



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

October 18, 1985

*Am dt 90 to DPR 11*

Docket No. 50-219  
LS05-85-10-027

Mr. P. B. Fiedler  
Vice President and Director  
Oyster Creek Nuclear Generating Station  
Post Office Box 388  
Forked River, New Jersey 08731

Dear Mr. Fiedler:

SUBJECT: INSERVICE INSPECTION AND TESTING

Re: Oyster Creek Nuclear Generating Station

The Commission has issued the enclosed Amendment No. 90 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment is in response to your application dated February 11, 1985.

This amendment authorizes changes to the Oyster Creek Appendix A Technical Specifications (TS) to revise Inservice Inspection (ISI) and Inservice Testing (IST) requirements in Section 4.3, Reactor Coolant, of the TS. The changes revise the existing Paragraphs 4.3.D and 4.3.E and Bases of Section 4.3 to replace existing detailed ISI and IST requirements in the TS with references to Section XI of the ASME Boiler and Pressure Code, the details of which are implemented in the station's ISI and IST Programs.

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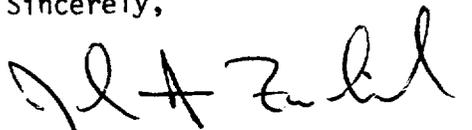
Mr. P. B. Fiedler

- 2 -

October 18, 1985

A copy of our related Safety Evaluation for this amendment is also enclosed. A notice of issuance pertaining to this action will appear in the Commission's biweekly notice publication in the Federal Register.

Sincerely,

A handwritten signature in black ink, appearing to read "John A. Zwolinski". The signature is written in a cursive style with a large initial "J" and "Z".

John A. Zwolinski, Chief  
Operating Reactors Branch #5  
Division of Licensing

Enclosures:

1. Amendment No. 90 to  
License No. DPR-16
2. Safety Evaluation

cc w/enclosures:  
See next page

Mr. P. B. Fiedler

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*Original signed by*

John A. Zwolinski, Chief  
Operating Reactors Branch #5  
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cc w/enclosures:  
See next page

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Mr. P. B. Fiedler  
Oyster Creek Nuclear Generating Station

Oyster Creek Nuclear  
Generating Station

cc:

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D. G. Holland  
Licensing Manager  
Oyster Creek Nuclear Generating Station  
Post Office Box 388  
Forked River, New Jersey 08731



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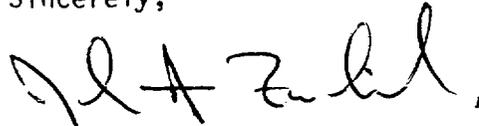
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Operating Reactors Branch #5  
Division of Licensing

Enclosures:

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2. Safety Evaluation

cc w/enclosures:  
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Oyster Creek Nuclear Generating Station

Oyster Creek Nuclear  
Generating Station

cc:

G. F. Trowbridge, Esquire  
Shaw, Pittman, Potts and Trowbridge  
1800 M Street, N.W.  
Washington, D.C. 20036

Resident Inspector  
c/o U.S. NRC  
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New Jersey Department of Energy  
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GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 90  
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by GPU Nuclear Corporation and Jersey Central Power and Light Company (the licensees) dated February 11, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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PDR ADOCK 05000219  
P PDR

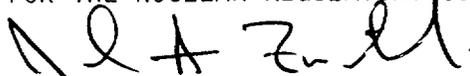
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 90, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John A. Zwolinski, Chief  
Operating Reactors Branch #5  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 18, 1985

ATTACHMENT TO LICENSE AMENDMENT NO.90

PROVISIONAL OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain vertical lines indicating the area of change.

REMOVE

4.3-1  
4.3-3

INSERT

4.3-1  
4.3-3

**Applicability:** Applies to the surveillance requirements for the reactor coolant system.

**Objective:** To determine the condition of the reactor coolant system and the operation of the safety devices related to it.

- Specification:**
- A. Neutron flux monitors shall be installed in the reactor vessel adjacent to the vessel wall at the core midplane level. The monitors shall be removed and tested at the first refueling outage to experimentally verify the calculated values of integrated neutron flux that are used to determine the NDTT from Figure 3.3.1.
  - B. Inservice inspection of ASME Code Class 1, Class 2 and Class 3 systems and components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10 CFR, Section 50.55a(g)(6)(i).
  - C. Inservice testing of ASME Code Class 1, Class 2 and Class 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10 CFR, Section 50.55a(g)(6)(i).
  - D. A visual examination for leaks shall be made with the reactor coolant system at pressure during each scheduled refueling outage or after major repairs have been made to the reactor coolant system in accordance with Article 5000, Section XI. The requirements of specification 3.3.A shall be met during the test.
  - E. Each replacement safety valve or valve that has been repaired shall be tested in accordance with subsection IWV-3510 of Section XI of the ASME Boiler and Pressure Vessel Code. Setpoints shall be as follows:

<u>Number of Valves</u>	<u>Set Point (psig)</u>
4	1212 ± 12
4	1221 ± 12
4	1230 ± 12
4	1239 ± 12

Bases:

Numerous data are available relating integrated flux and the change in Nil-Ductility Transition Temperature (NDTT) in various steels. The base metal has been demonstrated to be relatively insensitive to neutron irradiation (see expected NDT changes in FDSAR Table IV-1-1, and Figures IV-2-9 and IV-2-10). The most conservative data has been used in Specification 3.3. The integrated flux at the vessel wall is calculated from core physics data and will be measured using flux monitors installed inside the vessel. The measurements of the neutron flux at the vessel wall will be used to check and if necessary correct, the calculated data to determine an accurate flux. From this a conservative NDT temperature can be determined. Since no shift will occur until an integrated flux of  $10^{17}$  nvt is reached, the confirmation can be made long before an NDTT shift would occur.

The inspection program will reveal problem areas should they occur, before a leak develops. In addition, extensive visual inspection for leaks will be made on critical systems. Oyster Creek was designed and constructed prior to the existence of ASME Section XI. For this reason, the degree of access required by ASME Section XI is not generally available and will be addressed as "requests for relief" in accordance with 10 CFR 50.55a(g).

Experience in safety valve operation shows testing in accordance with Section XI of the ASME Boiler and Pressure Vessel Code is adequate to detect failures or deterioration. The tolerance value is specified in Section I of the ASME Code at +1% of design pressure. An analysis has been performed which shows that with all safety valves set 12 psig higher the safety limit of 1375 psig is not exceeded.

Conductivity instruments continuously monitor the reactor coolant. Experience indicates that a check of the conductivity instrumentation at least every 72 hours is adequate to ensure accurate readings. The reactor water sample will also be used to determine the chloride ion content to assure that the limits of 3.3.E are not exceeded. The chloride ion content will not change rapidly over a period of several days; therefore, the sampling frequency is adequate.



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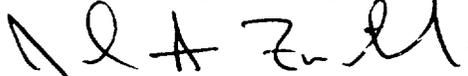
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FOR THE NUCLEAR REGULATORY COMMISSION



John A. Zwolinski, Chief  
Operating Reactors Branch #5  
Division of Licensing

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Date of Issuance: October 18, 1985

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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 90 TO PROVISIONAL OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND

JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated February 11, 1985, GPU Nuclear (the licensee) requested an amendment to the Appendix A Technical Specifications (TS) to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station (Oyster Creek). This amendment would replace certain existing detailed Inservice Inspection (ISI) and Inservice Testing (IST) requirements in Section 4.3, Reactor Coolant, of the TS with references to Section XI of the ASME Boiler and Pressure Code, the details of which are implemented by the licensee's ISI and IST Programs for Oyster Creek.

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2.0 DISCUSSION AND EVALUATION

Discussion and Evaluation

On February 27, 1976, the Nuclear Regulatory Commission revised the inservice inspection and testing requirements for ASME Code Class 1, 2, and 3 components for nuclear power plants in 10 CFR 50.55a. The revised regulations require inservice inspection and testing set forth in Section XI of the ASME Boiler and Pressure Vessel Code and Addenda. A review by the Commission of the 1974 edition ASME Section XI indicated that conflicts may occur between the ASME code requirements and the plant TS. To avoid such conflicts, the Commission requested that the licensee, in accordance with 50.55a(g)(5)(ii), apply for an amendment to the plant TS to replace such conflicting TS with a reference to 10 CFR 50.55a.

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P PDR

The licensee proposed by an amendment request dated June 8, 1984 to incorporate the requirements of the revised regulations on ISI and IST in the plant TS. The license amendment was granted, and the date of its issuance was May 22, 1985. The licensee proposed by an amendment request dated February 11, 1985, to further revise its ISI and IST requirements. In this amendment request, the text is revised to refer to 10 CFR 50.55a and the ASME Boiler and Pressure Vessel Code, Section XI for specific details.

The staff has reviewed the February 11, 1985 changes proposed by the licensee. The proposed changes revise TS 4.3.D and 4.3.E to incorporate requirements of the revised 10 CFR 50.55a and to avoid conflicts between the ASME Section XI requirements and the plant TS. The proposed change in 4.3.D inserts a reference to the appropriate Section XI, Article 5000. The proposed change in 4.3.E replaces text detailing requirements for the minimum number of valves to be bench checked or replaced with the equivalent, with an explicit reference to subsection IWV-3510 of Section XI. These additional TS changes are consistent with the guidance provided by the staff to the licensee in the letter dated April 26, 1976. Therefore, based on the above, the staff concludes that the proposed changes to Section 4.3 of the Appendix A TS in the licensee's letter dated February 11, 1985, are acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

### 5.0 ACKNOWLEDGEMENT

This evaluation was prepared by S. D. Kucharski, Region I.

Dated: October 18, 1985



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On February 27, 1976, the Nuclear Regulatory Commission revised the inservice inspection and testing requirements for ASME Code Class 1, 2, and 3 components for nuclear power plants in 10 CFR 50.55a. The revised regulations require inservice inspection and testing set forth in Section XI of the ASME Boiler and Pressure Vessel Code and Addenda. A review by the Commission of the 1974 edition ASME Section XI indicated that conflicts may occur between the ASME code requirements and the plant TS. To avoid such conflicts, the Commission requested that the licensee, in accordance with 50.55a(g)(5)(ii), apply for an amendment to the plant TS to replace such conflicting TS with a reference to 10 CFR 50.55a.

The licensee proposed by an amendment request dated June 8, 1984 to incorporate the requirements of the revised regulations on ISI and IST in the plant TS. The license amendment was granted, and the date of its issuance was May 22, 1985. The licensee proposed by an amendment request dated February 11, 1985, to further revise its ISI and IST requirements. In this amendment request, the text is revised to refer to 10 CFR 50.55a and the ASME Boiler and Pressure Vessel Code, Section XI for specific details.

The staff has reviewed the February 11, 1985 changes proposed by the licensee. The proposed changes revise TS 4.3.D and 4.3.E to incorporate requirements of the revised 10 CFR 50.55a and to avoid conflicts between the ASME Section XI requirements and the plant TS. The proposed change in 4.3.D inserts a reference to the appropriate Section XI, Article 5000. The proposed change in 4.3.E replaces text detailing requirements for the minimum number of valves to be bench checked or replaced with the equivalent, with an explicit reference to subsection IWV-3510 of Section XI. These additional TS changes are consistent with the guidance provided by the staff to the licensee in the letter dated April 26, 1976. Therefore, based on the above, the staff concludes that the proposed changes to Section 4.3 of the Appendix A TS in the licensee's letter dated February 11, 1985, are acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

### 5.0 ACKNOWLEDGEMENT

This evaluation was prepared by S. D. Kucharski, Region I.

Dated: October 18, 1985