocean County, New Jersey. The amendment is effective as of the date of issuance.

The amendment revises the Technical Specifications to delete Technical Specification Safety Limit 2.1.E which required at least two recirculation loops to be fully open except when the reactor vessel head is off and vessel water level is above the main steam nozzle. The amendment also incorporates this limitation in Technical Specification 3.3.F and that limitation is revised to require that at least one recirculation loop, instead of two be fully open during applicable plant conditions. The proposed change would also require that during power operations if at least four recirculation loops cannot be maintained inservice, then hot shutdown or refuel conditions must be reached in 12 hours. Presently, the requirement is that the unit be placed in cold shutdown conditions within 24 hours.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment and Opportunity for Hearing in connection with this action was published in the FEDERAL REGISTER on May 3, 1988 (53 FR 15756). No request for a hearing or petition for leave to intervene was filed following this notice.

The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement. Based upon the environmental assessment, the Commission has concluded that the issuance of this amendment will not have a significant effect on the quality of the human environment.

For further details with respect to the action see (1) the application for amendment dated March 31, 1988, and supplemented November 15, 1988, and August 23, 1989, (2) Amendment No. 135 to License No. DPR-16, (3) the Commission's related Safety Evaluation, and (4) the Commission's Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street N.W., Washington, D.C. and at the Ocean County Library, Reference Department, 101 Washington Street, Toms River, New Jersey 08753.

A copy of items (2), (3) and (4) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Projects - I/II.

Dated at Rockville, Maryland this 30th day of December 1989.

FOR THE NUCLEAR REGULATORY COMMISSION


Alexander W. Dromerick, Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation