

11/24/77

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Docket No. 50-219

Jersey Central Power & Light Company
ATTN: Mr. I. R. Finrock, Jr.
Vice President - Generation
Madison Avenue at Punch Bowl Road
Morristown, New Jersey 07960

Gentlemen:

The Commission has issued the enclosed Amendment No. 25 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment consists of changes to the Technical Specifications and is in response to your application dated June 24, 1977. This amendment revises Section 6.8, "Procedures" of the Technical Specifications in response to a request made by the NRC in a letter to the licensee dated October 22, 1974. In that letter, we requested that the Technical Specifications for Administrative Controls be revised to be consistent with the format and content of the Standard Technical Specifications developed by the Regulatory staff.

Your June 24, 1977 submittal also requested editorial corrections in Specifications 3.4, 3.5, 6.6, 6.7 and 6.8. In addition to your identification of these needed editorial changes, the NRC staff found errors in the June 24, 1977 submittal at Specifications 3.4.C, 3.4.C.3, 3.4.C.4, 3.5.A.7, 3.5.A.8.a, 3.5.A.8.d and 6.8.1. These latter errors were discussed with you, corrected, and are included in this amendment.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

1. Amendment No. 25 to License No. DPR-16
2. Safety Evaluation
3. Federal Register Notice

cc w/encs:

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OFFICE >						
SURNAME >						
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Docket No. 50-219

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Jersey Central Power & Light Company
 ATTN: Mr. I. R. Finfrock, Jr.
 Vice President - Generation
 Madison Avenue at Punch Bowl Road
 Morristown, New Jersey 07960

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Sincerely,

George Lear, Chief
 Operating Reactors Branch #3
 Division of Operating Reactors

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1. Amendment No. to License No. DPR-16
2. Safety Evaluation
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cc w/encs:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 25
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Jersey Central Power and Light Company (the licensee) dated June 24, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.25, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 28, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 25
TO THE TECHNICAL SPECIFICATIONS
PROVISIONAL OPERATING LICENSE NO. DPR-16
DOCKET NO. 50-219

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

Remove

3.4-2
3.5-3
6-10
6-11

Replace

3.4-2
3.5-3
6-10
6-11

2. If at any time there are only four operable electromatic relief valves, the reactor may remain in operation for a period not to exceed 3 days provided the motor operated isolation and condensate makeup valves in both isolation condensers are demonstrated daily to be operable.
3. If Specifications 3.4.B.1 and 3.4.B.2 are not met; reactor pressure shall be reduced to 110 psig or less, within 24 hours.
4. The time delay set point for initiation after coincidence of low-low-low reactor water level and high drywell pressure shall be set not to exceed two minutes.

c. Containment Spray System and Emergency Service Water System

1. The containment spray system and the emergency service water system shall be operable at all times with irradiated fuel in the reactor vessel, except as specified in Specifications 3.4.C.3, 3.4.C.4, 3.4.C.6 and 3.4.C.8.
2. The absorption chamber water volume shall not be less than 82,000 ft³ in order for the containment spray and emergency service water system to be considered operable.
3. If one emergency service water system loop becomes inoperable, its associated containment spray system loop shall be considered inoperable. If one containment spray system loop and/or its associated emergency service water system loop becomes inoperable during the run mode, the reactor may remain in operation for a period not to exceed 7 days provided the remaining containment spray system loop and its associated emergency service water system loop each have no inoperable components and are demonstrated daily to be operable.
4. If a pump in the containment spray system or emergency service water system becomes inoperable, the reactor may remain in operation for a period not to exceed 15 days provided the other similar pump is demonstrated daily to be operable. A maximum of two pumps may be inoperable provided the two pumps are not in the same loop. If more than two pumps become inoperable, the limits of Specification 3.4.C.3 shall apply.
5. During the period when one diesel is inoperable, the containment spray loop and emergency service water system loop connected to the operable diesel shall have no inoperable components.
6. If primary containment integrity is not required (see Specification 3.5.A), the containment spray system may be made inoperable.

- b. Two of the fourteen suppression chamber - drywell vacuum breakers may be inoperable provided that they are secured in the closed position.
 - c. One position alarm circuit for each operable vacuum breaker may be inoperable for up to 15 days provided that each operable suppression chamber - drywell vacuum breaker with one defective alarm circuit is physically verified to be closed immediately and daily during this period.
6. After completion of the startup test program and demonstration of plant electrical output, the primary containment atmosphere shall be reduced to less than 5.0% O₂ with nitrogen gas within 24 hours after the reactor mode selector switch is placed in the run mode. Primary containment deinerting may commence 24 hours prior to a scheduled shutdown.
 7. If specifications 3.5.A.1.a, b, c(1) and 3.5.A.2 through 3.5.A.5 cannot be met, reactor shutdown shall be initiated and the reactor shall be in the cold shutdown condition within 24 hours.
 8. Shock Suppressors (Snubbers)
 - a. During all modes of operation except cold shutdown and refuel, all safety related snubbers listed in Table 3.5.1 shall be operable except as noted 3.5.A.8.b, c and d below.
 - b. From and after the time that a snubber is determined to be inoperable, continued reactor operation is permissible only during the succeeding 72 hours unless the snubber is sooner made operable or replaced.
 - c. If the requirements of 3.5.A.8.a and 3.5.A.8.b cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition within 36 hours.
 - d. If a snubber is determined to be inoperable while the reactor is in the shutdown or refuel mode, the snubber shall be made operable or replaced prior to reactor startup.
 - e. Snubbers may be added to safety related systems without prior License Amendment to Table 3.5.1 provided that a revision to Table 3.5.1 is included with the next License Amendment request.

B. Secondary Containment

1. Secondary containment integrity shall be maintained at all times unless all of the following conditions are met.
 - a. The Reactor is subcritical and Specification 3.2.A is met.
 - b. The reactor is in the cold shutdown condition.
 - c. The reactor vessel head or the drywell head are in place.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken in the event of a Reportable Occurrence:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each Reportable Occurrence Report submitted to the Commission shall be reviewed by the Plant Operations Review Committee and submitted to the ISRG Coordinator and Manager, Generating Stations - Nuclear.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. If any Safety Limit is exceeded, the reactor shall be shut down immediately until the Commission authorizes the resumption of operation.
- b. The Safety Limit violation shall be reported to the Commission and the Manager - Generating Stations - Nuclear.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the Plant Operations Review Committee and submitted to the Manager, Generating Stations-Nuclear. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission within 10 days of the violation. It shall also be submitted to the ISRG Coordinator.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented, and maintained that meet or exceed the requirements of Section 5.1 and 5.3 of American National Standard N18.7-1972 and Appendix "A" of the Nuclear Regulatory Commission's Regulatory Guide 1.33-1972 except as provided in 6.8.2 and 6.8.3 below.

- 6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the Plant Operations Review Committee and approved by the Station Superintendent prior to implementation and periodically as specified in the Administrative Procedures.
- 6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:
- a. The intent of the original procedure is not altered.
 - b. The change is approved by two members of the supervisory staff, at least one of whom shall be a Group Shift-Supervisor.
 - c. The change is documented, subsequently reviewed by the Plant Operations Review Committee and approved by the Station Superintendent as specified in the Administrative Procedures.

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 25 TO LICENSE NO. DPR-16

JERSEY CENTRAL POWER AND LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 25 TO LICENSE NO. DPR-16

JERSEY CENTRAL POWER AND LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

Introduction

By letter dated June 24, 1977, Jersey Central Power & Light Company (JCP&L) requested an amendment to Provisional Operating License No. DPR-16. The amendment would modify the Technical Specifications for the Oyster Creek Nuclear Generating Station in response to a NRC request of October 22, 1974 to revise Section 6, Administrative Controls, to be consistent with the format and content of the standard technical specifications developed by the Regulatory staff. The amendment also corrects editorial oversights in Specifications 3.4, 3.5, 6.6 and 6.7.

Background

By letter dated October 22, 1974, the Regulatory staff transmitted copies of Regulatory Guide 1.16 and Standard Technical Specification Section 6 on Administrative Controls. In that letter, the NRC staff requested that JCP&L submit an application for amendment for their facility to amend their Technical Specifications to be consistent with the format and content of the Standard Technical Specifications. JCP&L submitted an application for license amendment on January 10, 1975 in response to our request. We issued License Amendment No. 10 for the Oyster Creek Nuclear Generating Station on January 16, 1976. The amendment incorporated into the Technical Specifications changes to the Administrative Controls proposed by JCP&L in their request dated January 10, 1975 as modified by their letter dated May 5, 1975 and by agreed-to changes necessary to meet our requirements. However, in their

January 10, 1975 submittal, JCP&L did not propose to incorporate the Standard Technical Specification requirements in Section 6.8.1, "Procedures", because the Oyster Creek Station procedures were being updated in accordance with Sections 5.1 and 5.3 of ANSI N18.7-1977 and Appendix A of Regulatory Guide 1.33. Therefore, by letter dated March 15, 1977, the NRC advised JCP&L that the requirements for administrative procedures must be satisfied no later than September 30, 1977. JCP&L responded to our March 15, 1977 letter with their June 24, 1977 request for an amendment.

Following their January 10, 1975 proposed amendment and subsequent issue of License Amendment No. 10, JCP&L submitted an amendment request on May 20, 1976 to change Administrative Control Technical Specifications 6.5, "Safety Review and Audit". In response to this application we issued License Amendment No. 17 on November 26, 1976. The new amendment separated the "line" and advisory management responsibility for the onsite and offsite safety review. The advisory groups consist of the Independent Safety Review Groups (ISRG) and the General Office Review Board (GORB). The ISRG performs routine review and audit functions. Through editorial oversight, Specifications 6.6 and 6.7 were not modified to include the ISRG review functions. In the June 24, 1977 request JCP&L proposes to correct this oversight.

Evaluation

In the Specification changes proposed by JCP&L, editorial changes were made to Sections 3.4.C.6, and 3.5.A.8.a, c, and 3 by correcting references. Sections 6.6.1.b, 6.7.1.b, and 6.7.1.d were changed to incorporate the functions of the ISRG coordinator into the review of Reportable Occurrences and Safety Violations to be consistent with the current approved Section 6.5. Section 6.8.3.b was changed by correcting the title of the Group Shift Supervisor.

Two additional corrections were discussed by the Staff with JCP&L. These corrections were: (1) reference to Table 3.6.1 instead of 3.5.1 in Specification 3.5.A.8.e and (2) the omission of the Manager-Generating Station-Nuclear in Specification 6.6.1.b. These corrections were agreed to by the JCP&L staff and are incorporated into this amendment.

In their June 24, 1977 letter, JCP&L committed to complete the revision of the Oyster Creek Station procedures by September 30, 1977 and requested that the NRC issue the proposed Specification 6.8.1 after September 30, 1977. The proposed change is consistent with the Standard Technical Specification recommended by the NRC and is therefore acceptable.

Environmental Consideration

We have determined that this amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §1.5(d)(4) that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 28, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-219JERSEY CENTRAL POWER & LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENTTO PROVISIONAL OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 25 to Provisional Operating License No. DPR-16 issued to Jersey Central Power & Light Company which revised Technical Specifications for operation of the Oyster Creek Nuclear Generating Station, located in Ocean County, New Jersey. The amendment is effective as of its date of issuance.

The amendment revises Section 6.8, "Procedures", of the Technical Specifications in response to a request made by the NRC in a letter to the licensee dated October 22, 1974. Editorial corrections are also being made to Specifications 3.4, 3.5, 6.6, 6.7 and 6.8.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

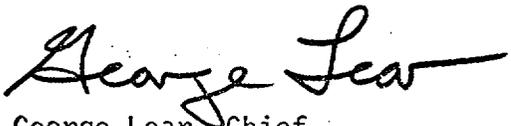
The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §1.5(d)(4) an environmental impact statement or negative

declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated June 24, 1977, (2) Amendment No. 25 to License No. DPR-16, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Ocean County Library, Brick Township Branch, 401 Chambers Bridge Road, Brick Town, New Jersey 08723. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 28 day of November 1977.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors