

June 17, 1983

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Docket No. 50-219
LS05-83-06-039

Mr. P. B. Fiedler
Vice President & Director
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

Dear Mr. Fiedler:

SUBJECT: ORDER CONFIRMING LICENSEE COMMITMENTS ON POST-TMI RELATED ISSUES
(GENERIC LETTER 82-05)

Oyster Creek Nuclear Generating Station

The Commission has issued the enclosed Order confirming your commitment to implement post-TMI related items II.B.3, Post Accident Sampling and II.E.4.2.7, Isolate Purge and Vent Valves on Radiation Signal as set forth in NUREG-0737 for which the staff requested completion on or after July 1, 1981. This Order is based on commitments contained in your letters dated April 15 and May 20, 1983 responding to the NRC's Generic Letter 82-05 dated March 17, 1982.

The Order references your letters and, in its Attachment, contains the applicable NUREG-0737 items with your schedular commitments. Other post-TMI items are addressed in the Order transmitted by our letter dated March 14, 1983.

The Commission's intention when it issued NUREG-0737 was that items would be completed in accordance with the staff's recommended schedule. However, our evaluation of your proposed schedule exceptions concludes that the proposed delays are acceptable. Among other things, the Order requires implementation of these items in accordance with your proposed schedule.

Some of the items set forth in the Attachment to the Order are subject to post implementation review and inspection. Our post implementation review and/or the development of Technical Specifications may identify alterations to your method of implementing and maintaining the requirements. Any identified alterations will be the subject of future correspondence.

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PDR ADDCK 05000219
P PDR

OFFICE
SURNAME
DATE

Mr. P. B. Fiedler

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June 17, 1983

A copy of this Order is being filed with the Office of the Federal Register for publication.

Sincerely,

Original signed by
Walter A. Paulson
for Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Enclosure:
Order

cc w/enclosure:
See next page

DL: ORB #5
Tippolito
6/18/83

6/18/83
Kedyn

WB

OFFICE	DL: ORB #5	DL: ORB #5	OELD	DL: ORB #5	DL: ORB/SA	DL: DIR	DSI: CSB
SURNAME	HSmith;cc	JLombardo	Calligan Wood	DCrutchfield	FMiaglia	DEisenhut	WButler
DATE	5/26/83	5/26/83	6/18/83	6/18/83	6/18/83	6/17/83	6/18/83

Mr. P. B. Fiedler

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June 17, 1983

cc

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Washington, D. C. 20036

Resident Inspector
c/o U. S. NRC
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Trenton, New Jersey 08625

Mayor
Lacey Township
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Forked River, New Jersey 08731

U. S. Environmental Protection
Agency
Region II Office
ATTN: Regional Radiation Representative
26 Federal Plaza
New York, New York 10007

Licensing Supervisor
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of)	
)	
GPU NUCLEAR CORPORATION AND JERSEY)	
CENTRAL POWER AND LIGHT COMPANY)	Docket No. 50-219
)	
(Oyster Creek Nuclear Generating)	
Station))	

ORDER CONFIRMING LICENSEE COMMITMENTS
ON POST-TMI RELATED ISSUES

I.

GPU Nuclear Corporation and Jersey Central Power and Light Company (the licensees) are the holders of Provisional Operating License No. DPR-16 which authorizes the operation of the Oyster Creek Nuclear Generating Station (the facility) at steady-state reactor power levels not in excess of 1930 megawatts thermal. The facility consists of a boiling water reactor (BWR) located in Ocean County, New Jersey.

II.

Following the accident at Three Mile Island Unit No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to provide substantial additional protection in the operation of nuclear facilities based on the

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experience from the accident at TMI-2 and the official studies and investigations of the accident. The staff's proposed requirements and schedule for implementation are set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements." Among these requirements are a number of items, consisting of hardware modifications, administrative procedure implementation and specific information to be submitted by the licensee, scheduled to be completed on or after July 1, 1981. On March 17, 1982, a letter (Generic Letter 82-05) was sent to all licensees of operating power reactors for those items that were scheduled to be implemented from July 1, 1981 through March 1, 1982. Subsequently on May 5, 1982, a letter (Generic Letter 82-10) was also sent to all licensees of operating power reactors for those items that were scheduled for implementation after March 1, 1982. These letters are hereby incorporated by reference. In these letters each licensee was requested to furnish within 30 days pursuant to 10 CFR 50.54(f) the following information for items which the staff has proposed for completion on or after July 1, 1981:

- (1) For applicable items that have been completed, confirmation of completion and the date of completion,
- (2) For items that have not been completed, a specific schedule for implementation, which the licensee committed to meet, and
- (3) Justification for delay, demonstration of need for the proposed schedule, and a description of the interim compensatory measures being taken.

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III.

GPU Nuclear Corporation responded to the Generic Letter 82-05 by letters dated April 20 and June 15, 1982, February 18, April 15 and May 20, 1983. In these submittals, GPU Nuclear Corporation confirmed that some of the items identified in the Generic Letters had been completed and made firm commitments to complete the remainder. The attached Table summarizes the licensee's schedular commitment for Items II.B.3 and II.E.4.2.7.

Generic Letter 82-05 applied to fourteen items and of the fourteen items listed, two items were not included in the Commission's Order dated March 14, 1983 (48 FR 12179, March 23, 1983). The licensee requested that implementation of item II.B.3, "Post Accident Sampling," be deferred until the Cycle 11 refueling/ maintenance outage. The NRC determined that postponement of equipment installation beyond the Cycle 10 outage would not be in the best interest of a sound emergency response position. Therefore, the Commission determined that item II.B.3 would be handled under a separate action. In addition, the licensee had also taken the position that item II.E.4.2(7), "Isolate Purge and Vent Valves on Radiation Signal," is not applicable to the Oyster Creek Plant. The staff did not concur with this conclusion and, therefore, the Commission also determined that item II.E.4.2(7) would be handled under a separate action.

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The staff's evaluation of the licensee's delays for implementation of items II.B.3 and II.E.4.2.7 is provided herein.

II.B.3 Post Accident Sampling System (PASS)

All installation work, which requires the plant to be in a shutdown condition, will be completed during the current (Cycle 10) refueling outage.

PASS will be fully operational within six (6) months after startup from the Cycle 10 refueling outage.

An alternate methodology for estimating the extent of post accident core degradation has been developed and demonstrated during the May 24, 1983, Emergency Preparedness Drill (Note: Region I will verify procedures prior to restart).

II.E.4.2.7 Isolate Purge and Vent Valves on Radiation Signal

The delay to Cycle 11 outage is necessary because the workload of the current outage does not allow for completion of the modification at this time. At the Oyster Creek Nuclear Generating Station, the reactor building ventilation exhaust is constantly monitored by a Process Radiation Monitoring System, containing two monitors.

During normal plant operation, shutdown, or refueling operations, the normal ventilation system provides fresh, filtered air to all levels and rooms of the Reactor Building. Normal ventilation provides a minimum of one air change per hour to all areas. Air flow is from filtered supply to uncontaminated

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areas to potentially contaminated areas and then to the stack. If at anytime the radiation level becomes higher than 17 mr/hr through the ventilation system, the normal ventilation is automatically shutdown and the exhaust is routed via the Standby Gas Treatment System (SGTS). The flow through the system is limited to 4000 CFM. In addition, the Stack Gas Monitoring System for the stack constantly monitors the release to the environment. Oyster Creek operating procedures and Technical Specifications require termination of release and or shutdown, if effluent limits are exceeded.

We find, based on the above evaluation, that (1) the licensee has taken corrective actions regarding the delays and has made a responsible effort to implement the NUREG-0737 requirements noted; (2) there is good cause for the several delays (unexpected design complexity, interface problems, and equipment delays); and (3) as noted above, interim compensatory measures have been provided.

In view of the foregoing, I have determined that these modifications and actions are required in the interest of public health and safety, and therefore the licensee's commitments should be confirmed by Order.

IV.

Accordingly, pursuant to Sections 103, 161i, and 161o of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED EFFECTIVE IMMEDIATELY THAT the licensees shall:

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Implement and maintain the specific items described in the Attachment to this Order in the manner described in the licensee's submittals noted in Section III herein no later than the dates in the Attachment.

The licensees may request a hearing on this Order within 20 days of the date of publication of this Order in the Federal Register. A request for a hearing shall be addressed to the Director, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555.

A copy shall also be sent to the Executive Legal Director at the same address. A REQUEST FOR HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

If a hearing is requested by the licensees, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether, the licensee should comply with the requirements set forth in Section IV of this Order.

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This Order is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Darrell Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland,
this 17 day of June
1983.

Attachment:
Licensee's Commitments on Applicable
NUREG-0737 Requirements from Generic
Letter 82-05

LICENSEE'S COMMITMENTS ON APPLICABLE NUREG-0737 ITEMS FROM GENERIC LETTER 82-05

Item	Title	NUREG-0737 Schedule	Requirement	Licensee's Completion Schedule (or status)**
II.B.3	Post Accident Sampling System (PASS)	01/01/82	Install Upgrade Post Accident Sampling Capability	*Refueling Outage 10 (2-83)
II.E.4.2.7	Containment Isolation Dependability	07/01/81	Isolate Purge & Vent Valves on Radiation Signal	Refueling Outage 11 (4-85)

*PASS will be fully operational within six (6) months after startup.

**Where completion date refers to a refueling outage (the estimated date when the outage begins), the item will be completed prior to the restart of the facility