



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

December 9, 1985

Docket No. 50-219

Mr. P. B. Fiedler  
Vice President and Director  
Oyster Creek Nuclear Generating Station  
Post Office Box 388  
Forked River, New Jersey 08731

Dear Mr. Fiedler:

SUBJECT: VALVE POSITION INDICATOR ACCIDENT MONITORING  
INSTRUMENTATION

Re: Oyster Creek Nuclear Generating Station

The Commission has issued the enclosed Amendment No. 96 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment is in response to your application dated February 11, 1985.

This amendment authorizes changes to the Appendix A Technical Specifications (TS) pertaining to Section 3.13, Accident Monitoring Instrumentation. The limiting conditions for operation for the Relief Valve and Safety Valve Position Indicators are being revised to conform with guidance contained in Generic Letter 83-36, NUREG-0737 Technical Specifications, dated November 1, 1983.

In the application dated February 11, 1985, you also requested the following changes to Sections 3.13 and 3.14 of the TS: 1) clarification of TS Table 3.13.1 on Relief Valve Position Indicators; 2) addition of limiting conditions for operation and surveillance requirements on the Torus Water Level Monitors, the Drywell Pressure Monitors and the Drywell Hydrogen Monitors; and 3) Revision of TS 3.13.A.1 for Relief Valve Position Indicators. The requested changes 1) and 3) above were withdrawn by the licensee in the August Progress Review Meeting on Licensing Actions of September 18, 1985. The meeting minutes for this meeting are dated October 29, 1985. The requested change 2) above was the subject of a separate staff action on Generic Letter 83-36 dated November 22, 1985.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on April 23, 1985 (50 FR 16004). No public comments or requests for hearing were received.

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Mr. P. B. Fiedler

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A copy of our related Safety Evaluation is also enclosed. A notice of issuance pertaining to this action will appear in the Commission's biweekly notice publication in the Federal Register.

Sincerely,

~~Original signed by~~

John A. Zwolinski, Director  
BWR Project Directorate #1  
Division of BWR Licensing

Enclosures:

1. Amendment No. 96 to License No. DPR-16
2. Safety Evaluation

cc w/enclosures:  
See next page

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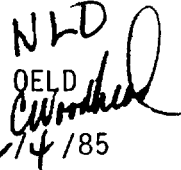
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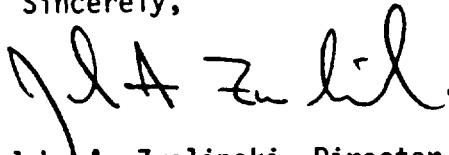
Mr. P. B. Fiedler

- 2 -

December 9, 1985

A copy of our related Safety Evaluation is also enclosed. A notice of issuance pertaining to this action will appear in the Commission's biweekly notice publication in the Federal Register.

Sincerely,

A handwritten signature in cursive script, appearing to read "John A. Zwolinski".

John A. Zwolinski, Director  
BWR Project Directorate #1  
Division of BWR Licensing

Enclosures:

1. Amendment No. 96 to  
License No. DPR-16
2. Safety Evaluation

cc w/enclosures:  
See next page

Mr. P. B. Fiedler  
Oyster Creek Nuclear Generating Station

Oyster Creek Nuclear  
Generating Station

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION  
AND  
JERSEY CENTRAL POWER & LIGHT COMPANY  
DOCKET NO. 50-219  
OYSTER CREEK NUCLEAR GENERATING STATION  
AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 96  
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by GPU Nuclear Corporation and Jersey Central Power and Light Company (the licensees) dated February 11, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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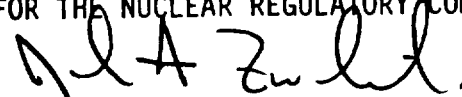
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 96, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John A. Zwolinski, Director  
BWR Project Directorate #1  
Division of BWR Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 9, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 96

PROVISIONAL OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain vertical lines indicating the area of change.

REMOVE

3.13-1

INSERT

3.13-1

**3.13 ACCIDENT MONITORING INSTRUMENTATION**

**Applicability:** Applies to the operating status of accident monitoring instrumentation.

**Objective:** To assure operability of accident monitoring instrumentation.

**Specification:** A. **Relief Valve Position Indicators**

1. The accident monitoring instrumentation channels shown in Table 3.13.1 shall be operable when the mode switch is in the Startup or Run positions.
2. With the number of operable accident monitoring instrumentation channels less than the Total Number of Channels shown in Table 3.13.1, either restore the inoperable channels to operable status within 7 days, or place the reactor in the shutdown condition within the next 24 hours.
3. With the number of operable accident monitoring instrumentation channels less than the Minimum Channels Operable requirements of Table 3.13.1, either restore the inoperable channel(s) to the operable status within 48 hours, or place the reactor in the shutdown condition within the next 24 hours.

B. **Safety Valve Position Indicators**

1. During power operation, both primary\* and backup\*\* safety valve monitoring instruments are required to be operable except as provided in 3.13.B.2 and 3.13.B.3.
  2. If either the primary\* or backup\*\* accident monitoring instruments on a valve become inoperable, the primary\* accident monitoring instrument on an adjacent valve must be operable, and its set point appropriately reduced.
  3. If both the primary\* and backup\*\* accident monitoring instruments on a valve become inoperable and the primary\* accident monitoring instrument on an adjacent valve is operable, either restore the inoperable channel(s) to an operable status within 7 days, or place the reactor in the shutdown condition within the next 24 hours.
  4. If the requirements of Section 3.13.B.2 or 3.13.B.3 cannot be met within 48 hours, place the reactor in the shutdown condition within the next 24 hours.
- C. In the event that any of these monitoring channels become inoperable, they shall be made operable prior to startup following the next cold shutdown.

\* Acoustic Monitor  
\*\* Thermocouple

Amendment No.: ~~54~~, ~~51~~, 96





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 96 TO PROVISIONAL OPERATING LICENSE NO. DPR-16  
GPU NUCLEAR CORPORATION AND  
JERSEY CENTRAL POWER & LIGHT COMPANY  
OYSTER CREEK NUCLEAR GENERATING STATION  
DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated February 11, 1985, GPU Nuclear (the licensee) requested an amendment to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station (OCNGS).

This amendment authorizes changes to the Appendix A Technical Specifications (TS) pertaining to Section 3.13, Accident Monitoring Instrumentation. The limiting conditions for operation for the Relief Valve and Safety Valve Position Indicators are being revised to conform to the guidance contained in Generic Letter 83-36, NUREG-0737 Technical Specifications, dated November 1, 1983.

In the application dated February 11, 1985, GPU Nuclear (the licensee) also requested the following changes to Sections 3.13 and 3.14 of the TS:

- 1) clarification of TS Table 3.13.1 on Relief Valve Position Indicators;
- 2) addition of limiting conditions for operation and surveillance requirements on the Torus Water Level Monitors, the Drywell Pressure Monitors and the Drywell Hydrogen Monitors; and
- 3) Revision of TS 3.13.A.1 for Relief Valve Position Indicators.

The requested changes 1) and 3) above were withdrawn by the licensee in the August Progress Review Meeting on Licensing Actions of September 18, 1985. The meeting minutes for this meeting are dated October 29, 1985. The requested change 2) above was the subject of a separate staff action on Generic Letter 83-36 dated November 22, 1985.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on April 23, 1985 (50 FR 16004). No public comments or requests for hearing were received.

2.0 DISCUSSION

In November 1980, the staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements," which included all TMI Action Plan items approved by the Commission for implementation at operating nuclear power reactors. NUREG-0737 identified items for which TS were scheduled for implementation by

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December 1981. The staff provided guidance on the scope of the TS which the staff would find acceptable for some of these items in Generic Letter (GL) No. 83-36, NUREG-0737 Technical Specifications. This GL was issued to all Boiling Water Reactor (BWR) licensees on November 1, 1983. In this GL, the staff requested the licensees to:

1. Review their facility's TS to determine if they are consistent with the guidance provided in the GL, and
2. Submit an application for a license amendment where deviations or absence of TSs were found.

This GL addressed the following NUREG-0737, TMI Action Plan, Items:

1. Reactor Coolant System Vents (II.B.1);
2. Post-Accident Sampling (II.B.3);
3. Noble Gas Effluent Monitors (II.F.1.1);
4. Sampling and Analysis of Plant Effluents (II.F.1.2);
5. Containment High-Range Radiation Monitor (II.F.1.3);
6. Containment Pressure Monitor (II.F.1.4);
7. Containment Water Level Monitor (II.F.1.5);
8. Containment Hydrogen Monitor (II.F.1.6);
9. Control Room Habitability Requirements (III.D.3.4).

By this present application dated February 11, 1985, the licensee submitted TS for TMI Action Plan Items II.F.1.4, II.F.1.5 and II.F.1.6: Drywell Pressure Monitors, Torus Water Level Monitors and Drywell Hydrogen Monitors, respectively. The staff's review and evaluation of these proposed TS are part of a separate action on GL 83-36.

In the February 11, 1985 application the licensee also requested that TS 3.13.A.1 on Relief Valve Position Indicators be revised to state that the channels "should" be operable in place of the current statement that the channels "shall" be operable. It is the staff's position that the proposed "should" is not a legally enforceable statement of what the licensee is required to do to be in compliance with the TS. This has been discussed with the licensee in the August Progress Review Meeting on Licensing Actions of September 18, 1985, and the licensee agreed to withdraw this proposed change.

In the August Progress Review Meeting, the licensee also agreed that Amendment 88 on Relief Valve Position Indication of July 1, 1985, superseded its requested change to TS Table 3.13.1. The licensee agreed to withdraw its requested change to TS Table 3.13.1 in the present application.

### 3.0 EVALUATION

In its February 11, 1985 application the licensee proposed to revise TS 3.13.A.2 and 3.13.A.3 on Relief Valve Position Indicators and TS 3.13.B.3 and 3.13.B.4 on Safety Valve Position Indicators. These changes were to revise the action to be taken if the limiting conditions for operation on the Relief Valve and Safety Valve Position Indicators in these TS were not met. The proposed action would be (1) to place the reactor in the shutdown condition instead of in the cold shutdown condition and (2) within the next 24 hours

instead of within 24 hours. The shutdown condition and cold shutdown condition are defined in the TS and are definitions 1.6 and 1.7, respectively, on TS page 1.0-1.

The reactor is in a shutdown condition when the reactor mode switch is in the shutdown mode position and there is fuel in the reactor vessel. In this condition, the reactor is subcritical, a control rod block is initiated, all operable control rods are fully inserted, and Specification 3.2-A is met. The reactor is at cold shutdown when the mode switch is in the shutdown mode position, there is fuel in the reactor vessel, all operable control rods are fully inserted, and the reactor coolant system is maintained at less than 212°F and vented.

The GL 83-36 provides acceptable TS for Safety and Relief Valve Position Indicators. These acceptable TS are an enclosure to the GL and provide acceptable action statements if the limiting conditions for operation are not met. These TS have been incorporated in the General Electric Boiling Water Reactor Standard Technical Specifications (BWR STS) NUREG-0123, Revision 4.

The TS proposed by the licensee are consistent with the TS in GL 83-36 and NUREG-0123. The "Hot Shutdown" in GL 83-36 and NUREG-0123 is the "shutdown condition" in the licensee's present application. The "within 24 hours" proposed by the licensee is consistent with that in GL 83-36 and NUREG-0123 and the same as that for similar action statements in the TS. Therefore, the staff concludes that the proposed changes to TS 3.13.A.2, 3.13.A.3, 3.13.B.3 and 3.13.B.4 are acceptable.

#### 4.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### 5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner;

and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

6.0 ACKNOWLEDGEMENT

This evaluation was prepared by J. Donohew.

Dated: December 9, 1985