

November 5, 1982

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Docket No. 50-219  
LS05-82-11-024

Mr. P. B. Fiedler  
Vice President and Director - Oyster Creek  
Oyster Creek Nuclear Generating Station  
Post Office Box 388  
Forked River, New Jersey 08731

Dear Mr. Fiedler:

SUBJECT: ADMINISTRATIVE ORGANIZATIONAL CHANGES - OYSTER CREEK

The Commission has issued the enclosed Amendment No. 65 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment consists of changes to the Technical Specifications in response to your application dated June 15, 1982.

The amendment authorizes changes to the administrative organization at the Oyster Creek plant by, (1) retitling the position of Director Station Operations to Deputy Director Oyster Creek, (2) assigning the same Technical Specification responsibilities of the Director Station Operations to the Vice President and Director Oyster Creek, (3) retitling the position of Fire Protection Manager to Fire Protection Supervisor, and (4) retitling the position of Radiological Controls Foreman to Radiological Controls Group Supervisor.

During our review we noted that the Administrative Controls section of the Oyster Creek Technical Specification does not adequately address the Fire Protection program, specifically in the areas covering audit and procedure requirements. We request that you submit an appropriate Technical Specification Change Request within 60 days.

Copies of our related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by/

Dennis M. Crutchfield, Chief  
Operating Reactors Branch #5  
Division of Licensing

DL:ORB#5  
 DCrutchfield  
 10/5/82  
 DL:AD-SA  
 FMiraglia  
 10/5/82

Enclosures:  
As stated

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DSA USE(SI)*

OFFICE	cc:w/enclosures:	DL:ORB#5	DL:ORB#5	DL:ORB#5	DL:ORB#5	OELD
SURNAME	See next page	HSmith	JLombardo	TWambach	H Rooper	H Rooper
DATE		10/28/82	10/29/82	10/28/82	10/29/82	10/3/82

November 5, 1982

cc

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c/o U. S. NRC  
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Licensing Supervisor  
Oyster Creek Nuclear Generating Station  
Post Office Box 388  
Forked River, New Jersey 08731



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO THE AMENDED PROVISIONAL OPERATING LICENSE

Amendment No. 65  
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by GPU Nuclear Corporation and Jersey Central Power and Light Company (the licensees) dated June 15, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 65 are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Dennis M. Crutchfield, Chief  
Operating Reactors Branch #5  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 5, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 65

PROVISIONAL OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace the following pages of the Technical Specifications with the enclosed pages. The revised pages are identified by the captioned amendment number and contain vertical lines indicating the area of change.

	<u>Page*</u>
(Appendix A)	6-1
	6-4
	6-5
	6-7 through
	6-16
(Appendix B)	5-3

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Page 6-12 is merely included for pagination purposes.

## ADMINISTRATIVE CONTROLS

### 6.1 RESPONSIBILITY

#### 6.1.1

The Vice President & Director shall be responsible for overall facility operation. Those responsibilities delegated to the Vice President & Director as stated in the Oyster Creek Technical Specifications may also be fulfilled by the Deputy Director. The Vice President & Director shall delegate in writing the succession to this responsibility during his and/or the Deputy Directors absence.

### 6.2 ORGANIZATION

#### 6.2.1 OFFSITE

The organization for GPU Nuclear Corporation for management and technical support shall be functionally as shown on Figure 6.2.1.

#### 6.2.2 FACILITY STAFF

The facility organization shall be as shown on Figure 6.2.2 and:

a. Each on duty shift shall include at least the shift staffing indicated on Figure 6.2.2.

b. At least one licensed reactor operator shall be in the control room when fuel is in the reactor.

c. Two licensed reactor operators shall be in the control room during all reactor startups, shutdowns, and other periods involving planned control rod manipulations.

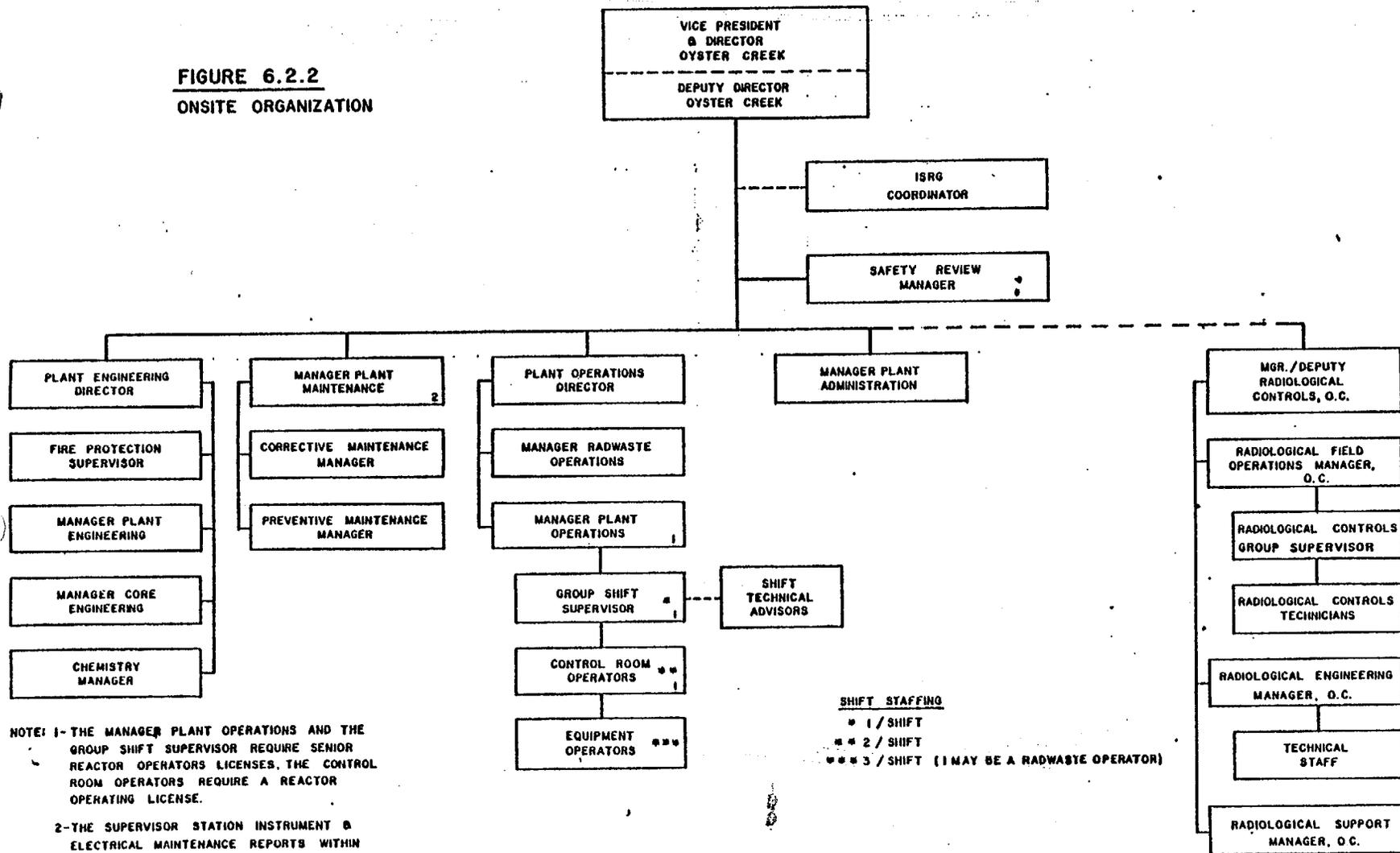
d. ALL CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

e. An individual qualified in radiation protection measures shall be on site when fuel is in the reactor.

f. A Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of the unit or any personnel required for other essential functions during a fire emergency.

g. Each on duty shift shall include a Shift Technical Advisor except that the Shift Technical Advisors position need not be filled if the reactor is in the refuel or shutdown mode and the reactor is less than 212°F.

**FIGURE 6.2.2**  
**ONSITE ORGANIZATION**



NOTE: 1- THE MANAGER PLANT OPERATIONS AND THE GROUP SHIFT SUPERVISOR REQUIRE SENIOR REACTOR OPERATORS LICENSES, THE CONTROL ROOM OPERATORS REQUIRE A REACTOR OPERATING LICENSE.

2- THE SUPERVISOR STATION INSTRUMENT & ELECTRICAL MAINTENANCE REPORTS WITHIN THIS GROUP.

## 6.3 FACILITY STAFF QUALIFICATIONS

### 6.3.1

The members of the facility staff shall meet or exceed the following qualifications:

#### Vice President & Director/Deputy Director

Requirements: Ten years total power plant experience of which three years must be nuclear power plant experience. A maximum of four years of academic training may fulfill four of the remaining seven years of required experience. Both must be capable of obtaining or possess a Senior Reactor Operator's License.

#### Plant Operations Director

Requirements: Eight years total power plant experience of which three years must be nuclear power plant experience. A maximum of two years of academic or related technical training may fulfill two years of the remaining five years of required experience. The Plant Operations Director must be capable of obtaining or possess a Senior Reactor Operator's License.

#### Plant Engineering Director

Requirements: Eight years of responsible positions related to power generation, of which three years shall be nuclear power plant experience. A maximum of four of the remaining five years of experience may be fulfilled by satisfactory completion of academic or related technical training.

#### Manager Plant Administration

Requirements: Eight years total power plant experience of which four years must have been in nuclear power plant experience. The Manager should possess a four year college degree or equivalent in Business Administration or an Engineering discipline.

#### Manager Plant Operations

Requirements: Eight years total power plant experience of which three years must be nuclear power plant experience. A maximum of two years of academic or related technical training may fulfill two of the remaining five years of required experience. The Manager Plant Operations must possess a Senior Reactor Operator's License.

#### Safety Review Manager

Requirements: Eight years total power plant experience of which three years must be nuclear power plant experience. A maximum of two years of academic or related technical training may fulfill two of the remaining five years of required experience.

plant. A maximum of four years of this five year experience may be fulfilled by related technical or academic training.

#### Shift Technical Advisor

Requirements: Bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

### 6.3.2

Each member of the radiation protection organization for which there is a comparable position described in ANSI N18.1-1971 shall meet or exceed the minimum qualifications specified therein, or in the case of radiation protection technicians, they shall have at least one year's continuous experience in applied radiation protection work in a nuclear facility dealing with radiological problems similar to those encountered in nuclear power stations, and shall have been certified by the Mgr/Deputy Radiological Controls, as qualified to perform assigned functions. This certification must be based on an NRC approved, documented program consisting of classroom training with appropriate examinations and documented positive findings by responsible supervision that the individual has demonstrated his ability to perform each specified procedure and assigned function with an understanding of its basis and purpose.

## 6.4 TRAINING

### 6.4.1

A retraining program for operators shall be maintained under the direction of the Manager Plant Training Oyster Creek and shall meet the requirements and recommendation of Appendix A of 10CFR Part 55. Replacement training programs, the content of which shall meet the requirements of 10CFR Part 55, shall be conducted under the direction of the Manager Plant Training Oyster Creek for licensed operators and Senior Reactor Operators.

### 6.4.2

A training program for the Fire Brigade shall be maintained under the direction of the Manager Plant Training Oyster Creek.

## 6.5 SAFETY REVIEW AND AUDIT

The Vice President & Director and three organizational units, the Plant Operations Review Committee (PORC), the Independent Safety Review Groups (ISRG) and the General Office Review Board (GORB) function to accomplish nuclear safety review and audit of the Oyster Creek Station.

## 6.5.1 VICE PRESIDENT & DIRECTOR

### 6.5.1.1 FUNCTION

The Vice President & Director shall ensure that:

- a. All proposed changes to equipment or systems have been evaluated to determine if they constitute a change to the facility or procedures as described in the Safety Analysis Report.
- b. All proposed changes to equipment or systems that constitute a change of the facility or procedures as described in the Safety Analysis Report have been evaluated to determine whether they involve an unreviewed safety question as defined in paragraph 50.59, Part 50, Title 10, Code of Federal Regulations.
- c. All proposed tests and experiments have been evaluated to determine whether or not they involve unreviewed safety questions as defined in paragraph 50.59, Part 50, Title 10, Code of Federal Regulations.

### 6.5.1.2 AUTHORITY

The Vice President & Director has the authority to:

- a. Make a determination as to whether proposed changes to equipment, or systems involve a change to the procedures or facility as described in the Safety Analysis Report.
- b. Make a determination as to whether or not proposed tests or experiments and changes to equipment or systems involve an unreviewed safety question.
- c. Direct the Plant Operations Review Committee to review safety-evaluations of proposed changes to equipment or systems and safety-evaluations of proposed tests and experiments to determine whether or not such changes, tests or experiments involve unreviewed safety questions.

NOTE: Each determination that a proposed test, experiment, or change to a system or equipment that does not involve an unreviewed safety question shall be reviewed by the Independent Safety Review Groups to verify that the determination was correct. This review shall be documented but is not a pre-requisite of the test, experiment, or change to a system or equipment.

### 6.5.1.3 RECORDS

Any safety evaluations done in accordance with 6.5.1.1 (b) and (c) and any determinations made pursuant to 6.5.1.2(b) must be documented. Copies of these determinations shall be provided to the ISRG Coordinator and the Chairman of the General Office Review Board. Records of all tests and experiments performed and all changes to equipment or systems made under the provisions of 10 CFR

Part 50.59 shall also be maintained at the station.

6.5.2 PLANT OPERATIONS REVIEW COMMITTEE (PORC)

6.5.2.1 FUNCTION

The PORC shall function to advise the Vice President & Director.

6.5.2.2 COMPOSITION

The PORC shall consist of the following plant personnel:

Safety Review Manager  
Plant Operations Director  
Plant Engineering Director  
Manager Plant Maintenance  
Mgr/Deputy Radiological Controls

The Vice President & Director shall designate the Chairman and Vice-Chairman from among the PORC members.

6.5.2.3 ALTERNATES

Alternate members shall be appointed in writing by the PORC Chairman and will have the type of experience and training required of regular members. However, they need not have the extensive longevity in the designated fields as long as in the opinion of the Chairman, their experience and judgement are adequate.

6.5.2.4 MEETING FREQUENCY

The PORC shall meet at least once per calendar month and as convened by the PORC Chairman or the Vice President & Director.

6.5.2.5 QUORUM

A quorum of the PORC shall consist of the Chairman or Vice Chairman and three members/alternates. No more than two alternate members shall be counted in establishing a quorum.

6.5.2.6 RESPONSIBILITIES

The responsibilities of the PORC are included in Table 6.5.1.

6.5.2.7 AUTHORITY

a. The PORC shall be advisory to the Vice President & Director. Nothing herein shall relieve the Vice President & Director of his responsibility or authority for overall safety operations including taking immediate emergency action. Determinations on Items a and b of Table 6.5-1 shall be documented in writing.

#### 6.5.2.8 RECORDS

The PORC shall maintain written minutes of each meeting and copies of minutes and determinations shall be provided to the Vice President & Director, ISRG Coordinator, and the Chairman of the GORB.

#### 6.5.3 INDEPENDENT SAFETY REVIEW GROUPS (ISRG)

##### 6.5.3.1 FUNCTION AND COMPOSITION

The ISRG shall function under the direction of an ISRG Coordinator, who shall be appointed by the Vice President & Director to provide safety reviews. The ISRG Coordinator shall have available the competence to review problems in the following area:

- a. Nuclear Power Plant Operations
- b. Nuclear Engineering
- c. Chemistry and Radiochemistry
- d. Metallurgy
- e. Instrumentation and Control
- f. Radiological Safety
- g. Mechanical and Electrical Engineering
- h. Quality Assurance Practices

The Coordinator shall establish, as needed, groups of two or more individuals with the expertise required for each topic to be reviewed.

##### 6.5.3.2 CONSULTANTS

Consultants shall be utilized as necessary to supplement the expertise available in the GPU Nuclear Corporation.

##### 6.5.3.3 RESPONSIBILITIES

The specific responsibility to ensure accomplishment of the independent safety review of the Vice President & Director determinations involving safety questions is assigned to the ISRG Coordinator and is accomplished by utilizing, as necessary, the full scope of expertise available in the GPU Nuclear Corporation staff, consultants, contractors and vendors as appropriate. Table 6.5-1 defines the specific independent safety review responsibilities.

##### 6.5.3.4 AUTHORITY

The ISRG advises the Vice President & Director. It has the authority to conduct reviews and investigations, which will be documented.

##### 6.5.3.5 AUDITS

Audits of facility activities shall be performed under the cognizance of the Vice President Nuclear Assurance. These audits shall encompass:

a. The conformance of facility operation to all provisions contained within the Technical Specifications and applicable license conditions at least once per year.

b. The performance training and qualifications of the entire facility staff at least once per year.

c. The results of all actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.

d. The Facility Emergency Plan and implementing procedures at least once per two years.

e. The Facility Security Plan and implementing procedures at least once per two years.

f. Any other area of facility operation considered appropriate by the GORB or the Vice President & Director.

#### 6.5.3.6 RECORDS

Written documentation of all independent safety reviews and investigations will be forwarded to the Vice President & Director and the Chairman of the General Office Review Board. In addition, any reportable occurrence or item involving an unreviewed safety question which is identified by the ISRG will be documented and reported immediately to the above mentioned persons.

The audit findings which result from all audits conducted in accordance with Section 6.5.3.5 shall be documented and reported to the above mentioned persons within 30 days after completing the audit. Reports documenting corrective action will receive the same distribution and they will also be forwarded to the ISRG Coordinator.

#### 6.5.4 GENERAL OFFICE REVIEW BOARD (GORB)

##### 6.5.4.1 FUNCTION

The technical and administrative function of the GORB is to provide independent review of major safety issues, to foresee potentially significant nuclear and radiation safety problems, and to advise the Office of the President on these matters.

##### 6.5.4.2 COMPOSITION

Members of the General Office Review Board shall possess extensive experience in their individual specialities and collectively have the competence in the following areas:

- a. Nuclear Power Plant Operations
- b. Nuclear Engineering
- c. Chemistry and Radiochemistry

- d. Metallurgy
- e. Instrumentation and Control
- f. Radiological Safety
- g. Mechanical and Electrical Engineering

The Chairman and Vice Chairman shall be appointed by the Office of the President. (Neither shall be an individual with line responsibility for operation of the plant).

The Chairman shall designate a minimum of six additional members. No more than a minority of the Board shall have line responsibility for operation of Oyster Creek Nuclear Generating Station.

#### 6.5.4.3 ALTERNATES

Alternate members shall be appointed in writing by the GORB Chairman and will have the type of experience and training required of regular members, however, they need not have the extensive longevity in the designated fields as long as, in the opinion of the Chairman, their experience and judgement are adequate.

#### 6.5.4.4 MEETING FREQUENCY

The GORB shall meet at least semi-annually and any time at the request of the Chairman or the Office of the President.

#### 6.5.4.5 QUORUM

A quorum shall consist of the Chairman or Vice Chairman and three members/alternates. No more than one alternate member shall be counted when establishing a quorum and no more than a minority of the quorum shall hold line responsibility for operations of the Oyster Creek Station.

#### 6.5.4.6 RESPONSIBILITIES

a. The primary responsibility of the GORB is to foresee potentially significant nuclear and radiation safety problems and to recommend to the Office of the President how they may be avoided or mitigated.

b. Carry out the specific independent safety review responsibilities listed in Table 6.5-1.

#### 6.5.4.7 AUTHORITY

The GORB shall be advisory to the Office of the President and shall have the authority to conduct reviews, audits, and investigations requested by the Office of the President or as deemed necessary by the GORB in the fulfillment of its responsibilities.

#### 6.5.4.8 AUDITS

The report of the management review of the QA Plan, initiated by the Vice President & Director in accordance with the Operational Quality Assurance Plan, shall be reviewed by the GORB with respect to safety and administrative safety issues.

#### 6.5.4.9 RECORDS

Minutes of each GORB meeting shall be recorded and approved by the GORB Chairman. Copies of approved minutes will be forwarded to the Office of the President, Vice President & Director, PORC Chairman, and others designated by the GORB Chairman. GORB recommendations to the Office of the President will be documented in a letter from the GORB Chairman to the Office of the President. Included with each letter will be any dissenting opinions of members of the Board.

### 6.6 REPORTABLE OCCURRENCE ACTION

#### 6.6.1

The following actions shall be taken in the event of a Reportable Occurrence:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each Reportable Occurrence Report submitted to the Commission shall be reviewed by the Plant Operations Review Committee and submitted to the ISRG Coordinator and the Vice President & Director.

### 6.7 SAFETY LIMIT VIOLATION

#### 6.7.1

The following actions shall be taken in the event a Safety Limit is violated:

- a. If any Safety Limit is exceeded, the reactor shall be shut down immediately until the Commission authorizes the resumption of operation.
- b. The Safety Limit violation shall be reported to the Commission and the Vice President & Director.

TABLE 6.5-1  
SAFETY REVIEW RESPONSIBILITIES

ITEM	INITIAL ACTION	PORC	INDEPENDENT REVIEW	
			ISRG	GORB
a) Proposed change to equipment, or systems subject to the Provisions of Section 50.59, Part 50, Title 10 Code of Federal Regulations.	Initiator: Must prepare a complete description of the proposed changes and ensure a safety evaluation of the change is included. Vice President & Director (1) Must determine if the item is an actual change to equipment or systems as described in the FSAR. (2) Must determine if the item involves an unreviewed safety question. (3) May request the PORC to assist in the above determinations.	Must review items to determine whether or not an unreviewed safety question is involved, if requested by the Vice President & Director.	Must review all determinations by the Vice President & Director.	May review any determination, but must review those for which the Vice President & Director has requested GORB review.
b) Proposed tests and experiments, (subject to provisions of) 50.59, Part 50, Title 10, Code of Federal Regulations.	Initiator: Must prepare a complete description of the proposed test or experiment and ensure a safety evaluation of the test or experiment is included. Vice President & Director (1) Must determine if the item involves an unreviewed safety question. (2) May request the PORC to assist in the above determinations.	Must review item to determine whether or not an unreviewed safety question is involved, if requested by the Vice President & Director.	As above.	As above.
c) Proposed changes in Technical Specifications or in the NRC Operating License.	Initiator: Must prepare a complete description of the proposed change and ensure a safety evaluation of the change is included.	(1) Must review the item for nuclear and radiological safety. (2) Must make recommendations to the Vice President & Director as to whether or not the change is safe.	Must review change and PORC recommendation prior to submittal to the NRC.	May review any item but must review those for which the Vice President & Director or his supervisor have requested GORB review.
d) Reportable occurrences	Vice President & Director: Must have investigations performed for all Reportable Occurrences and a report prepared including the safety significance of the incident.	Must review Reportable Occurrences Report for safety significance and make recommendations to the Vice President & Director on how to avoid recurrence.	Must review Reportable Occurrence Reports for safety significance and review PORC recommendations.	As above.

TABLE 6.5-1 (CON'T)

## SAFETY REVIEW RESPONSIBILITIES

ITEM	INITIAL ACTION	PORC	INDEPENDENT REVIEW	
			ISRG	GORB
e) Facility operations including Security Plan, Emergency Plan and Implementing procedures; review is to detect potential safety hazards.		Continuing responsibility	See Item i below	As above
f) Significance operation abnormalities or deviations from normal and expected performance.	Vice President & Director Report such matters to the PORC ISRG Coordinator and the Chairman GORB.	Review matter and report evaluation of safety significance to the ISRG and GORB.	Perform independent review of PORC evaluation.	As above
g) Any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems or components.	As above	As above	As above	As above
h) PORC minutes and reports.			Review to determine if any matters discussed involve unrelated safety questions.	As above
i) Audit Reports and NRC Inspection Reports.			Review to determine if any matters reported involve Violations of Technical Specifications, license requirements or regulations or have any nuclear or radiation safety implications.	The report of the management review of the QA Plan, initiated by the Vice President, and Director Oyster Creek in accordance with the Operational Quality Assurance Plan, shall be reviewed by the GORB with respect to technical and administrative safety issues.

c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the Plant Operations Review Committee and submitted to the Vice President & Director. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components systems or structures, and (3) corrective action taken to prevent recurrence.

d. The Safety Limit Violation Report shall be submitted to the Commission within 10 days of the violation. It shall also be submitted to the ISRG Coordinator.

## 6.8 PROCEDURES

### 6.8.1

Written procedures shall be established, implemented, and maintained that meet or exceed the requirements of Section 5.1 and 5.3 of American National Standard N18.7-1972 and Appendix "A" of the Nuclear Regulatory Commission's Regulatory Guide 1.33-1972 except as provided in 6.8.2 and 6.8.3 below.

### 6.8.2

Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the Plant Operations Review Committee and approved by the Vice President & Director prior to implementation and periodically as specified in the Administrative Procedures.

### 6.8.3

Temporary changes to procedures 6.8.1 above may be made provided:

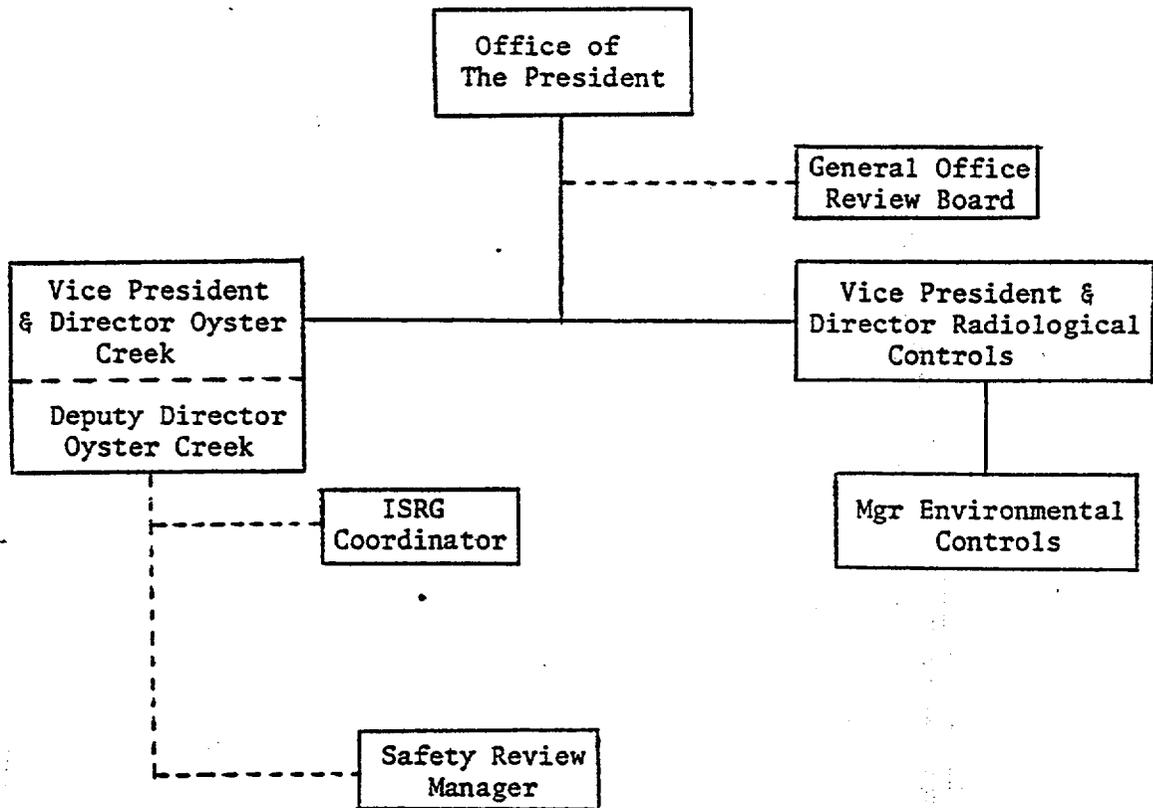
- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the supervisory staff, at least one of whom possesses a Senior Reactor Operator's License.
- c. The change is documented, subsequently reviewed by the Plant Operations Review Committee and approved by the Vice President & Director as specified in the Administrative Procedures.

## 6.9 REPORTING REQUIREMENTS

In addition to the applicable reporting requirements of 10 CFR, the following identified reports shall be submitted to the Director of the appropriate Regional Office of Inspection and Inforcement unless otherwise noted.

### 6.9.1 ROUTINE REPORTS

- a. Startup Report. A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an



Legend: \_\_\_\_\_ Direct Responsibility  
 ----- Advisory Capacity

FIGURE 5-1

ORGANIZATION FOR THE IMPLEMENTATION OF  
 THE ENVIRONMENTAL TECHNICAL SPECIFICATIONS



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

FOR OYSTER CREEK NUCLEAR GENERATING STATION

SUPPORTING AMENDMENT NO. 65 TO PROVISIONAL OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated June 15, 1982 GPU Nuclear Corporation and Jersey Central Power & Light Company (the licensees) requested an amendment to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment would authorize changes to the administrative organization at the Oyster Creek plant by, (1) retitling the position of Director Station Operations to Deputy Director Oyster Creek, (2) assigning the same Technical Specification responsibilities of the Director Station Operations to the Vice President and Director Oyster Creek, (3) retitling the position of Fire Protection Manager to Fire Protection Supervisor, and (4) retitling the position of Radiological Controls Foreman to Radiological Controls Group Supervisor.

2.0 DISCUSSION AND EVALUATION

The licensee's proposed Technical Specification change request would revise various organizational positions and responsibilities. Specifically the position of Director Station Operations would be retitled, Deputy Director Oyster Creek, and the responsibilities of that position would remain the same. Also, these responsibilities would be concurrently assigned to the Vice President and Director Oyster Creek. The licensee has defined the qualification requirements of both positions to be equivalent and identical to the existing requirements.

In addition, the position of Radiological Controls Foreman would be retitled, Radiological Controls Group Supervisor, and the position of Fire Protection Manager would be retitled, Fire Protection Supervisor. Their respective qualifications and responsibilities remain unchanged, therefore, these changes are administrative title changes only.

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P PDR

The licensee has also proposed the following reporting requirement changes. (1) personnel who perform the fire protection surveillances that presently report to the Fire Protection Supervisor will report to the Preventive Maintenance Manager, and (2) personnel who perform the plant chemistry work that now report to the Chemistry Manager will report to the Operations Department. In each case, the change is an administrative change, and the responsibilities presently assigned to the Fire Protection Supervisor and the Chemistry Manager remain the same.

In addition, the licensee proposes to add a note to Figure 6.2.2 concerning the manning of equipment operators. At the present time an onshift equipment operator is assigned on an as needed basis to be a radwaste operator. This change would require the radwaste operator to be a full-time position and would be considered part of the minimum shift staffing of equipment operators.

The guidance used by us in determining acceptability of the plant organization is contained in Regulatory Guide 1.33, Quality Assurance Program Requirements (Operation). The proposed changes are consistent with the criteria stated in the Regulatory Guide. Thus, we have concluded that all of the changes discussed above are administrative in nature, do not reduce the effectiveness for the management or safe operation of the Oyster Creek Nuclear Generating Station. Therefore, we find the proposed changes acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §1.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

### 4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction

in a margin of safety, the amendment does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (3) such activities will be conducted in compliance with the Commission's regulation and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

#### 5.0 ACKNOWLEDGEMENTS

J. Lombardo prepared this evaluation.

Date: November 5, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-219GPU NUCLEAR CORPORATION ANDJERSEY CENTRAL POWER & LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 65 to Provisional Operating License No. DPR-16, issued to GPU Nuclear Corporation and Jersey Central Power & Light Company (the licensees), which revised the Technical Specifications for operation of the Oyster Creek Nuclear Generating Station (the facility) located in Ocean County, New Jersey. The amendment is effective as of its date of issuance.

The amendment authorizes changes to the Appendices A and B Technical Specifications to reflect organizational changes.

The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated June 15, 1982, (2) Amendment No. 65 to License No. DPR-16, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C., and the Local Public Document Room, 101 Washington Street, Toms River, New Jersey 08753. A single copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 5th day of November, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Dennis M. Crutchfield, Chief  
Operating Reactors Branch #5  
Division of Licensing