

January 14, 1986

Docket No. 50-219

Mr. P. B. Fiedler
Vice President and Director
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

Dear Mr. Fiedler:

SUBJECT: POST ACCIDENT SAMPLING PROGRAM

Re: Oyster Creek Nuclear Generating Station

The Commission has issued the enclosed Amendment No. 98 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment is in response to your application dated June 19, 1985.

This amendment authorizes changes to the Appendix A Technical Specifications (TS) which are new requirements pertaining to the Post Accident Sampling System. These changes are to Section 6, Administrative Controls.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on August 28, 1985 (50 FR 34941). No public comments or requests for hearing were received.

A copy of our related Safety Evaluation is also enclosed. A notice of issuance pertaining to this action will appear in the Commission's biweekly notice publication in the Federal Register.

Sincerely,
Original signed by:

John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

Enclosures:

- 1. Amendment No. 98 to License No. DPR-16
- 2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. P. B. Fiedler
Oyster Creek Nuclear Generating Station

Oyster Creek Nuclear
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 98
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear Corporation and Jersey Central Power and Light Company (the licensees) dated June 19, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 98, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 14, 1986

ATTACHMENT TO LICENSE AMENDMENT NO.98

PROVISIONAL OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain vertical lines indicating the area of change.

REMOVE

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*Issued by NRC Order dated 10/24/80

6.17 The following program shall be established, implemented, and maintained.

Post Accident Sampling

A program has been established which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions.

The program shall include the following:

1. Training of personnel in sampling and analysis
2. Procedures for sampling and analysis
3. Provisions for verifying operability of the System.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 98 TO PROVISIONAL OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND
JERSEY CENTRAL POWER & LIGHT COMPANY
OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated June 19, 1985, GPU Nuclear (the licensee) requested an amendment to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station (OCNGS). This amendment would authorize changes to the Appendix A Technical Specifications (TS) which are new requirements pertaining to the Post Accident Sampling System. These changes are to Section 6, Administrative Controls.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on August 28, 1985 (50 FR 34941). No public comments or requests for hearing were received.

2.0 DISCUSSION AND EVALUATION

In November 1980, the staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements", which included all TMI Action Plan items approved by the Commission for implementation at operating nuclear power reactors. NUREG-0737 identified items for which TS were scheduled for implementation by December 1981. The staff provided guidance on the scope of the TS which the staff would find acceptable for some of these items in Generic Letter (GL) No. 83-36, NUREG-0737 Technical Specifications. This GL was issued to all boiling water reactor (BWR) licensees on November 1, 1983. In this GL, the staff requested the licensees to:

1. Review their facility's TS to determine if they are consistent with the guidance provided in the GL, and
2. Submit an application for a license amendment where deviations or absence of TS were found.

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One of the items identified in GL 83-36 was the Post-Accident Sampling System (PASS), NUREG-0737 Item II.B.3. The guidance in GL 83-36 stated that an administrative program should be established, implemented and maintained to ensure the plant has the capability to obtain and analyze reactor coolant and containment atmosphere samples under accident conditions. The program should include:

- a) training of personnel
- b) procedures for sampling and analysis, and
- c) provisions for maintenance of sampling and analysis equipment.

By letter dated June 19, 1985, the licensee proposed changes to the TS pertaining to the PASS. These requested changes are to Section 6, Administrative Controls. The proposed changes are to incorporate the guidance given in GL 83-36 into the TS.

The proposed TS are consistent with the guidance on the PASS given in GL 83-36. Therefore, the staff concludes that the proposed TS are acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

5.0 ACKNOWLEDGEMENT

This evaluation was prepared by J. Donohew.

Dated: January 14, 1986.