



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

November 22, 1985

Docket No. 50-219  
LS05-85-11-032

Mr. P. B. Fiedler  
Vice President and Director  
Oyster Creek Nuclear Generating Station  
Post Office Box 388  
Forked River, New Jersey 08731

Dear Mr. Fiedler:

SUBJECT: NUREG-0737 TECHNICAL SPECIFICATIONS (GENERIC LETTER NO. 83-36)

Re: Oyster Creek Nuclear Generating Station

The Commission has issued the enclosed Amendment No. 94 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment is in response to your application dated February 11, 1985.

This amendment authorizes changes to the Oyster Creek Appendix A Technical Specifications (TS) to add limiting conditions for operation and surveillance requirements on the following post-accident monitoring instrumentation: drywell pressure monitors, torus water level monitors, and drywell hydrogen monitors. These monitors are NUREG-0737 Items II.F.1.4, II.F.1.5, and II.F.1.6, respectively. The TS for these monitors were requested by the staff in Generic Letter 83-36. In your application dated February 11, 1985, you also requested changes to TS on relief valve position indication and safety valve position indication. The staff will address these changes in a later safety evaluation.

In November 1980, the staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements," which included all TMI Action Plan items approved by the Commission for implementation at operating nuclear power reactors. NUREG-0737 identified items for which TS were scheduled for implementation by December 1981. The staff provided guidance on the scope of the TS which the staff would find acceptable for some of these items in Generic Letter (GL) No. 83-36, NUREG-0737 Technical Specifications. This GL was issued to all boiling water reactor (BWR) licensees on November 1, 1983.

By letters dated October 5, 1984, and February 11, 1985, you submitted the responses for Oyster Creek to GL No. 83-36. This letter covers the following TMI Action Plan items identified in the GL:

1. Reactor Coolant System Vents (II.B.1);
2. Post-Accident Sampling (II.B.3);
3. Noble Gas Effluent Monitors (II.F.1.1);
4. Sampling and Analysis of Plant Effluents (II.F.1.2);
5. Containment High-Range Radiation Monitor (II.F.1.3);
6. Containment Pressure Monitor (II.F.1.4);
7. Containment Water Level Monitor (II.F.1.5);
8. Containment Hydrogen Monitor (II.F.1.6); and
9. Control Room Habitability Requirements (III.D.3.4)

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P PDR

The summary of the resolution of these TMI Action Items listed above is given in the table in the enclosed Safety Evaluation related to this action. The staff requests that you propose appropriate TS for the following items in GL No. 83-36: II.F.1.1, II.F.1.2, II.F.1.3, II.B.1, and III.D.3.4. The TS are requested prior to plant restart from the Cycle 11 Refueling Outage. Sample TS for these items are provided in Enclosure 3 to GL 83-36.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on April 23, 1985 (50 FR 16004). No public comments or requests for hearing were received.

A notice of issuance pertaining to this action will appear in the Commission's biweekly notice publication in the Federal Register.

Sincerely,

Original signed by

John A. Zwolinski, Chief  
Operating Reactors Branch #5  
Division of Licensing

Enclosures:

- 1. Amendment No. 94 to License No. DPR-16
- 2. Safety Evaluation for Generic Letter 83-36 NUREG-0737 Technical Specifications.

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See next page

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DCrutchfield  
11/22/85

Mr. P. B. Fiedler

- 2 -

The summary of the resolution of these TMI Action Items listed above is given in the table in the enclosed Safety Evaluation related to this action. The staff requests that the licensee propose appropriate TS for the following items in GL No. 83-36: II.F.1.1, II.F.1.2, II.F.1.3, II.B.1, and III.D.3.4. The TS are requested prior to plant restart from the Cycle 11 Refueling Outage. Sample TS for these items are provided in Enclosure 3 to GL 83-36.

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Sincerely,

John A. Zwolinski, Chief  
Operating Reactors Branch #5  
Division of Licensing

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2. Safety Evaluation for Generic Letter 83-36 NUREG-0737 Technical Specifications.

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OELD  
*Chubb*  
10/31/85

DL: ORR #5  
JZwolinski  
/ /85

DL: AD/SA  
DCrutchfield  
/ /85

Mr. P. B. Fiedler  
Oyster Creek Nuclear Generating Station

Oyster Creek Nuclear  
Generating Station

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 94  
License No. DPP-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by GPU Nuclear Corporation and Jersey Central Power and Light Company (the licensees) dated February 11, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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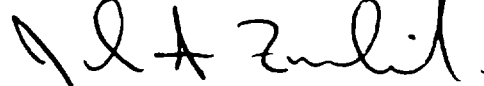
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. , are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John A. Zwolinski, Chief  
Operating Reactors Branch #5  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 22, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 94  
PROVISIONAL OPERATING LICENSE NO. DPR-16  
DOCKET NO. 50-219

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain vertical lines indicating the area of change.

<u>REMOVE</u>	<u>INSERT</u>
--	3.13-1a
3.13-2	3.13-2
3.13-3	3.13-3
4.13-1	4.13-1
4.13-2	4.13-2

**D. Wide Range Torus Water Level Monitor**

1. Two wide range torus water level monitor channels shall be continuously indicated in the control room during Power Operation.

2. With the number of operable accident monitoring channels less than the total Number of Channels shown in Table 3.13.1, restore the inoperable channel(s) to Operable status within 7 days or place the reactor in the shutdown condition within the next 24 hours.

3. With the number of operable accident monitoring instrumentation channels less than the Minimum Channels operable requirements of Table 3.13.1 restore the inoperable channel(s) to operable status within 48 hours or place the reactor in the shutdown condition within the next 24 hours.

**E. Wide Range Drywell Pressure Monitor**

1. Two Wide Range Drywell Pressure monitor channels shall be continuously indicated in the control room during Power Operation.

2. With the number of operable accident monitoring channels less than the total Number of Channels shown in Table 3.13.1, restore the inoperable channel(s) to Operable status within 7 days or place the reactor in the shutdown condition within the next 24 hours.

3. With the number of operable accident monitoring instrumentation channels less than the Minimum Channels operable requirements of 3.13.1 restore the inoperable channel(s) to operable status within 48 hours or place the reactor in the shutdown condition within the next 24 hours.

**F. Drywell H<sub>2</sub> Monitor**

1. Two drywell hydrogen monitor channels shall be capable of continuously indicating in the control room during power operation.

2. With the number of operable channels less than the total number of channels shown in Table 3.13.1, restore the inoperable channel to operable status within 30 days or place the reactor in the shutdown condition within the next 24 hours.

3. With the number of operable channels less than the minimum channels operable requirements of Table 3.13.1, restore at least one channel to operable status within 7 days or place the reactor in the shutdown condition within the next 24 hours.



## BASES

The purpose of the safety/relief valve accident monitoring instrumentation is to alert the operator to a stuck open safety/relief valve which could result in an inventory threatening event.

As the safety valves present distinctly different concerns than those related to relief valves, the technical specifications are separated as to the actions taken upon inoperability. Clearly, the actuation of a safety valve will be immediately detectable by observed increase in drywell pressure. Further confirmation can be gained by observing reactor pressure and water level. Operator action in response to these symptoms would be taken regardless of the acoustic monitoring system status. Acoustic monitors act only to confirm the reseating of the safety valve. In actuality, the operator actions in response to the lifting of a safety valve will not change whether or not the safety valve reseats. Therefore, the actions taken for inoperable acoustic monitors on safety valves are significantly less stringent than that taken for those monitors associated with relief valves.

Should an acoustic monitor on a safety valve become inoperable, setpoints on adjacent monitors will be reduced to assure alarm actuation should the safety valve lift, since it is of no importance to the operator as to which valves lift but only that one has lifted. Analyses, using very conservative blowdown forces and attenuation factors, show that reducing the alarm setpoint on adjacent monitors to  $<1.4g$  will assure alarm actuation should the adjacent safety valve lift. Minimum blowdown force considered was  $30g$  with a maximum attenuation of  $27dB$ . In actuality, a safety valve lift would result in considerably larger blowdown force. The maximum attenuation of  $27 dB$  was determined based on actual testing of a similar monitoring system installed in a similar configuration.

The operability of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with NUREGs 0578 and 0737.

TABLE 3.13.1

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Relief Valve Position Indicator (Primary Detector*)	1/valve	1/valve
Relief Valve Position Indicator (Backup Indications**)	1/valve	
* Acoustic Monitor		
** Thermocouple		
Thermocouple TE 65A can be substituted for thermocouple TE210-43V, W, or X		
Thermocouple TE 65B can be substituted for thermocouple TE210-43Y or Z		
2. Wide Range Drywell Pressure Monitor (PT/PR-53 & 54)	2	1
3. Wide Range Torus Water Level Monitor (LT/LR-37 & 38)	2	1
4. Drywell H <sub>2</sub> Monitor	2	1

**4.13 ACCIDENT MONITORING INSTRUMENTATION**

- Applicability:** Applies to surveillance requirements for the accident monitoring instrumentation
- Objective:** To verify the operability of the accident monitoring instrumentation.
- Specification:**
- A. Safety & Relief Valve Position Indicators**  
Each accident monitoring instrumentation channel shall be demonstrated operable by performance of the Channel Check and Channel Calibration operations at the frequencies shown in Table 4.13.1.
  - B. Wide Range Drywell Pressure Monitor**  
Each accident monitoring instrumentation channel shall be demonstrated operable by performance of the Channel Check and Channel Calibration operations at the frequencies shown in Table 4.13.1.
  - C. Wide Range Torus Water Level Monitor**  
Each accident monitoring instrumentation channel shall be demonstrated operable by performance of the Channel Check and Channel Calibration operations at the frequencies shown in Table 4.13.1.
  - D. Drywell H<sub>2</sub> Monitor**  
Each accident monitoring instrumentation channel shall be demonstrated operable by performance of the Channel Check and Channel Calibration operations at the frequencies shown in Table 4.13.1.

**Bases:**

The operability of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with NUREGs 0578 and 0737.

TABLE 4.13-1

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Primary and Safety Valve Position Indicator (Primary Detector*)	A	B
Relief and Safety Valve Position Indicator (Backup Indications**)	A	B
Relief Valve Position Indicator (Common Header Temperature Element**)	C	B***
2. Wide Range Drywell Pressure Monitor (PT/PR 53 & 54)	A	D
3. Wide Range Torus Water Level Monitor (LT/LR 37 & 38)	A	D
4. Drywell H <sub>2</sub> Monitor	A <sup>1</sup>	E

## Legend:

A = at least once per 31 days; R = at least once per 18 months (550 days).

C = at least once per 15 days until channel calibration is performed and thence at least once per 31 days.

D = at least once per 6 months; E = at least once per 12 months; 1 = Span and Zero using calibration gases.

\* Acoustic Monitor

\*\* Thermocouple

\*\*\* This surveillance will commence at the first cold shutdown after July 1, 1985.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

GENERIC LETTER 83-36, NUREG-0737 TECHNICAL SPECIFICATIONS

GPU NUCLEAR CORPORATION AND

JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

1.0 INTRODUCTION

In November 1980, the staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements," which included all TMI Action Plan items approved by the Commission for implementation at operating nuclear power reactors. NUREG-0737 identified items for which Technical Specifications (TSs) were scheduled for implementation by December 1981. The staff provided guidance on the scope of the TSs which the staff would find acceptable for some of these items in Generic Letter (GL) No. 83-36, NUREG-0737 Technical Specifications. This GL was issued to all boiling water reactor (BWR) licensees on November 1, 1983. In this GL, the staff requested the licensees to:

1. Review their facility's TSs to determine if they are consistent with the guidance provided in the GL, and
2. Submit an application for a license amendment where deviations or absence of TSs were found.

By letters dated October 5, 1984, and February 11, 1985, GPU Nuclear (the licensee) submitted the responses for Oyster Creek to GL 83-36. This evaluation covers the following TMI Action Plan items identified in the GL:

1. Reactor Coolant System Vents (II.B.1);
2. Post-Accident Sampling (II.B.3);
3. Noble Gas Effluent Monitors (II.F.1.1);
4. Sampling and Analysis of Plant Effluents (II.F.1.2);
5. Containment High-Range Radiation Monitor (II.F.1.3);
6. Containment Pressure Monitor (II.F.1.4);
7. Containment Water Level Monitor (II.F.1.5);
8. Containment Hydrogen Monitor (II.F.1.6); and
9. Control Room Habitability Requirements (III.D.3.4)

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The summary of the resolution of these TMI Action Items listed above is given in the enclosed Table.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on April 23, 1985 (50 FR 16004). No public comments or requests for hearing were received.

## 2.0 EVALUATION

In the licensee's submittal dated October 5, 1984, only six of the above items were discussed. These items were the following:

2. Post-Accident Sampling (II.B.3);
3. Noble Gas Effluent Monitors (II.F.1.1);
4. Sampling and Analysis of Plant Effluents (II.F.1.2);
6. Containment Pressure Monitor (II.F.1.4);
7. Containment Water Level Monitor (II.F.1.5); and
8. Containment Hydrogen Monitor (II.F.1.6).

### Item II.B.3

For Post-Accident Sampling, Item II.B.3, the licensee stated in its letter dated October 5, 1984, that it would include the appropriate requirements in a revision to Section 6.0, Administrative Controls, of the Oyster Creek Appendix A Technical Specifications (TSS). By letter dated June 19, 1985, the licensee proposed TSS on post-accident sampling. The review of the licensee's proposed TSS for this item is being handled as a separate plant-specific action.

### Item II.F.1.1

For Noble Gas Effluent Monitors, Item II.F.1.1, the licensee stated in its letter dated October 5, 1984, that the installed system of high range noble gas effluent monitors was undergoing evaluation and should changes be warranted they would be submitted to the staff by June 1985. This response does not address the staff's request in GL 83-36 for the licensee to review its facility's TSS and to submit an application for a license amendment where absence of TSS were found.

The licensee is requested to propose TSS on the high range noble gas effluent monitors before the restart from the Cycle 11 Refueling (Cycle 11R) outage. An acceptable specification on these monitors would be the model TSS in GL 83-36.

Item II.F.1.2

For Sampling and Analysis of Plant Effluents, Item II.F.1.2, the staff by letter dated May 8, 1981 authorized Amendment 54 to the Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment authorized TS 6.16, Iodine Monitoring, "a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions." This amendment approved changes to the TSs to incorporate TMI-2 Lessons Learned Category "A" requirements; however, it does not meet the requirements of Item II.F.1.2 in which each operating nuclear power plant shall have the capability to collect and analyze or measure representative samples of "radioactive iodines and particulates in gaseous effluents during and following an accident." The licensee stated in its letter dated October 5, 1984 that the system for this item was installed. The licensee is requested to propose changes to the administrative section of the TSs to meet the requirements of this item before the restart from the Cycle 11R outage. Model TSs for this item can be found in Enclosure 3 to GL 83-36.

Item II.F.1.4

For Containment Pressure Monitor, Item II.F.1.4, the licensee proposed TSs by letter dated February 11, 1985, to incorporate the requirements identified in the model TSs in GL 83-36 for this item. The proposed TSs are limiting conditions for operation (Section 3.13 of the TSs) and surveillance requirements (Section 4.13 of the TSs) on the wide range drywell pressure monitors. These are the Containment Pressure Monitors for Oyster Creek.

The proposed TSs have been reviewed against the model TSs in GL 83-36. The proposed TSs for the wide range drywell pressure monitors are consistent with the model TSs for Containment Pressure Monitors in GL 83-36 and with the Oyster Creek TSs except for the following: the licensee proposed that the two wide range drywell pressure monitor channels should be continuously indicated in the control room during Power Operation in proposed TS 3.13.E.1. The "should" in the previous sentence is not considered by the staff to be a legally enforceable statement of what the licensee is required to do to be in compliance with the plant TSs. In the August Progress Review Meeting On Licensing Actions of September 18, 1985 documented in a staff meeting summary (dated October 29, 1985), the licensee agreed to having the word "shall" replace the word "should" in proposed TS 3.13.E.1. A reference is made to the proposed TSs being consistent to the Oyster Creek TSs because (1) the Oyster Creek

TSs do not include the operational conditions given in the model TSs in GL 83-36 and (2) the proposed times in the action statements if the requirements in the proposed limiting conditions for operation cannot be met are the same as those for the accident monitoring instrumentation in Section 3.13 of the Oyster Creek TSs. The proposed TSs are written in terms of the operational conditions listed in Section 1, Definitions, of the Oyster Creek TSs.

Because the licensee's proposed TSs for the wide range drywell pressure monitors are consistent with the model TSs for Containment Pressure Monitors in GL 83-36 and consistent with the Oyster Creek TSs, the staff concludes that the proposed TS are acceptable.

This completes the staff's action on this item as part of GL 83-36.

#### Item II.F.1.5

For Containment Water Level Monitor, Item II.F.1.5, the licensee proposed TSs by letter dated February 11, 1985, to incorporate the requirements identified in the model TSs in GL 83-36 for this item. The proposed TSs are limiting conditions for operation (Section 3.13 of the TSs) and surveillance requirements (Section 4.13 of the TSs) on the wide range torus water level monitors. These are the Containment Water Level Monitors for Oyster Creek.

The proposed TSs have been reviewed against the model TSs in GL 83-36. The proposed TSs for the wide range torus water level monitors are consistent with the model TSs for Containment Water Level Monitors in GL 83-36 and with the Oyster Creek TSs except for the following: the licensee proposed that two wide range torus water level monitor channels should be continuously indicated in the control room during power operation in proposed TS 3.13.D.1. The "should" in the previous sentence is not considered by the staff to be a legally enforceable statement of what the licensee is required to do to be in compliance with the plant TSs. In the August Progress Review Meeting On Licensing Actions of September 18, 1985, noted in staff's meeting summary dated October 29, 1985, the licensee agreed to having the word "shall" replace the word "should" in proposed TS 3.13.D.1. A reference is made to the proposed TSs being consistent to the Oyster Creek TSs because (1) the Oyster Creek TSs do not include the operational conditions given in the model TSs in GL 83-36 and (2) the proposed times in the action statements if the requirements in the proposed limiting conditions for operation cannot be met are the same as those for the accident monitoring instrumentation in Section 3.13 of the Oyster Creek TSs. The proposed TSs are written in terms of the operational conditions listed in Section 1, Definitions, of the Oyster Creek TSs.

Because the licensee's proposed TSs for the wide range torus water level monitors are consistent with the model TSs for Containment Water Level Monitors in GL 83-36 and consistent with the Oyster Creek TSs, the staff concludes that the proposed TSs are acceptable.

This completes the staff's action on this item as part of GL 83-36.



Item II.F.1.6

For Containment Hydrogen Monitor, Item II.F.1.6, the licensee proposed TSs by letter dated February 11, 1985, to incorporate the requirements identified in the model TSs in GL 83-36 for this item. The proposed TSs are limiting conditions for operation (Section 3.13 of the TSs) and surveillance requirements (Section 4.13 of the TSs) on the drywell hydrogen monitors. These are the Containment Hydrogen Monitors for Oyster Creek.

The proposed TSs have been reviewed against the model TSs in GL 83-36. The proposed TSs for the drywell hydrogen monitors are consistent with the model TSs for Containment Hydrogen Monitors in GL 83-36 and with the Oyster Creek TSs except for the following: the licensee proposed that two drywell hydrogen monitor channels should be continuously indicated in the control room during power operation in proposed TS 3.13.F.1. The "should" in the previous sentence is not considered by the staff to be a legally enforceable statement of what the licensee is required to do to be in compliance with the plant TSs. In the August Progress Review Meeting On Licensing Actions of September 18, 1985, as noted in staff's summary October 29, 1985, the licensee agreed to having the word "shall" replace the word "should" in proposed TS 3.13.F.1. A reference is made to the proposed TSs being consistent to the Oyster Creek TSs because (1) the Oyster Creek TSs do not include the operational conditions given in the model TSs in GL 83-36 and (2) the proposed times in the action statements if the requirements in the proposed limiting conditions for operation cannot be met are the same as those for the accident monitoring instrumentation in Section 3.13 of the Oyster Creek TSs. The proposed TSs are written in terms of the operational conditions listed in Section 1, Definitions, of the Oyster Creek TSs.

Because the licensee's proposed TSs for the drywell hydrogen monitors are consistent with the model TSs for Containment Hydrogen Monitors in GL 83-36 and consistent with the Oyster Creek TSs, the staff concludes that the proposed TSs are acceptable.

This completes the staff's action on this item as part of GL 83-36.

The licensee's submittal of October 5, 1984, did not discuss three of the nine items in GL 83-36. These items are discussed below:

Item II.B.1

For Reactor Coolant System Vents, Item II.B.1, the staff in its letter dated August 9, 1984, granted an exemption to the scheduler requirements for installation of isolation condenser high point vents in 10 CFR 50.44(c)(3)(iii). The exemption is effective until the startup from the Cycle 11 Refueling Outage. The installation of the high point vents shall be complete for the plant to start up from this outage.

By letter dated July 23, 1985, the licensee requested an exemption to 10 CFR 50.44(c)(3)(iii) and the installation of Isolation Condenser High Point Vents at Oyster Creek. The requested exemption is under review by the staff. If the exemption is granted, the Reactor Coolant System Vents will not have to be installed at Oyster Creek and the TSs on these vents would not be required.

The licensee is requested to submit the TSs changes for this item before the restart from the Cycle 11 Refueling Outage if the staff does not grant the requested exemption. Appropriate TSs for this item can be found in Enclosure 3 to GL 83-36.

#### Item II.F.1.3

For Containment High-Range Radiation Monitors, Item II.F.1.3, the licensee will install these monitors during the Cycle 11 Refueling Outage which is scheduled to begin in April 1986. The staff accepted this in its letter to the licensee dated October 20, 1982. Requirements on this equipment do not exist in the TSs. The licensee is requested to propose TSs for this item before the restart from the Cycle 11 Refueling Outage. An acceptable specification on these monitors would be the model TSs in GL 83-36.

#### Item III.D.3.4

For Control Room Habitability Requirements, Item III.D.3.4, the licensee submitted a letter dated June 4, 1985, to document the commitments of the licensee to implement the control room habitability requirements by the restart from the Cycle 12 Refueling Outage. The licensee committed to submit the appropriate TSs for this item before the restart from the Cycle 11 Refueling Outage. This is acceptable to the staff.

An acceptable specification on control room habitability would be the model TS in GL 83-36.

### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment on the drywell pressure monitors, the torus water level monitors and the drywell hydrogen monitors involves a change to a requirement with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no

significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### 4.0 CONCLUSION

A summary of the resolution of NUREG-0737 TSs in GL 83-36 is given in the enclosed Table.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

#### 5.0 ACKNOWLEDGEMENT

This evaluation was prepared by J. Donohew.

Dated: November 22, 1985

RESOLUTION OF NUREG-0737

TECHNICAL SPECIFICATIONS (TSs) IN GENERIC LETTER 83-36\*

<u>NUREG-0737 Item</u>	<u>Title</u>	<u>Resolution of NUREG-0737 TSs</u>
II.B.1	Reactor Coolant System Vents	Staff requests TSs before restart from Cycle 11R* outage.
II.B.3	Post-Accident Sampling	Staff reviewing licensee's TSs.
II.F.1.1	Noble Gas Effluent Monitors	Staff requests TSs before restart from Cycle 11R outage.
II.F.1.2	Sampling and Analysis of Plant Effluents	Staff requests TSs before restart from Cycle 11R outage.
II.F.1.3	Containment High-Range Radiation Monitor	Staff requests TSs before restart from Cycle 11R outage.
II.F.1.4	Containment Pressure Monitor	Staff concludes that the licensee's proposed TSs are acceptable in this letter.
II.F.1.5	Containment Water Level Monitor	Staff concludes that the licensee's proposed TSs are acceptable in this letter.
II.F.1.6	Containment Hydrogen Monitor	Staff concludes that the licensee's proposed TSs are acceptable in this letter.
III.D.3.4	Control Room Habitability Requirements	Staff accepts licensee's commitment to submit TSs before restart from Cycle 11R outage.

\*Cycle 11R = Cycle 11 Refueling Outage