

June 11, 1982

DISTRIBUTION
Docket
NRC PDR
Local PDR
ORB Reading
DCrutchfield
HSmith
JLombardo
OELD
OI&E (2)
ACRS (10)
SEPB
RDiggs
LSchneider
TBarnhart (4)

Docket No. 50-219
LS05-82-06-029

Mr. P. B. Fiedler
Vice President and Director
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

Dear Mr. Fiedler:

SUBJECT: SURVEILLANCE REQUIREMENTS

The Commission has issued the enclosed Amendment No. 61 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment consists of changes to the Appendix A Technical Specifications in response to your application dated May 28, 1982.

This amendment authorizes a one time only change to Section 1.12, Definition of Refueling Outage, to extend the time between successive tests or surveillances, prior to cycle 10 only, from not exceeding 20 months to not exceeding 30 months.

Copies of our related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

*SEOI
DSU USE (51)*

Enclosures:

1. Amendment No. 61 to License No. DPR-16
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

*F.R. NOTICE
AMENDMENT*

8206240131 820611
PDR ADDCK 05000219
P PDR

OFFICE	DL: ORB #5	DL: ORB #5	OELD	DL: ORB #5	DL: AD/SA	DE: NTEB
SURNAME	HSmith:cc	JLombardo	W... ..	DCrutchfield	GLainas	Whazelton
DATE	6/7/82	6/8/82	6-8-82	6/8/82	6/11/82	6/11/82

June 11, 1982

CC

G. F. Trowbridge, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

Resident Inspector
c/o U. S. NRC
Post Office Box 445
Forked River, New Jersey 08731

J. B. Lieberman, Esquire
Berlack, Israels & Lieberman
26 Broadway
New York, New York 10004

Commissioner
New Jersey Department of Energy
101 Commerce Street
Newark, New Jersey 07102

Ronald C. Haynes, Regional Administrator
Nuclear Regulatory Commission, Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

J. Knubel
BWR Licensing Manager
GPU Nuclear
100 Interplace Parkway
Parsippany, New Jersey 07054

Deputy Attorney General
State of New Jersey
Department of Law and Public Safety
36 West State Street - CN 112
Trenton, New Jersey 08625

Mayor
Lacey Township
818 Lacey Road
Forked River, New Jersey 08731

U. S. Environmental Protection
Agency
Region II Office
ATTN: Regional Radiation Representative
26 Federal Plaza
New York, New York 10007

Licensing Supervisor
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 61
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear Corporation and Jersey Central Power and Light Company (the licensees) dated May 28, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.


2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 61, are hereby incorporated in the license. GPU Nuclear Corporation operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 11, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 61

PROVISIONAL OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace page 1.0-2 of the Appendix "A" Technical Specifications with the enclosed page 1.0-2. The revised page contains the captioned amendment number and vertical lines indicating the area of change.

1.8 PLACE IN SHUTDOWN CONDITION

Proceed with and maintain an uninterrupted normal plant shutdown operation until the shutdown condition is met.

1.9 PLACE IN COLD SHUTDOWN CONDITION

Proceed with and maintain an uninterrupted normal plant shutdown operation until the cold shutdown condition is met.

1.10 PLACE IN ISOLATED CONDITION

Proceed with and maintain an uninterrupted normal isolation of the reactor from the turbine condenser system including closure of the main steam isolation valves.

1.11 REFUEL MODE

The reactor is in the refuel mode when the reactor mode switch is in the refuel mode position and there is fuel in the reactor vessel. In this mode the refueling platform interlocks are in operation.

1.12 REFUELING OUTAGE

For the purpose of designating frequency of testing and surveillance, a refueling outage shall mean a regularly scheduled refueling outage; however, where such outages occur within 8 months of the end of the previous refueling outage, the test or surveillance need not be performed until the next regularly scheduled outage. Following the first refueling outage, the time between successive tests or surveillance shall not exceed 20 months.*

1.13 PRIMARY CONTAINMENT INTEGRITY

Primary containment integrity means that the drywell and adsorption chamber are closed and all of the following conditions are satisfied:

- A. All non-automatic primary containment isolation valves which are not required to be open for plant operation are closed.
- B. At least one door in the airlock is closed and sealed.
- C. All automatic containment isolation valves specified in Table 3.5.2 are operable or are secured in the closed position.
- D. All blind flanges and manways are closed.

1.14 SECONDARY CONTAINMENT INTEGRITY

Secondary containment integrity means that the reactor building is closed and the following conditions are met:

* The time between successive tests or surveillances shall not exceed 30 months prior to the cycle 10 refueling outage only.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

FOR OYSTER CREEK NUCLEAR GENERATING STATION

SUPPORTING AMENDMENT NO. 61 TO PROVISIONAL OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated May 28, 1982 GPU Nuclear Corporation and Jersey Central Power & Light Company (the licensees) requested an amendment to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station.

The amendment would authorize a one time only change to Section 1.12, Definition of Refueling Outage, to extend the time between successive tests of surveillances, prior to cycle 10 only, from not exceeding 20 months to not exceeding 30 months. The last refueling/maintenance outage (cycle 9) was completed July 20, 1980. From July 20, 1980 to the present approximately ten months of unscheduled maintenance outages have occurred. It is the expressed purpose of this request to extend the surveillance interval to be in keeping with the projected cycle 10 refueling/maintenance outage scheduled for January 15, 1983.

2.0 DISCUSSION AND EVALUATION

The present Appendix A Technical Specification Section 1.12, Definition of Refueling Outage, requires that after the first refueling outage, the time between successive tests or surveillance shall not exceed 20 months. GPU Nuclear Corporation has requested that for one time only, prior to the cycle 10 refueling outage, this requirement be extended from 20 months to 30 months. The surveillances impacted by this change request are (1) inspection of four torus-to-drywell vacuum breakers, (2) replacement of five safety valves, (3) inspection of the core spray sparger, and (4) visual inspection of the interior of the torus.

For Item 1, GPU Nuclear Corporation has stated that the vacuum breakers are operability tested once each month, and following any release of energy which would tend to increase pressure to the suppression chamber per Technical Specification Section 4.5.I.5.a. This monthly operability test will give assurance that the vacuum breakers are able to perform their safety function during extension of the surveillance interval.

With respect to Item 2, GPU Nuclear Corporation has stated that the valves will perform as required to relieve pressure to prevent the reactor pressure from exceeding the safety limit of 1375 psig. Sixteen main steam safety valves from another nuclear power plant which are of the same type as those used at Oyster Creek, were tested at the Wyle Laboratories. These tests indicated that 4 valves were within proper setpoint tolerance and that the remaining 12 valves had "as found" set points of 20 - 50 psi below the required set points. This experience along with other similar valves indicated that when valves were out of setpoint tolerance the drift was also downward.

In regard to Item 3, GPU Nuclear Corporation states there is a high probability that additional cracking if any during the requested surveillance extension time will not be of significance to affect the intended function of the core spray system. Their conclusion is based on (1) extensive additional cracking is not likely in the next cycle of operation; (2) existing, and new cracks, should there be any are not likely to propagate during normal operation; (3) the only significant operational loads that can be postulated are those associated with initiation of the core spray system, and significant crack propagation is not expected; (4) for reasonable assumptions on the existing crack size and propagation, analyses demonstrates that the design nominal flow and distribution of the core spray system can be maintained within acceptable limits and that a margin remains for new cracks should there be any; (5) the clamp assemblies presently in place at all significant crack locations provide for additional structural capability of the sparger and minimize the potential for geometric changes that are significant to the hydraulic performance to occur; and (6) flow out of the cracks is unlikely to significantly affect the overall spray distribution. A repair proposal and Safety Evaluation was presented to the NRC on April 1, 1980. Based on a review of the licensee's submittal, Amendment No. 47 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station was issued on May 15, 1980.

For Item 4, GPU Nuclear corporation stated that based on past inspection history of not finding any significant problems, the requested surveillance extension would not create a condition adverse to the health and safety of the public. Their conclusion is supported by the following discussion. In the early seventies, inspections resulted in their finding loose baffles which were removed at that time. The remaining baffles, although not loose, were removed in 1977. Moreover, a general corrosion on the shell was noted during these inspections and corrective action was taken to remove this corrosion and to build up the areas with welding. The corrosion problem was not considered significant.

Based on our review we conclude that the proposed change to the Technical Specifications would not adversely impact the safety of the plant. We, therefore, find that the request to extend the surveillance interval from 20 months to 30 months on a one time basis prior to the Cycle 10 refueling outage is acceptable. This change request does not affect Section 1.24, Surveillance Requirements, in any way.

3.0 ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §1.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

5.0 ACKNOWLEDGMENTS:

This evaluation has been prepared by J. Lombardo.

Date: June 11, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-219GPU NUCLEAR CORPORATION ANDJERSEY CENTRAL POWER & LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 61 to Provisional Operating License No. DPR-16, issued to GPU Nuclear Corporation and Jersey Central Power & Light Company (the licensees), which revised the Technical Specifications for operation of the Oyster Creek Nuclear Generating Station (the facility) located in Ocean County, New Jersey. The amendment is effective as of its date of issuance.

This amendment authorizes a one time only change to Section 1.12, Definition of Refueling Outage, to extend the time between successive tests of surveillances, prior to cycle 10 only, from not exceeding 20 months to not exceeding 30 months.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.


- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated May 28, 1982, (2) Amendment No. 61 to License No. DPR-16, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and the Local Public Document Room, 101 Washington Street, Toms River, New Jersey 08753. A single copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 11th day of June, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing