

March 14, 1983

Docket No. 50-219
LS05-83-03-024

Mr. P. B. Fiedler
Vice President & Director
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

DISTRIBUTION
Docket
NRC PDR
Local PDR
ORB Reading
NSIC
DEisenhut
DCrutchfield
HSmith
JLombardo
TBarnhart (4)
ACRS (10)
OPA (Clare Miles)
JMTaylor
RDiggs
JHeltemes
ORAB-Talexion
OELD
ELJordan
OELD - K. Cyr

Dear Mr. Fiedler:

SUBJECT: ORDER CONFIRMING LICENSEE COMMITMENTS ON POST-TMI RELATED ISSUES
(GENERIC LETTERS 82-05 AND 82-10)

Oyster Creek Nuclear Generating Station

The Commission has issued the enclosed Order confirming your commitments to implement those post-TMI related items set forth in NUREG-0737 for which the staff requested completion on or after July 1, 1981. This Order is based on commitments contained in your letters responding to the NRC's Generic Letters 82-05 and 82-10 dated March 17, 1982 and May 5, 1982, respectively.

The Order references your letters and, in its Attachments, contains lists of the applicable NUREG-0737 items with your schedular commitments. As discussed in the Order, several of the items listed in Generic Letter 82-10 will be handled outside of this Order.

The Commission's intention when it issued NUREG-0737 was that items would be completed in accordance with the staff's recommended schedule. However, our evaluation of your proposed schedule exceptions concludes that the proposed delays are acceptable. Among other things, the Order requires implementation of these items in accordance with your proposed schedule.

SEO!
DSA USE(SI)

Some of the items set forth in the Attachments to the Order are subject to post implementation review and inspection. Our post implementation review and/or the development of Technical Specifications may identify alterations to your method of implementing and maintaining the requirements. Any identified alterations will be the subject of future correspondence.

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PDR ADOCK 05000219
P PDR

OFFICE ▶
SURNAME ▶
DATE ▶

Mr. P. B. Fiedler

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March 14, 1983

A copy of this Order is being filed with the Office of the Federal Register for publication.

Sincerely,

Original signed by

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Enclosure:
Order

cc w/enclosure:
See next page

OFFICE	DL: ORB #5	DL: ORB #5	DL: ORAB	OELD	DL: ORB #5	DL: SA	DL: DIR
SURNAME	HSmith:cc	JLombard	Alexion		DCrutchfield	FMitaglia	DRsenhut
DATE	2/28/83	2/28/83	3/17/83	3/17/83	3/14/83	3/17/83	3/14/83

Mr. P. B. Fiedler

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March 14, 1983

cc

G. F. Trowbridge, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

Resident Inspector
c/o U. S. NRC
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Berlack, Israels & Lieberman
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Commissioner
New Jersey Department of Energy
101 Commerce Street
Newark, New Jersey 07102

Ronald C. Haynes, Regional Administrator
Nuclear Regulatory Commission, Region I
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BWR Licensing Manager
GPU Nuclear
100 Interplace Parkway
Parsippany, New Jersey 07054

Deputy Attorney General
State of New Jersey
Department of Law and Public Safety
36 West State Street - CN 112
Trenton, New Jersey 08625

Mayor
Lacey Township
818 Lacey Road
Forked River, New Jersey 08731

U. S. Environmental Protection
Agency
Region II Office
ATTN: Regional Radiation Representative
26 Federal Plaza
New York, New York 10007

Licensing Supervisor
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of)	
)	
GPU NUCLEAR CORPORATION AND JERSEY)	
CENTRAL POWER AND LIGHT COMPANY)	Docket No. 50-219
)	
(Oyster Creek Nuclear Generating)	
Station))	

ORDER CONFIRMING LICENSEE COMMITMENTS
ON POST-TMI RELATED ISSUES

I.

GPU Nuclear Corporation and Jersey Central Power and Light Company (the licensees) are the holders of Provisional Operating License No. DPR-16 which authorizes the operation of the Oyster Creek Nuclear Generating Station (the facility) at steady-state reactor power levels not in excess of 1930 megawatts thermal. The facility consists of a boiling water reactor (BWR) located in Ocean County, New Jersey.

II.

Following the accident at Three Mile Island Unit No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to provide substantial additional protection in the operation of nuclear facilities based on the

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experience from the accident at TMI-2 and the official studies and investigations of the accident. The staff's proposed requirements and schedule for implementation are set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements." Among these requirements are a number of items, consisting of hardware modifications, administrative procedure implementation and specific information to be submitted by the licensee, scheduled to be completed on or after July 1, 1981. On March 17, 1982, a letter (Generic Letter 82-05) was sent to all licensees of operating power reactors for those items that were scheduled to be implemented from July 1, 1981 through March 1, 1982. Subsequently on May 5, 1982, a letter (Generic Letter 82-10) was also sent to all licensees of operating power reactors for those items that were scheduled for implementation after March 1, 1982. These letters are hereby incorporated by reference. In these letters each licensee was requested to furnish within 30 days pursuant to 10 CFR 50.54(f) the following information for items which the staff has proposed for completion on or after July 1, 1981:

- (1) For applicable items that have been completed, confirmation of completion and the date of completion,
- (2) For items that have not been completed, a specific schedule for implementation, which the licensee committed to meet, and
- (3) Justification for delay, demonstration of need for the proposed schedule, and a description of the interim compensatory measures being taken.

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III.

GPU Nuclear Corporation responded to the Generic Letter 82-05 by letters dated April 20 and June 15, 1982 and February 18, 1983. GPU Nuclear Corporation responded to the Generic Letter 82-10 by letters dated October 8 and 19, 1982 and January 7, 1983. In these submittals, GPU Nuclear Corporation confirmed that some of the items identified in the Generic Letters had been completed and made firm commitments to complete the remainder. The attached Tables summarizing the licensee's schedular commitments or status were developed by the staff from the Generic Letters and the licensee-provided information.

Generic Letters 82-05 and 82-10 applied to fourteen and ten items respectively. Of the fourteen items listed in Generic Letter 82-05, two items are not included in this Order. You have requested that implementation of item II.B.3, "Post Accident Sampling," be deferred until the Cycle 11 refueling/maintenance outage. We believe that postponement of equipment installation beyond the Cycle 10 outage is not in the best interest of a sound emergency response position. Therefore item II.B.3 will be removed from this Order and its disposition will be determined under a separate licensing action. In addition, you have also taken the position that item II.E.4.2(7), "Isolate Purge and Vent Valves on Radiation Signal," is not applicable to the Oyster Creek Plant. We do not concur with this conclusion and therefore, item II.E.4.2(7) will be removed from this Order and disposition will be determined under separate licensing action.

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There are several items in Generic Letter 82-10 that, as noted in Attachment 2, have licensee schedules which are yet to be determined. These items are therefore not included in this Order. The licensee considers some of the items addressed in this Order to be completed or to require no modifications. These items are so noted in Attachments 1 and 2. The staff's evaluation of the licensee's delays for implementation of the remaining items is provided herein.

II.B.2 Plant Shielding

This item is complete with the exception of tie-in for the Standby Gas Treatment System (SBGTS). The SBGTS modifications will be completed during the present refueling outage (Cycle 10) which began February 12, 1983. This tie-in could only be accomplished during a refueling outage. During the interim there were two trains of SBGT, and in the event one filter train became saturated, the redundant train would have been available.

II.F.1 Accident Monitoring (1-6)

A plant shutdown is required to accomplish the modifications. The current refueling outage which began on February 12, 1983 is the first available outage of sufficient duration which has occurred since the issuance of NUREG-0737. Items II.F.1.1, 2, 4, 5 and 6 will be completed during the Cycle 10 refueling outage. For II.F.1.1, 4, 5 and 6 there are existing monitors, however, the instruments ranges are not as specified in NUREG-0737. The licensee believes that these monitors would be acceptable for all but the

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most severe accidents. Emergency response recommendations to local agencies would be based on other plant parameters and inputs from offsite monitoring teams. For II.F.1.2 the licensee states that presently there are charcoal filters in the stack gas monitoring system, however, for severe accidents they would not be accessible due to high radiation levels. In such cases the recommendations to local agencies would be accomplished based upon other plant parameters and inputs from offsite monitoring teams. For item II.F.1.3 completion can only be accomplished during a refueling outage and it has been deferred to the Cycle 11 outage because of the magnitude of the work scheduled for the present outage. The licensee has stated that there are other methods for detecting a major accident within containment such as high drywell pressure, low reactor water level, torus water level, and neutron monitoring.

II.K.3.19 Interlock of Recirculation Pump

Completion of this item can only be accomplished during a refueling outage and it has been deferred to the Cycle 11 outage because of the magnitude of the work scheduled for the present outage.

The Oyster Creek Technical Specifications (TS) safety limits require that at least two recirculation loop suction valves and their associated discharge valves be in the full open position. This requirement is also addressed in the plant operating procedures, and in addition, the licensee has installed hinged covers over each valve operating switch which serves as a warning to the operator not to violate the requirement.

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We find, based on the above evaluation, that (1) the licensee has taken corrective actions regarding the delays and has made a responsible effort to implement the NUREG-0737 requirements noted; (2) there is good cause for the several delays (unexpected design complexity, interface problems, and equipment delays); and (3) as noted above, interim compensatory measures have been provided.

In view of the foregoing, I have determined that these modifications and actions are required in the interest of public health and safety, and therefore the licensee's commitments should be confirmed by Order.

IV.

Accordingly, pursuant to Sections 103, 161i, and 161o of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED EFFECTIVE IMMEDIATELY THAT the licensees shall:

Implement and maintain the specific items described in the Attachments to this Order in the manner described in the licensee's submittals noted in Section III herein no later than the dates in the Attachments.

The licensees may request a hearing on this Order within 20 days of the date of publication of this Order in the Federal Register. A request for a hearing shall be addressed to the Director, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555.

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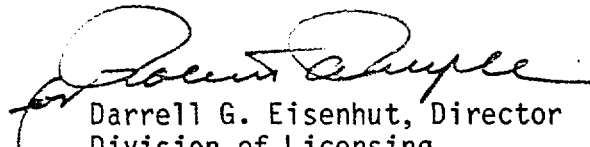
A copy shall also be sent to the Executive Legal Director at the same address. A REQUEST FOR HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

If a hearing is requested by the licensees, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether, the licensee should comply with the requirements set forth in Section IV of this Order.

This Order is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland,
this 14 day of March
1983.

Attachments:

1. Licensee's Commitments on Applicable NUREG-0737 Requirements from Generic Letter 82-05
2. Licensee's Commitments on Applicable NUREG-0737 Requirements from Generic Letter 82-10

~~REGISTER GREEN~~
 LICENSEE'S COMMITMENTS ON APPLICABLE NUREG-0737 ITEMS FROM GENERIC LETTER 82-05

Item	Title	NUREG-0737 Schedule	Requirement	Licensee's Completion Schedule (or status)*
I.A.3.1	Simulator Exams	10/01/81	Include simulator exams in licensing examinations	Complete
II.B.2	Plant Shielding	01/01/82	Modify facility to provide access to vital areas under accident conditions	Refueling outage 10 (2-83)
II.B.3	**Post Accident Sampling	01/01/82	Install upgrade post accident sampling capability	To be determined
II.B.4	Training for Mitigating Core Damage	10/01/81	Complete training program	Complete
II.E.4.2	Containment Isolation Dependability	07/01/81	Part 5-lower containment pressure setpoint to level compatible w/normal operation.	Complete
		07/01/81	**Part 7-isolate purge & vent. valves on radiation signal	To be determined
II.F.1	Accident Monitoring	01/01/82	(1) Install noble gas effluent monitors.	Refueling outage 10 (2-83)
		01/01/82	(2) Provide capability for effluent monitoring of iodine	Refueling outage 10 (2-83)
		01/01/82	(3) Install incontainment radiation-level monitors	Refueling outage 11 (4-85)
		01/01/82	(4) Provide continuous indication of containment pressure	Refueling outage 10 (2-83)
		01/01/82	(5) Provide continuous indication of containment water level.	Refueling outage 10 (2-83)
		01/01/82	(6) Provide continuous indication of hydrogen concentration in containment.	Refueling outage 10 (2-83)

*Where completion date refers to a refueling outage (the estimated date when the outage begins), the item will be completed prior to the restart of the facility

**Not Part of Confirmatory Order.

OYSTER CREEKLICENSEE'S COMMITMENTS ON APPLICABLE NUREG-0737 ITEMS FROM GENERIC LETTER 82-05

Item	Title	NUREG-0737 Schedule	Requirement	Licensee's Completion Schedule (or status)*
II.K.3.19	Interlock on Recirculation Pumps	07/01/81	Install interlocks on recirculation pump loops	Refueling outage 11 (4-85)
II.K.3.27	Common Reference Level	07/01/81	Provide common reference level for vessel level instrumentation	Complete

*Where completion date refers to a refueling outage (the estimated date when the outage begins), the item will be completed prior to the restart of the facility

LICENSEE'S COMMITMENTS ON APPLICABLE NUREG-0737 ITEMS FROM GENERIC LETTER 82-10

Item	Title	NUREG-0737 Schedule	Requirement	Licensee's Completion Schedule (or status)*
I.A.1.3.1	Limit Overtime	10/01/82 per Gen Ltr 82-12 dtd. 6/15/82	Revise administrative procedures to limit overtime in accordance w/NRC Policy Statement issued by Generic Ltr. No. 82-12, dtd 6/15/82	Complete
I.A.1.3.2	** Minimum Shift Crew	To be superseded by Proposed Rule	To be addressed in the Final Rule on Licensed Operator Staffing at Nuclear Power Units	To be addressed when Final Rule is issued.
I.C.1	** Revise Emergency Procedures	Superseded by SECY 82-111	Reference SECY 82-111, Requirements for Emergency Response Capability	To be determined
II.D.1.2	RV and SV Test Programs	04/01/82	Submit plant specific reports on relief valve and safety valve program.	Complete
II.K.3.18	ADS Actuation	09/30/82	Submit revised position on need for modifications.	Complete
II.K.3.30 & 31	** SBLOCA Analysis	1 year after staff approval of model	Submit plant specific analyses	To be determined following staff approval of model
III.A.1.2	** Staffing Levels for Emergency Situations	Superseded by 82-111	Reference SECY 82-111, Requirements for Emergency Response Capability	To be Determined
III.A.1.2	** Upgrade Emergency Support Facilities	" " " " " " " "	" " " "	" " "
III.A.2.2	** Meteorological Data	" " " " " " " "	" " " "	" " "
III.D.3.4	Control Room Habitability	To be determined by licensee	Modify facility as identified by licensee study.	Refueling outage 11 (4-85)

*Where completion date refers to a refueling outage (the estimated date when the outage begins), the item will be completed prior to the restart of the facility.
 **Not Part of Confirmatory Order.