Docket No. 50-219
LS05-81-12-092

December 29, 1981

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Mr. I. R. Finfrock, Jr.
Vice President
Jersey Central Power & Light Company
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

Dear Mr. Finfrock:

SUBJECT: LICENSE AMENDMENT NO. 59 AUTHORIZING GPU TO OPERATE OYSTER CREEK

The Commission has issued the enclosed Amendment No. 59 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment responds to your request dated October 2, 1981, as supplemented December 4 and 16,1981.

The amendment amends the license in its entirety and approves changes to the Technical Specifications to reflect that GPU Nuclear Corporation is to replace Jersey Central Power & Light Company as the licensee authorized to possess, use and operate the Oyster Creek Nuclear Generating Station. Jersey Central Power & Light Company is the owner of the plant and, therefore, remains as one of the licensees.

We have received appropriate documentation verifying that the requirements of the Securities and Exchange Commission, State of New Jersey Department of Energy and the Federal Energy Regulatory Commission have been met. Therefore, the amended license will become effective January 1, 1982.

During our review of your proposed license change we found it necessary to make certain modifications to make the amended license consistent with current license format. We discussed these changes with your representative and mutually agreed upon them.

Copies of our related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by
Walter A. Paulson for/

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

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Enclosures:
1. Amendment No. 59 to License No. DPR-16
2. Safety Evaluation
3. Notice of Issuance

no legal objection to F.R. notice and amendment

OFFICE		DL:ORB #5/LA	DL:ORB #5/BC	OELD	DL:ORB #5/AC	State Prog.	State Prog.
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDED PROVISIONAL OPERATING LICENSE

Amendment No. 59
License No. DPR-16*

1. The Nuclear Regulatory Commission (the Commission) has found that:

A. The application for amendment filed by Jersey Central Power & Light Company and GPU Nuclear Corporation (the licensees), dated October 2, 1981, as supplemented December 4 and 16, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

B. Construction of the Oyster Creek Nuclear Generating Station (the facility) has been completed in conformity with Provisional Construction Permit No. CPPR-15, dated December 15, 1964, the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

D. There is reasonable assurance (i) that the activities authorized by this amended license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

E. The issuance of this amended license will not be inimical to the common defense and security or to the health and safety of the public;

F. GPU Nuclear Corporation is technically qualified and Jersey Central Power & Light Company is financially qualified to engage in the activities authorized by this amended operating license in accordance with the rules and regulations of the Commission;**

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*Previously revised by Amendment Nos. 3, 19, 38, 47 and 53.

**This amendment does not affect the previous finding that the owner, Jersey Central Power & Light Company, is financially qualified to obtain the necessary funds for the operation and safe shutdown, if necessary, of the Oyster Creek Nuclear Generating Station. The new operator, GPU Nuclear Corporation, derives its financial qualifications from the owner's to obtain the necessary funds for the operation of the facility.

G. Jersey Central Power & Light Company has satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements", of the Commission's regulations;

H. The receipt, possession and use of source, byproduct and special nuclear materials as authorized by this amended license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40 and 70; and

I. The issuance of this amended license is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Provisional Operating License No. DPR-16, as amended, is hereby amended in its entirety and issued to GPU Nuclear Corporation and Jersey Central Power & Light Company to read as follows:

A. This amended license applies to the Oyster Creek Nuclear Generating Station (the facility), a single cycle, forced circulation, boiling light-water reactor, and electric generating equipment (the facility). The facility is located on Jersey Central's Oyster Creek site in Lacey Township, Ocean County, New Jersey, approximately sixty miles south of Newark and forty-five miles east of Philadelphia, Pennsylvania, and is described in license application Amendment No. 3, "Facility Description and Safety Analysis Report", as supplemented and amended, and the Environmental Report (Amendment No. 68 to License Application).

B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses GPU Nuclear Corporation:

(1) Pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities", to possess, use and operate Oyster Creek Nuclear Generating Station, and Jersey Central Power & Light Company to possess the Oyster Creek Nuclear Generating Station at the designated location on the Oyster Creek site in Ocean County, New Jersey, in accordance with the procedures and limitations set forth in this amended license;

(2) Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Facility Description and Safety Analysis Report, as supplemented and amended as of December 7, 1976;

(3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use at any time any byproduct, source, and special nuclear materials as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special materials without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess such byproduct and special nuclear materials as may be produced by the operation of the facility, but not to separate such materials retained within the fuel cladding.

C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

(1) Maximum Power Level

GPU Nuclear Corporation is authorized to operate the facility at steady state power levels up to a maximum of 1930 megawatts (thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. , are hereby incorporated in the amended license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

(3) Reports

GPU Nuclear Corporation shall make certain reports in accordance with the requirements of the Technical Specifications.

(4) Records

GPU Nuclear Corporation shall keep facility operating records in accordance with the requirements of the Technical Specifications.

(5) Fire Protection

GPU Nuclear Corporation may proceed with and is required to complete the modifications identified in Paragraphs 3.1.1 through 3.1.23 of the NRC's Fire Protection Safety Evaluation (SE) on the facility dated March 3, 1978, and supplements thereto. These modifications shall be completed as specified in Table 3.1 of the SE, and supplements thereto.

(6) Physical Protection

GPU Nuclear Corporation shall maintain in effect and fully implement all provisions of the following Commission-approved documents, including amendments and changes made pursuant to the authority of 10 CFR 50.54(p), which are being withheld from public disclosure pursuant to 10 CFR 2.790(d):

(a) Security Plan collectively titled "Oyster Creek Physical Security Plan", dated December 7, 1977, as revised June 16, 1978, February 14, 1979, June 26, 1979, and August 21, 1979.

(b) Contingency Plan identified as "Oyster Creek Safeguards Contingency Plan, Revision 1", transmitted by letter dated June 3, 1980, submitted pursuant to 10 CFR 73.40. The Contingency Plan shall be fully implemented, in accordance with 10 CFR 73.40(b), within 30 days of this approval by the Commission.

(c) Guard Training and Qualification Plan identified as "Oyster Creek Nuclear Power Plant Guard Training and Qualification Plan", submitted by letter dated July 28, 1980, as revised January 7, 1981. This plan shall be fully implemented in accordance with 10 CFR 73.55(b)(4) within 60 days of this approval by the Commission. All security personnel shall be qualified within two years of this approval.

(7) GPU Nuclear Corporation shall replace all core spray spargers during the 1981 refueling outage as described in the NRC's Staff Safety Evaluation dated May 15, 1980.

3. The effectiveness of this amended license is encompassed within the effective period which has been specified for the existing license: This license is effective as of the date of issuance and shall expire at midnight April 9, 1972.*

4. This license amendment shall become effective January 1, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter A. Paulson

for Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

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*The license remains effective since a timely application dated March 6, 1972, has been filed for a Full Term License.

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 29, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 59

PROVISIONAL OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace the following pages of Appendices A and B Technical Specifications with the enclosed pages. The revised pages are identified by captioned amendment number and contain vertical lines indicating the area of change.

REMOVE PAGES	INSERT PAGES*
Appendix A	Appendix A
ii	ii
6-1 - 6-31	6-1 - 6-26
Appendix B	Appendix B
5-1 - 5-5	5-1 - 5-5

*There were no changes to the provisions contained on the following pages; they are merely included for pagination purposes: 6-17 through 6-22.

		Page
Section 4 Surveillance Requirements		
4.1	Protective Instrumentation	4.1-1
4.2	Reactivity Control	4.2-1
4.3	Reactor Coolant	4.3-1
4.4	Emergency Cooling	4.4-1
4.5	Containment	4.5-1
4.6	Radioactive Effluents	4.6-1
4.7	Auxiliary Electrical Power	4.7-1
4.8	Isolation Condenser	4.8-1
4.9	Refueling	4.9-1
4.10	ECCS Related Core Limits	4.10-1
4.11	Sealed Source Contamination	4.11-1
4.12	Fire Protection	4.12-1
4.13	Accident Monitoring Instrumentation	4.13-1
Section 5 Design Features		
5.1	Site	5.1-1
5.2	Containment	5.2-1
5.3	Auxiliary Equipment	5.3-1
Section 6 Administrative Controls		
6.1	Responsibility	6-1
6.2	Organization	6-1
6.3	Facility Staff Qualifications	6-2
6.4	Training	6-7
6.5	Review and Audit	6-7
6.6	Reportable Occurrence Action	6-13
6.7	Safety Limit Vilation	6-13
6.8	Procedures	6-16
6.9	Reporting Reauirements	6-16
6.10	Record Retention	6-23
6.11	Radiation Protection Program	6-24
6.12	(Deleted)	
6.13	High Radiation Area	6-24
6.14	Environmental Qualification	6-25*
6.15	Integrity of Systems Outside Containment	6-25
6.16	Iodine Monitoring	6-26

*Issued by NRC Order dated 10-24-80.

Amendment No. ~~29~~, ~~31~~, ~~54~~, 59

6.1 RESPONSIBILITY

6.1.1

The Director Station Operations shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

6.2.1 OFFSITE

The organizaion for GPU Nuclear Corporation for management and technical support shall be functionally as shown on Figure 6.2.1.

6.2.2 FACILITY STAFF

The facility organization shall be as shown on Figure 6.2.2 and:

a. Each on duty shift shall include at least the shift staffing indicated on Figure 6.2.2.

b. At least one licensed reactor operator shall be in the control room when fuel is in the reactor.

c. Two licensed reactor operators shall be in the control room during all reactor startups, shutdowns, and other periods involving planned control rod manipulations.

d. ALL CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

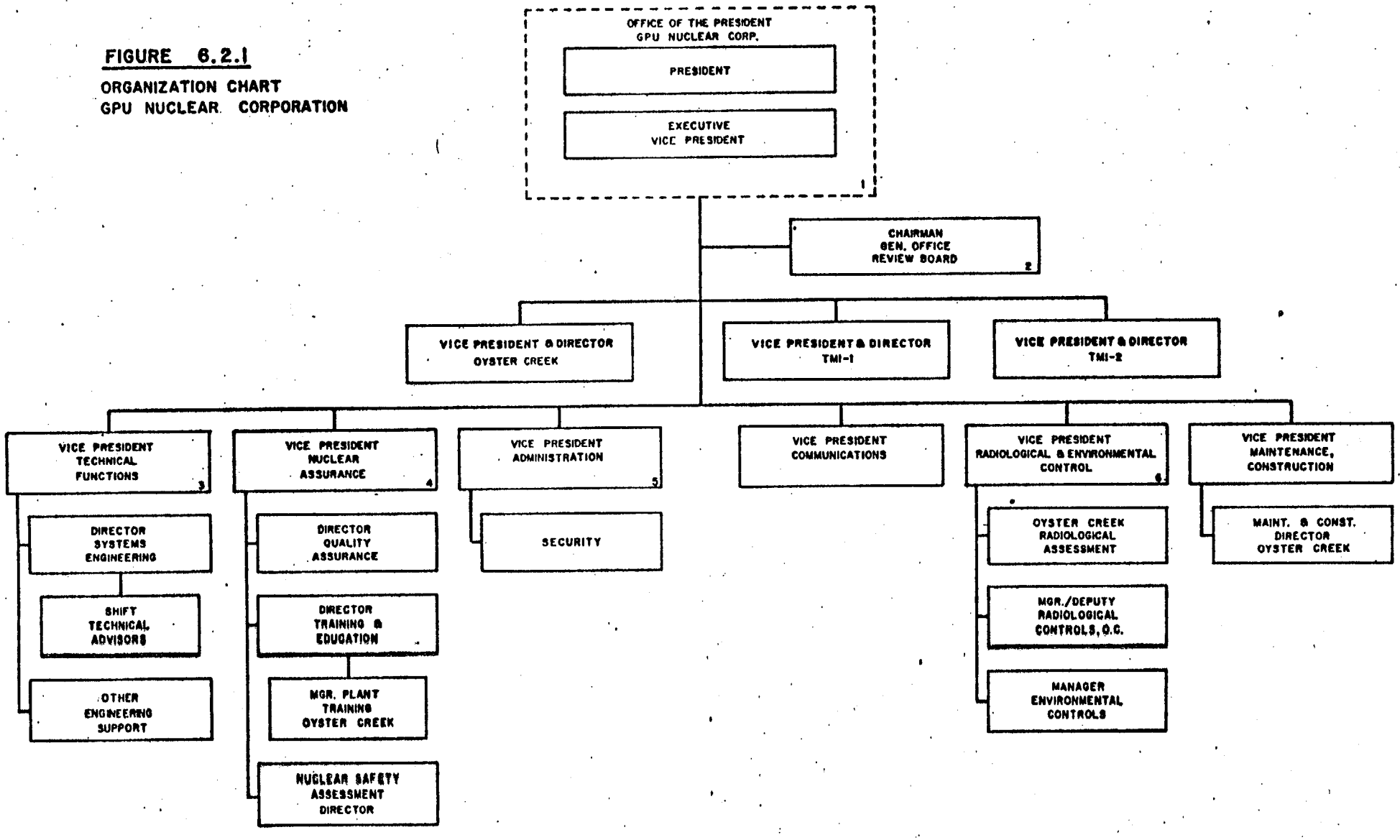
e. An individual qualified in radiation protection measures shall be on site when fuel is in the reactor.

f. A Fire Brigade of at least 5 members shall be maintained onsite at all time. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of the unit or any personnel required for other essential functions during a fire emergency.

g. Each on duty shift shall include a Shift Technical Advisor except that the Shift Technical Advisors position need not be filled if the reactor is in the refuel or shutdown mode and the reactor is less than 212°F.

FIGURE 6.2.1

ORGANIZATION CHART
GPU NUCLEAR CORPORATION



NOTES TO FIGURE 6.2.1

1. The President of the GPU Nuclear Corporation is a Senior Vice President of Met-Ed and is a Vice President of JCP&L. The Executive Vice President is a Vice President of both JCP&L and Med-Ed.

2. The General Office Review Board reports to and gets general direction from the Office of the President - GPU Nuclear Corporation. However, the GORB has direct access to the Presidents, Chief Executive Officers and Boards of Directors of the Companies involved.

3. The project engineering, the shift technical advisors, and licensing functions assigned to each nuclear plant site will report to the Vice President Technical Functions.

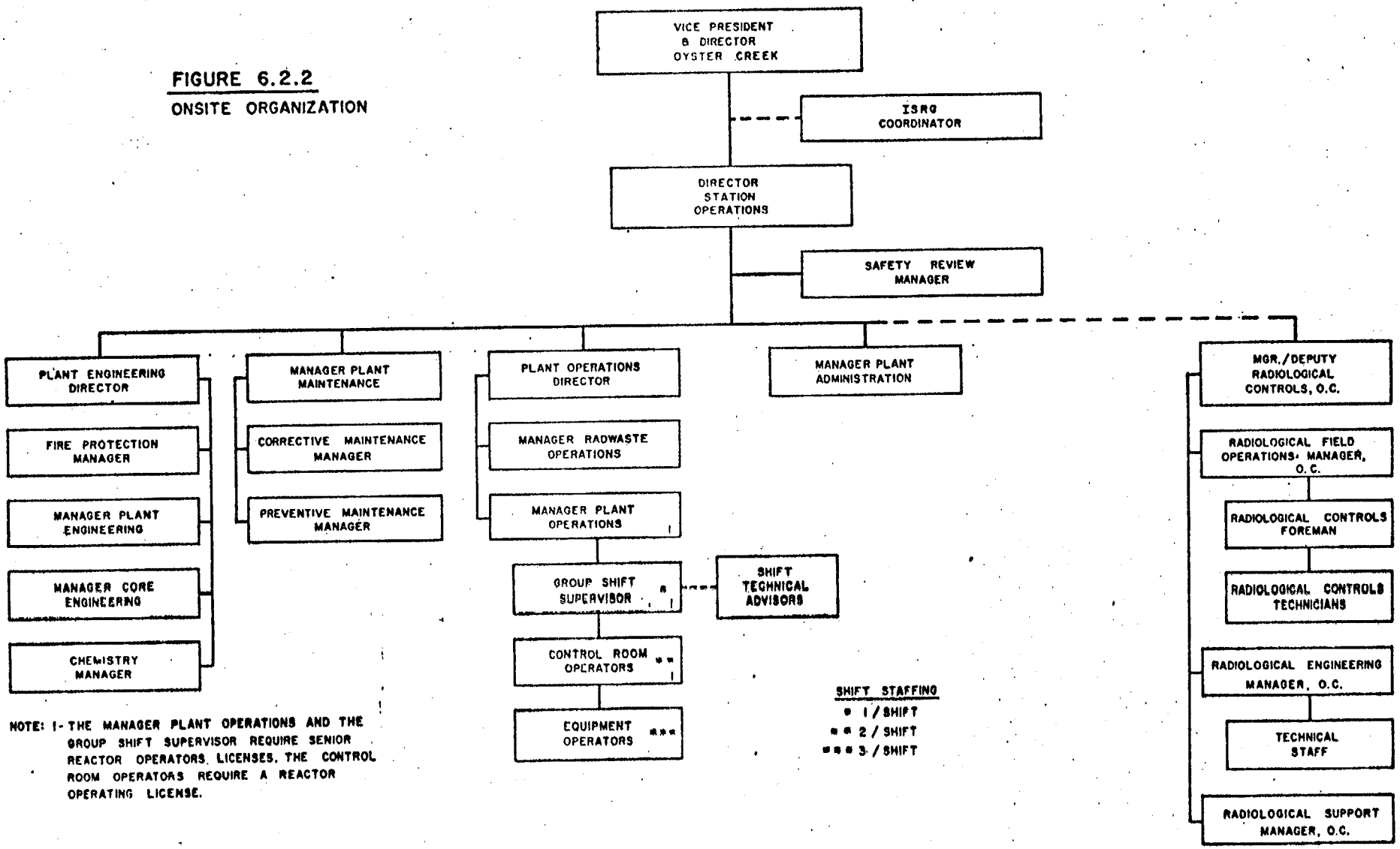
4. The quality assurance, emergency planning and training functions assigned to each nuclear plant site will report to the Vice President Nuclear Assurance.

5. The security, materials management, personnel and general administrative functions assigned to each nuclear plant site will report to the Vice President Administration.

6. The radiological and offsite environmental control functions assigned to each nuclear plant site will report to the Vice President Radiological and Environmental Controls.

FIGURE 6.2.2

ONSITE ORGANIZATION



NOTE: 1- THE MANAGER PLANT OPERATIONS AND THE GROUP SHIFT SUPERVISOR REQUIRE SENIOR REACTOR OPERATORS LICENSES. THE CONTROL ROOM OPERATORS REQUIRE A REACTOR OPERATING LICENSE.

Amendment No. ~~10~~, ~~40~~, ~~50~~, 59

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1

The members of the facility staff shall meet or exceed the following qualifications:

Director Station Operations

Requirements: Ten years total power plant experience of which three years must be nuclear power plant experience. A maximum of four years of academic training may fulfill four of the remaining seven years of required experience. The Director must be capable of obtaining or possess a Senior Reactor Operator"s License.

Plant Operations Director

Requirements: Eight years total power plant experience of which three years must be nuclear power plant experience. A maximum of two years of academic or related technical training may fulfill two years of the remaining five years of required experience. The Plant Operations Director must be capable of obtaining or possess a Senior Reactor Operator's License.

Plant Engineering Director

Requirements: Eight years of responsible positions related to power generation, of which three years shall be nuclear power plant experience. A maximum of four of the remaining five years of experience may be fulfilled by satisfactory completion of academic or related technical training.

Manager Plant Administration

Requirements: Eight years total power plant experience of which four years must have been in nuclear power plant experience. The Manager should possess a four year college degree or equivalent in Business Administration or an Engineering discipline.

Manager Plant Operations

Requirements: Eight years total power plant experience of which three years must be nuclear power plant experience. A maximum of two years of academic or related technical training may fulfill two of the remaining five years of required experience. The Manager Plant Operations must possess a Senior Reactor Operator's License.

Safety Review Manager

Requirements: Eight years total power plant experience of which three years must be nuclear power plant experience. A maximum of two years of academic or related technical training may fulfill two of the remaining five years of required experience.

Manager Core Engineering

At the time of initial core loading or appointment to the position, whichever is later, the responsible person shall have a Bachelor's Degree in Engineering or the Physical Sciences and four years experience or a graduate degree and three years experience. Two of these years shall be nuclear power plant experience. The experience shall be in such areas as reactor physics, core measurements, core heat transfer, and core physics testing programs. Successful completion of a reactor engineering training program (such as the 12 week concentrated programs offered by NSS Vendors) may be equivalent to one year's nuclear power plant experience.

Manager Plant Maintenance

Requirements: Seven years of total power plant experience of which one year must be nuclear power plant experience. Two years of academic or related technical training may fulfill two of the remaining six years of required experience.

Instrument Maintenance Supervisor

Requirements: Five years of experience in instrumentation and control, of which a minimum of one year shall be in nuclear instrumentation and control at an operating nuclear power plant. A maximum of four years of this five year experience may be fulfilled by related technical or academic training.

Manager Plant Engineering

The engineer in charge of technical support shall have a Bachelor's Degree in Engineering or the Physical Sciences and have three years of professional level experience in nuclear services, nuclear plant operation, or nuclear engineering, and the necessary overall nuclear background to determine when to call consultants and contractors for dealing with complex problems beyond the scope of owner-organization expertise.

Mgr/Deputy Radiological Controls (Reports Offsite)

Requirements: Bachelor's degree or the equivalent in a science or engineering subject, including some formal training in radiation protection. Five years of professional experience in applied radiation protection. (Master's degree equivalent to one year experience and Doctor's degree equivalent to two years experience where coursework related to radiation protection is involved.) Three years of this professional experience should be in applied radiation protection work in a nuclear facility dealing with radiological problems similar to those encountered in nuclear power stations.

Chemistry Manager

Requirements: Five years experience in chemistry of which a minimum of one year shall be in radiochemistry at an operating nuclear power

plant. A maximum of four years of this five year experience may be fulfilled by related technical or academic training.

Shift Technical Advisor

Requirements: Bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.3.2

Each member of the radiation protection organization for which there is a comparable position described in ANSI N18.1-1971 shall meet or exceed the minimum qualifications specified therein, or in the case of radiation protection technicians, they shall have at least one year's continuous experience in applied radiation protection work in a nucleaar facility dealing with radiological problems similar to those encountered in nuclear power stations, and shall have been certified by the Mgr/Deputy Radiological Controls, as qualified to perform assigned functions. This certification must be based on an NRC approved, documented program consisting of classroom training with appropriate examinations and documented positive findings by responsible supervision that the individual has demonstrated his ability to perform each specified procedure and assigned function with an understanding of its basis and purpose.

6.4 TRAINING

6.4.1

A retraining program for operators shall be maintained under the direction of the Manager Plant Training Oyster Creek and shall meet the requirements and recommendation of Appendix A of 10CFR Part 55. Replacement training programs, the content of which shall meet the requirements of 10CFR Part 55, shall be conducted under the direction of the Manager Plant Training Oyster Creek for licensed operators and Senior Reactor Operators.

6.4.2

A training program for the Fire Brigade shall be maintained under the direction of the Manager Plant Training Oyster Creek.

6.5 SAFETY REVIEW AND AUDIT

The Director Station Operations and three organizational units, the Plant Operations Review Committee (PORC), the Independent Safety Review Groups (ISRG) and the General Office Review Board (GORB) function to accomplish nuclear safety review and audit of the Oyster Creek Station.

6.5.1 DIRECTOR STATION OPERATIONS

6.5.1.1 FUNCTION

The Director Station Operations shall ensure that:

a. All proposed changes to equipment or systems have been evaluated to determine if they constitute a change to the facility or procedures as described in the Safety Analysis Report.

b. All proposed changes to equipment or systems that constitute a change of the facility or procedures as described in the Safety Analysis Report have been evaluated to determine whether they involve an unreviewed safety question as defined in paragraph 50.59, Part 50, Title 10, Code of Federal Regulations.

c. All proposed tests and experiments have been evaluated to determine whether or not they involve unreviewed safety questions as defined in paragraph 50.59, Part 50, Title 10, Code of Federal Regulations.

6.5.1.2 AUTHORITY

The Director Station Operations has the authority to:

a. Make a determination as to whether proposed changes to equipment, or systems involve a change to the procedures or facility as described in the Safety Analysis Report.

b. Make a determination as to whether or not proposed tests or experiments and changes to equipment or systems involve an unreviewed safety question.

c. Direct the Plant Operations Review Committee to review safety evalutions of proposed changes to equipment or systems and safety evaluations of proposed tests and experiments to determine whether or not such changes, tests or experiments involve unreviewed safety questions.

NOTE: Each determination that a proposed test, experiment, or change to a system or equipment that does not involve an unreviewed safety question shall be reviewed by the Independent Safety Review Groups to verify that the determination was correct. This review shall be documented but is not a pre-requisite of the test, experiment, or change to a system or equipment.

6.5.1.3 RECORDS

Any safety evaluations done in accordance with 6.5.1.1 (b) and (c) and any determinations made pursuant to 6.5.1.2(b) must be documented. Copies of these determinations shall be provided to the ISRG Coordinator and the Chairman of the General Office Review Board. Records of all tests and experiments performed and all changes to equipment or systems made under the provisions of 10 CFR

Part 50.59 shall also be maintained at the station.

6.5.2 PLANT OPERATIONS REVIEW COMMITTEE (PORC)

6.5.2.1 FUNCTION

The PORC shall function to advise the Director Station Operations.

6.5.2.2 COMPOSITION

The PORC shall consist of the following plant personnel:

Safety Review Manager
Plant Operations Director
Plant Engineering Director
Manager Plant Maintenance
Mgr/Deputy Radiological Controls

The Director Station Operations shall designate the Chairman and Vice-Chairman from among the PORC members.

6.5.2.3 ALTERNATES

Alternate members shall be appointed in writing by the PORC Chairman and will have the type of experience and training required of regular members. However, they need not have the extensive longevity in the designated fields as long as in the opinion of the Chairman, their experience and judgement are adequate.

6.5.2.4 MEETING FREQUENCY

The PORC shall meet at least once per calendar month and as convened by the PORC Chairman or the Director Station Operations.

6.5.2.5 QUORUM

A quorum of the PORC shall consist of the Chairman or Vice Chairman and three members/alternates. No more than two alternate members shall be counted in establishing a quorum.

6.5.2.6 RESPONSIBILITIES

The responsibilities of the PORC are included in Table 6.5.1.

6.5.2.7 AUTHORITY

a. The PORC shall be advisory to the Director Station Operations. Nothing herein shall relieve the Director Station Operations of his responsibility or authority for overall safety operations including taking immediate emergency action. Determinations on items a and b of Table 6.5-1 shall be documented in writing.

b. The PORC Chairman shall immediately notify the Vice President

TABLE 6.5-1

SAFETY REVIEW RESPONSIBILITIES

ITEM	INITIAL ACTION	PORC	INDEPENDENT REVIEW ISRG	INDEPENDENT REVIEW GORB
a) Proposed change to equipment, or systems subject to the Provisions of Section 50.59, Part 50, Title 10 Code of Federal Regulations.	Initiator: Must prepare a complete description of the proposed changes and ensure a safety evaluation of the change is included. Director Station Operations (1) Must determine if the item is an actual change to equipment or systems as described in the FSAR. (2) Must determine if the item involves an unreviewed safety question. (3) May request the PORC to assist in the above determinations.	Must review items to determine whether or not an unreviewed safety question is involved, if required by the Director Station Operations.	Must review all determinations by the Director Station Operations.	May review any determination, but must review those for which the Director Station Operations has requested GORB review.
b) Proposed tests and experiments, (subject to provisions of) 50.59, Part 50, Title 10, Code of Federal Regulations.	Initiator: Must prepare a complete description of the proposed test or experiment and ensure a safety evaluation of the test or experiment is included. Director Station Operations (1) Must determine if the item involves an unreviewed safety question. (2) May request the PORC to assist in the above determinations.	Must review item to determine whether or not an unreviewed safety question is involved, if requested by the Director Station Operations.	As above.	As above.
c) Proposed changes in Technical Specifications or in the NRC Operating License.	Initiator: Must prepare a complete description of the proposed change and ensure a safety evaluation of the change is included.	(1) Must review the item for nuclear and radiological safety. (2) Must make recommendations to the Director Station Operations as to whether or not the change is safe.	Must review change and PORC recommendation prior to submittal to the NRC.	May review any item but must review those for which the Director Station Operations or his supervisor have requested GORB review.
d) Reportable occurrences	Director Station Operations: Must have investigations performed for all Reportable Occurrences and a report prepared including the safety significance of the incident.	Must review Reportable Occurrences Report for safety significance and make recommendations to the Director Station Operations on how to avoid recurrence.	Must review Reportable Occurrence Reports for safety significance and review PORC recommendations.	As above.

TABLE 6.5-1 (CON'T)

SAFETY REVIEW RESPONSIBILITIES

ITEM	INITIAL ACTION	PORC	INDEPENDENT REVIEW ISRG	INDEPENDENT REVIEW GORB
e) Facility operations including Security Plan, Emergency Plan and Implementing procedures; review is to detect potential safety hazards.		Continuing responsibility	See Item i below	As above
f) Significance operation abnormalities or deviations from normal and expected performance.	Director, Station Operations Report such matters to the PORC ISRG Coordinator and the Chairman GORB.	Review matter and report evaluation of safety significance to the ISRG and GORB.	Perform independent review of PORC evaluation.	As above
g) Any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems or components.	As above	As above	As above	As above
h) PORC minutes and reports.			Review to determine if any matters discussed involve unrelated safety questions.	As above
i) Audit Reports and NRC Inspection Reports.			Review to determine if any matters reported involve Violations of Technical Specifications license requirements or regulations or have any nuclear or radiation safety implications.	The report of the management review of the QA Plan, initiated by the Vice President, and Director Oyster Creek in accordance with the Operational Quality Assurance Plan, shall be reviewed by the GORB with respect to technical and administrative safety issues.

and Director Oyster Creek and the Chairman of the GORB of any disagreements between the PORC and the Director Station Operations. However, the Director Station Operations shall have the responsibility for resolution of such disgreements pursuant to 6.1.1.

6.5.2.8 RECORDS

The PORC shall maintain written minutes of each meeting and copies of minutes and determinations shall be provided to the Director Station Operations, Vice President and Director Oyster Creek, ISRG Coordinator, and the Chairman of the GORB.

6.5.3 INDEPENDENT SAFETY REVIEW GROUPS (ISRG)

6.5.3.1 FUNCTION AND COMPOSITION

The ISRG shall function under the direction of an ISRG Coordinator, who shall be appointed by the Vice President and Director Oyster Creek to provide safety reviews. The Coordinator shall have available the competence to review problems in the following area:

a. Nuclear Power Plant Operations
b. Nuclear Engineering
c. Chemistry and Radiochemistry
d. Metallurgy
e. Instrumentation and Control
f. Radiological Safety
g. Mechanical and Electrical Engineering
h. Quality Assurance Practices

The Coordinator shall establish, as needed, groups of two or more individuals with the expertise required for each topic to be reviewed.

6.5.3.2 CONSULTANTS

Consultants shall be utilized as necessary to supplement the expertise available in the GPU Nuclear Corporation.

6.5.3.3 RESPONSIBILITIES

The specific responsibility to ensure accomplishment of independent safety review of the Director Station Operations determinations involving safety questions is assigned to the ISRG Coordinator and is accomplishd by utilizing, as necessary, the full scope of expertise available in the GPU Nuclear Corporation staff, consultants, contractors and vendors as appropriate. Table 6.5-1 defines the specific independent safety review responsibilities.

6.5.3.4 AUTHORITY

The ISRG advises the Vice President and Director Oyster Creek. It has the authority to conduct reviews and investigations, which will

be documented.

6.5.3.5 AUDITS

Audits of facility activities shall be performed under the cognizance of the Vice President Nuclear Assurance. These audits shall encompass:

a. The conformance of facility operation to all provisions contained within the Technical Specifications and applicable license conditions at least once per year.

b. The performance training and qualifications of the entire facility staff at least once per year.

c. The results of all actions taken to correct deficiences occurring in faciliy equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.

d. The Facility Emergency Plan and implementing procedures at least once per two years.

e. The Facility Security Plan and implementing procedures at least once per two years.

f. Any other area of facility operation considered appropriate by the GORB or the Vice President and Director Oyster Creek.

6.5.3.6 RECORDS

Written documentation of all independent safety reviews and investigations will be forwarded to the Director Station Operations, Vice President and Director Oyster Creek and the Chairman of the General Office Review Board. In addition, any reportable occurrence or item involving an unreviewed safety question which is identified by the ISRG will be documented and reported immediately to the above mentioned persons.

The audit findings which result from all audits conducted in accordance with Section 6.5.3.5 shall be documented and reported to the above mentioned persons within 30 days after completing the audit. Reports documenting corrective action will receive the same distribution and they will also be forwarded to the ISRG Coordinator.

6.5.4 GENERAL OFFICE REVIEW BOARD (GORB)

6.5.4.1 FUNCTION

The technical and administrative function of the GORB is to provide independent review of major safety issues, to foresee potentially significant nuclear and radiation safety problems, and to advise the Office of the President on these matters.

6.5.4.2 COMPOSITION

Members of the General Office Review Board shall possess extensive experience in their individual specialities and collectively have the competence in the following areas:

a. Nuclear Power Plant Operations
b. Nuclear Engineering
c. Chemistry and Radiochemistry
d. Metallurgy
e. Instrumentation and Control
f. Radiological Safety
g. Mechanical and Electrical Engineering

The Chairman and Vice Chairman shall be appointed by the Office of the President. (Neither shall be an individual with line responsibility for operation of the plant).

The Chairman shall designate a minimum of six additional members. No more than a minority of the Board shall have line responsibility for operation of Oyster Creek Nuclear Generating Station.

6.5.4.3 ALTERNATES

Alternate members shall be appointed in writing by the GORB Chairman and will have the type of experience and training required of regular members, however, they need not have the extensive longevity in the designated fields as long as, in the opinion of the Chairman, their experience and judgement are adequate.

6.5.4.4 MEETING FREQUENCY

The GORB shall meet at least semi-anually and any time at the request of the Chairman or the Office of the President.

6.5.4.5 QUORUM

A quorum shall consist of the Chairman or Vice Chairman and three members/alternates. No more than one alternate member shall be counted when establishing a quorum and no more than a minority of the quorum shall hold line responsibility for operations of the Oyster Creek Station.

6.5.4.6 RESPONSIBILITIES

a. The primary responsibility of the GORB is to foresee potentially significant nuclear and radiation safety problems and to recommend to the Office of the President how they may be avoided or mitigated.

b. Carry out the specific independent safety review responsibilities listed in Table 6.5-1.

6.5.4.7 AUTHORITY

The GORB shall be advisory to the Office of the President and shall have the authority to conduct reviews, audits, and investigations requested by the Office of the President or as deemed necessary by the GORB in the fulfillment of its responsibilities.

6.5.4.8 AUDITS

The report of the management review of the QA Plan, initiated by the Vice President and Director Oyster Creek in accordance with the Operational Quality Assurance Plan, shall be reviewed by the GORB with respect to safety and administrative safety issues.

6.5.4.9 RECORDS

Minutes of each GORB meeting shall be recorded and approved by the GORB Chairman. Copies of approved minutes will be forwarded to the Office of the President, Vice President and Director Oyster Creek, Director Station Operations, PORC Chairman, and others designated by the GORB Chairman. GORB recommendations to the Office of the President will be documented in a letter from the GORB Chairman to the Office of the President. Included with each letter will be any dissenting opinions of members of the Board.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1

The following actions shall be taken in the event of a Reportable Occurrence:

a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.

b. Each Reportable Occurrence Report submitted to the Commission shall be reviewed by the Plant Operations Review Committee and submitted to the ISRG Coordinator, Director Station Operations, and Vice President and Director Oyster Creek.

6.7 SAFETY LIMIT VIOLATION

6.7.1

The following actions shall be taken in the event a Safety Limit is violated:

a. If any Safety Limit is exceeded, the reactor shall be shut down immediately until the Commission authorizes the resumption of operation.

b. The Safety Limit violation shall be reported to the Commission and the Vice President and Director Oyster Creek.

c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the Plant Operations Review Committee and submitted to the Director Station Operations and Vice President and Director Oyster Creek. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components systems or structures, and (3) corrective action taken to prevent recurrence.

d. The Safety Limit Violation Report shall be submitted to the Commission within 10 days of the violation. It shall also be submitted to the ISRG Coordinator.

6.8 PROCEDURES

6.8.1

Written procedures shall be established, implemented, and maintained that meet or exceed the requirements of Section 5.1 and 5.3 of American National Standard N18.7-1972 and Appendix "A" of the Nuclear Regulatory Commission's Regulatory Guide 1.33-1972 except as provided in 6.8.2 and 6.8.3 below.

6.8.2

Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the Plant Operations Review Committee and approved by the Director Station Operations prior to implementation and periodically as specified in the Administrative Procedures.

6.8.3

Temporary changes to procedures 6.8.1 above may be made provided:

a. The intent of the original procedure is not altered.

b. The change is approved by two members of the supervisory staff, at least one of whom possesses a Senior Reactor Operator's License.

c. The change is documented, subsequently reviewed by the Plant Operations Review Committee and approved by the Director Station Operations as specified in the Administrative Procedures.

6.9 REPORTING REQUIREMENTS

In addition to the applicable reporting requirements of 10 CFR, the following identified reports shall be submitted to the Director of the appropriate Regional Office of Inspection and Inforcement unless otherwise noted.

6.9.1 ROUTINE REPORTS

a. Startup Report. A summay report of plant startup and power escalation testing shall be submitted folowing (1) receipt of an

operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtan satisfactory operation shall also be decribed. Any additional specified details required in license conditions based on other commitments shall be included in this report.

Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplemetary reports shall be submitted at least every three months until all three events have been completed.

b. Annual Exposure Data Report. Routine exposure data reports covering the operation of the unit during the previous calendar year shall be submitted prior to March 1 of each year. Reports shall contain a tabulation on an annual basis of the number of station, utility, and other personnel (including contractors) receiving exposures greater than 100 mrem/year and their associated man rem exposure according to work and job functions (This tabulation supplements the requirements of 10 CFR 20.407), e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintence (describe maintence), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose recieved from external sources shall be assigned to specific major work functions.

c. Monthly Operating Report. Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis which will include a narrative of operating experience, to the Director, Office of Management and Program Control, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the Regional Office of I&E, no later than the 15th of each month following the calendar month covered by the report.

6.9.2 REPORTABLE OCCURRENCES

Reportable occurrences, including corrective actions and measures to prevent reoccurrences, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of

occurrence. In case of corrected or supplemental reports, a license event report shall be completed and reference shall be made to the original report date.

a. Prompt Notification With Written Followup. The types of events listed below shall be reported as expeditiously as possible, but within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the appropriate Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide a complete explanation of the circumstances surrounding the event.

(1) Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protecive function by the time a monitored parameter reaches the setpoint specified as the limiting system setting in the technical specifications or failure to complete the required protective function.

NOTE: Instrument drift discovered as a result of testing need not be reported under this item but may be reportable under items 2.a(5), 2.a(6), or 2.b(1) below.

(2) Operation of the unit or affected systems when any parameter or operation subject to a limiting condition is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.

NOTE: If specified action is taken when a system is found to be operating between the most conservative and the least conservative aspects of a limiting condition for operation listed in the technical specifications, the limiting condition for operation is not considered to have been violated and need not be reported under this item, but it may be reportable under item 2.b(2) below.

(3) Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

NOTE: Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

(4) Reactivity anormalies, involving disagreement with the predicted value of reactivity balance under steady state conditions during power operation, greater than or equal to 1% $\Delta k/k$; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if sub-critical,

an unplanned reactivity insertion of more than 0.5% $\Delta k/k$ or occurrence of any unplanned criticality.

(5) Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.

(6) Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.

NOTE: For items 2.a(5) and 2.a(6) reduced redundancy that does not result in a loss of system function need not be reported under this section but may be reportable under items 2.b(2) and 2.b(3) below.

(7) Conditions arising from natural or man-made events that, as a direct result of the event require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.

(8) Errors discovered in the transient or accident analyses or in methods used for such analyses as described in the safety report or in the bases for the technical specifications that or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

(9) Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

NOTE: This item is intended to provide for the reporting of potentially generic problems.

b. Thirty Day Written Reports. The reportable occurrences discussed below shall be the subject of written reports to the Director of the appropriate Regional Office within thirty days of occurrence of the event. The written report shall include, as a minimum, a complete copy of a licensee event report form. Information provided on the licensee event form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

(1) Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional

requirements of affected systems.

(2) Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

NOTE: Routine surveillance testing, instrument calibration, or preventive maintenance which require system configurations as described in items 2.b(1) and 2.b(2) need not be reported except where test results themselves reveal a degraded mode as described above.

(3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.

(4) Abnormal degradation of systems other than those specified in item 2.a(3) above designed to contain radioactive material resulting from the fission process.

NOTE: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

6.9.3 UNIQUE REPORTING REQUIREMENTS

Special reports shall be submitted to the Director of Regulatory Operations Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification.

a. Materials Radiation Surveillance Specimen Reports (4.3A)

b. Integrated Primary Containment Leakage Tests (4.5)

c. Semi-annual reports specifying effluent release shall be submitted to the NRC. These reports shall include the following:

(1) Radioactive Effluent Releases

A statement of the quantities of radioactive effluents released from the plant with data summarized on a monthly basis following the format of USAEC Guide 1.21.

(a). Gaseous Effluents

1. Gross Radioactivity Releases

a. Total gross radioactivity (in curies), primarily noble and activation gases.

b. Maximum gross radioactivity release rate during any one-hour period.

c. Total gross radioactivity (in curies) by nuclide release based on representative isotopic analyses performed.

d. Percent of technical specification limit.

2. Iodine Releases

a. Total iodine radioactivity (in curies) by nuclide released based on representative isotopic analyses performed.

b. Percent of technical specification limit for I-131 released.

3. Particulate Releases

a. Total gross radioactivity (β,γ) released (in curies) excluding background radioactivity.

b. Gross alpha radioactivity released (in curies) excluding background radioactivity.

c. Total gross radioactivity (in curies) of nuclides with half-lives greater than eight days.

d. Percent of technical specification limit for particulate radioactivity with half-lives greater than eight days.

4. Liquid Effluents

a. Total gross radioactivity (β,γ) released (in curies) excluding tritium and average concentration released to the unrestricted area.

b. The maximun concentration of gross radioactivity (β,γ) released to the unrestricted area (averaged over the period of release).

c. Total tritium and total alpha radioactivity (in curies) released and average concentration released to the unrestricted area.

d. Total dissolved gas radioactivity (in curies) and averaged concentration released to the unrestricted area.

e. Total volume (in liters) of liquid waste released.

f. Total volume (in liters) of dilution water used prior to release from the restricted area.

g. Total gross radioactivity (in curies) by nuclide released based on representative isotopic analyses performed.

h. Percent of technical specification limit for total radioactivity.

(2). Solid Waste

(a). The total amount of solid waste shipped (in cubic feet).

(b). The total estimated radioactivity (in curies) involved.

(c). Disposition including date and destination.

(3). Environmental Monitoring

(a). For each medium sampled during the reporting period, e.g., air, baybottom, surface water, soil, fish, include:

1. Number of sampling locations.

2. Total number of samples.

3. Number of locations at which levels are found to be significantly above local backgrounds, and

4. Highest, lowest, and the average concentrations or level of radiation for the sampling point with the highest average and description of the location of that point with respect to the site.

(b). If levels of radioactive materials in environmental media as determined by an environmental monitoring program indicate the likelihood of public intakes in excess of 1% of those that could result from continuous exposure to the concentration values listed in Appendix B, Table II, Part 20 estimates of the likely resultant exposure to individuals and to population groups, and assumptions upon which estimates are based shall be provided.

(c). If statistically significant variations of offsite evironmental concentrations with time are observed, correlation of these results with effluent release shall be provided.

(d). Results of required leak tests performed on sealed sources if the tests reveal the presence of 0.005

microcuries or more of removeable contamination.

d. Inoperable Fire Protection Equipment (3.12)

e. Core Spray Sparger Inservice Inspection (Table 4.3.1-9)

Prior to startup of each cycle, a special report presenting the results of the inservice inspection of the Core Spray Spargers during each refueling outage shall be submitted to the Commission for review.

6.10 RECORD RETENTION

6.10.1

The following records shall be retained for at least five years:

a. Records and logs of facility operation covering time interval at each power level.

b. Records and logs of principle maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.

c. Reportable occurrence reports.

d. Records of surveillance activities, inspections and calibrations required by these technical specifications.

e. Records of reactor tests and experiments.

f. Records of changes made to operating procedures.

g. Records of radioactive shipments.

h. Records of sealed source leak tests and results.

i. Records of annual physical inventory of all source material of record.

6.10.2

The following records shall be retained for the duration of the Facility Operating License:

a. Record and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.

b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.

c. Records of facility radiation and contamination surveys.

d. Records of radiation exposure for all individuals entering radiation control areas.

e. Records of gaseous and liquid radioactive material released to the environs.

f. Records of transient or operational cycles for those facilitiy components designed for a limited number of transients or cycles.

g. Records of training and qualification for current members of the plant staff.

h. Records of inservice inspections performed pursuant to these technical specifications.

i. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.

j. Records of meetings of the Plant Operations Review Committee and the General Office Review Board.

k. Records for Environmental Qualification which are covered under the provisions of paragraph 6.14

6.10.3

Quality Assurance Records shall be retained as specified by the Quality Assurance Plan.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 (Deleted)

6.13 HIGH RADIATION AREA

6.13.1

In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP).

NOTE: Health Physics personnel shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they are following plant radiation protection procedures for entry into high radiation ares.

An individual or group of individuals permitted to enter such areas shall be provided with one or more of the following:

a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.

b. A radiation monitoring device which continuously intergrates the radiation dose rate in the area and alarms when a pre-set integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them.

c. A health physics qualified individual (i.e. qualified in radiation protection procedures) with a radiation dose rate monitoring device who is responsible for providing positive exposure control over the activities within the area and who will perform periodic radiation surveillance at the frequency in the RWP. The surveillance frequency will be established by the Radiological Controls Manager.

6.13.2

Specification 6.13.1 shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of operations and/or radiation protection supervision on duty.

6.14 ENVIRONMENTAL QUALIFICATION

A. By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of: Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class IE Electrical Equipment in Operating Reactors" (DOR Guidelines); or, NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment", December 1979. Copies of these documents are attached to Order for Modification of License DPR-16 dated October 24, 1980.

B. By no later than December 1, 1980, complete and auditible records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Therafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

6.15 Integrity of Systems Outside Containment

The licensee shall implement a program to reduce leakage from

systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1) Provisions establishing preventive maintenance and periodic visual inspection requirements, and

2) System leak test requirements, to the extent permitted by system design and radiological conditions, for each system at a frequency not to exceed refueling cycle intervals. The systems subject to this testing are (1) Core Spray, (2) Containment Spray, (3) Reactor Water Cleanup, (4) Isolation Condenser and (5) Shutdown Cooling.

6.16 Iodine Monitoring

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas* under accident conditions. This program shall include the following:

1. Training of personnel,

2. Procedures for monitoring, and

3. Provisions for maintenance of sampling and analysis equipment.

* Areas requiring personnel access for establishing hot shutdown condition.

APPENDIX B

TO FULL-TERM OPERATING LICENSE

ENVIRONMENTAL TECHNICAL SPECIFICATIONS

FOR

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

OCEAN COUNTY, NEW JERSEY

JERSEY CENTRAL POWER & LIGHT COMPANY

GPU NUCLEAR CORPORATION

November, 1978*

*Issued June 6, 1979 by Amendment No. 37

5.0 ADMINISTRATIVE CONTROLS

This section describes administrative and management controls established by the Applicant to provide continuing protection to the environment and to implement the environmental technical specifications.

5.1 Responsibility

Corporate responsibility for implementation of the Oyster Creek Environmental Technical Specifications and for assuring that plant operations are controlled in such a manner as to provide continuing protection of the environment has been assigned by the Office of the President to the Vice President and Director Oyster Creek.

The responsibility for conducting the studies as set forth in Section 3.1 (Non-Radiological Surveillance) and all of Section 4.0 (Special Surveillance Programs) rests with the GPUNC Manager, Environmental Controls.

Administrative measures are defined in Section 5.3 which provide that the individual or group responsible for auditing or otherwise verifying that an activity has been correctly performed is independent of the individual or group responsible for performing the activity.

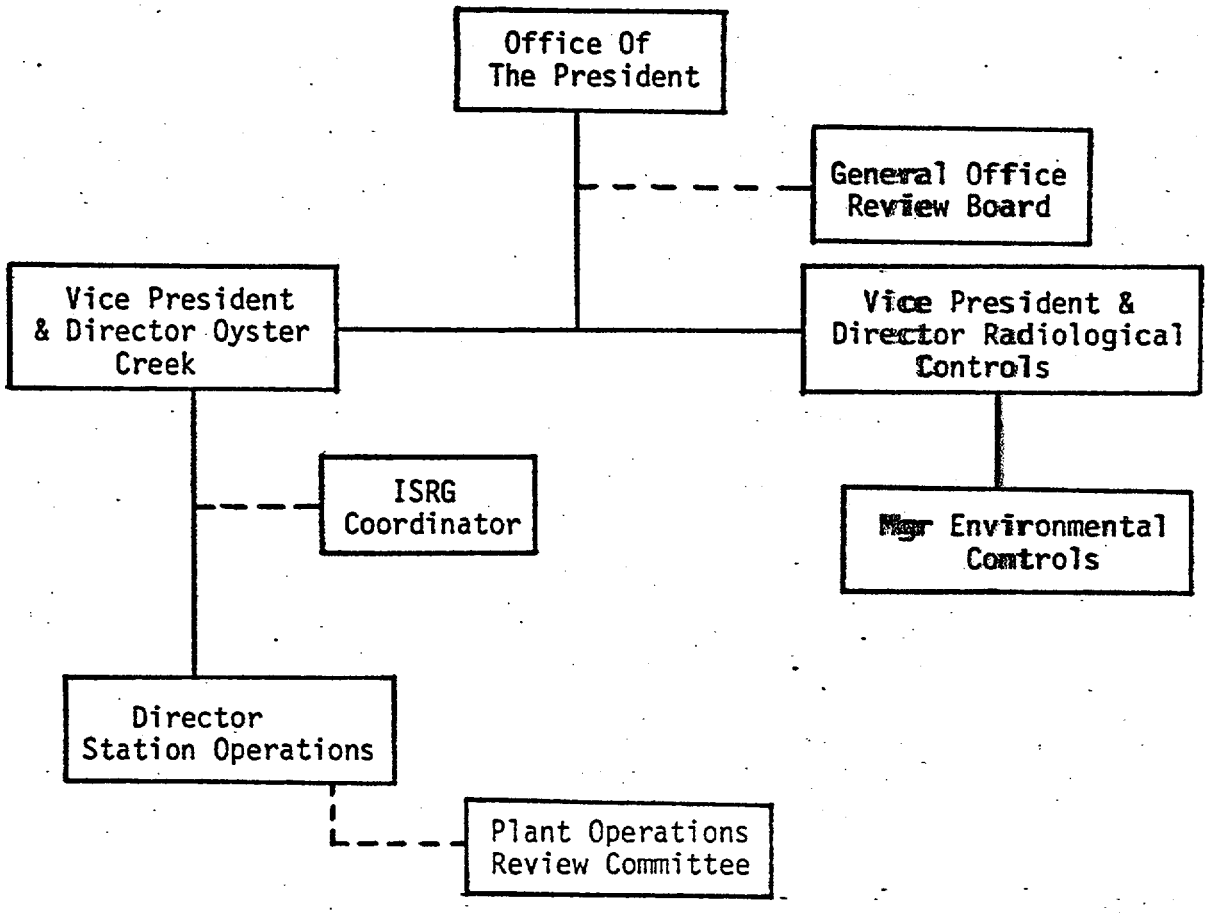
5.2 Organization

The organization of the personnel responsible for implementation, audit, and review of the OCETS is shown in Figure 5-1.

5.3 Review and Audit

Independent audit and review functions for environmental matters are the responsibility of the GPUNC Manager, Environmental Controls. This department reports directly to the Vice President and Director Radiological & Environmental Controls and is independent of line responsibility for the operation of the plant. The independent reviews and audits of the OCETS will be carried out by personnel from the Environmental Controls Department or by other personnel from GPUNC, GPUSC, outside contractors or consultants at the request of the Environmental Controls Department.

When individuals in the Environmental Controls Department of GPUNC perform any function relating to the OCETS other than independent audit and review, the Vice President and Director of Oyster Creek will ensure that an independent review and audit of that work is performed by another individual in the Environmental Controls Department or some other who is not directly responsible for the specific activity being reviewed and audited.



Legend: _____ Direct Responsibility

-------- Advisory Capacity

FIGURE 5-1

ORGANIZATION FOR THE IMPLEMENTATION OF
THE ENVIRONMENTAL TECHNICAL SPECIFICATIONS

The audits and reviews will be performed as required or requested but in no case less than yearly. The results of all reviews and audits will be documented in reports directly to the Vice President and Director Oyster Creek.

Independent audits and reviews will encompass:

A. Coordination of the OCETS with the safety technical specifications to avoid conflicts and maintain consistency.

B. Compliance of station activities and operations with the OCETS.

C. Adequacy of the programs and station procedures which are involved in ensuring the plant is operated in accordance with the OCETS.

D. The proper functioning in accordance with the responsibilities listed in Section 5.1 of the OCETS.

E. Proposed changes to the OCETS and the evaluation of the impacts resulting from the changes.

F. Proposed written procedures, as described in Section 5.5.1 and 5.5.3 and proposed changes thereto which affect the environmental impact of the plant.

G. Proposed changes or modifications to plant systems or equipment and a determination of the environmental impact resulting from the changes.

H. Adequacy of the procedures described in Section 5.5.2 and the results obtained and conclusions drawn from the monitoring programs and special studies involved in the OCETS.

I. Adequacy of investigations of violations of the OCETS and adequacy of and implementation of the recommendations to prevent recurrence of the violations.

5.4 Action To Be Taken If A Limiting Condition For Operation Is Exceeded

5.4.1 Any remedial action permitted by the OCETS will be taken until the limiting conditions can be met.

5.4.2 An investigation of the circumstances surrounding the violation of the limiting condition for operation will be initiated by the Director Station Operations and reviewed in accordance with Section 5.3.

5.4.3 A report of each occurrence of a violation of a limiting condition for operation of the OCETS will be prepared as specified in Section 5.6.2.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 59 TO PROVISIONAL OPERATING LICENSE NO. DPR-16

GPU NUCLEAR CORPORATION AND

JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION UNIT NO. 1

DOCKET NO. 50-219

INTRODUCTION

By letter dated October 2, 1981, as supplemented December 4 and 16, 1981, Jersey Central Power & Light Company (JCP&L) requested that the license be amended to reflect that General Public Utilities Nuclear Corporation (GPUNC) would replace JCP&L as the licensee authorized to operate the Oyster Creek Nuclear Generating Station (the facility). The amendment would revise the Provisional Operating License No. DPR-16, and change Appendix A Section 6 and Appendix B Section 5, to reflect this change in authority.

DISCUSSION AND EVALUATION

Jersey Central Power & Light Company proposes to amend Provisional Operating Licens No. DPR-16 to permit the GPU to replace JCP&L as the sole entity authorized to operate the facility. Changing the name in the license which identifies the authorized operator of the facility has no adverse impact on safety issues related to plant operation so long as the authorized operator is qualified from a technical and managerial standpoint to adequately operate Oyster Creek. The staff has reviewed technical and managerial qualifications of GPUNC and finds them acceptable to operate Oyster Creek under License No. DPR-16.

In addition, Amendment No. 50 to the Provisional Operating License for Oyster Creek Nuclear Generating Station (DPR-16), issued on September 15, 1980, approved changes requested by JCP&L permitting the establishment of the GPU Nuclear Group (GPUNG). This group combined the technical and managerial resources of JCP&L, Metropolitan Edison Company (Met-Ed), and the GPU Service Corporation generation divisions into a single organizational entity. GPUNG was formed for the express purpose of managing nuclear affairs for GPU. In its application for approval of the GPUNG, JCP&L stated that the ultimate goal was to form a GPUNC which would replace GPUNG. Staff evaluation of the requested change to GPUNG, approved by Amendment No. 50, was performed

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with the knowledge that a further request would be forthcoming to change from the GPUNG to the GPUNC consisting of the technical and management resources of GPUNG organized into a separate corporate structure. JCP&L and GPUNC now request this change from GPUNG to GPUNC.

Under the terms of the requested change, the operating license would recognize GPUNC as the legal entity who will provide the technical and managerial resources for the continued safe operation of Oyster Creek. The requested change involves no change in the ownership of Oyster Creek and no changes to the facility. However, operational control of Oyster Creek would shift from JCP&L to GPUNC.

GPUNC would not own or finance nuclear or other utility assets. Under the proposed arrangement, financial responsibility for Oyster Creek operation would remain with GPU's operating subsidiary JCP&L. Under this proposed agreement, JCP&L would provide GPUNC with the necessary financing to cover the costs of operating Oyster Creek.

All of the existing functions of GPUNG, as approved by Amendment No. 50 to the Provisional Operating License, are included in the new organization. The onsite groups will continue to receive backup support from GPUNC. So far as operation of Oyster Creek is concerned, the only changes are in position titles. Personnel remain the same. The requested changes to Appendices A and B of the Technical Specifications would reflect these changes to the position titles. No changes to the qualifications of any positions delineated in Appendices A and B are involved.

Based on the above evaluation and our review of JCP&L's submittal, we find the request to replace JCP&L with GPUNC as the licensee to operate Oyster Creek is acceptable.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal, need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 29, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-219

GPU NUCLEAR CORPORATION AND

JERSEY CENTRAL POWER & LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 59 to Provisional Operating License No. DPR-16, issued to Jersey Central Power & Light Company (the licensee), which revised the Provisional Operating License and Technical Specifications for operation of the Oyster Creek Nuclear Generating Station (the facility) located in Ocean County, New Jersey. This amendment shall be effective January 1, 1982.

The amendment revises the Provisional Operating License and the Technical Specifications to reflect that GPU Nuclear Corporation is to replace Jersey Central Power & Light Company as the licensee authorized to operate the Oyster Creek Nuclear Generating Station.

The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

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For further details with respect to this action, see (1) the application for amendment dated October 2, 1981, and supplements thereto dated December 4 and 16, 1981, (2) Amendment No. 59 to License No. DPR-16, and (3) the Commission's related Safety Evaluation. All of the items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W. Washington, D. C. 20555, and at 101 Washington Street, Toms River, New Jersey 08753. A single copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 29th day of December, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter A. Paulson

Walter A. Paulson, Acting Chief
Operating Reactors Branch #5
Division of Licensing

(10-80)

U.S. NUCLEAR REGULATORY COMMISSI

REACTOR FACILITY FEE DETERMINATION

1. FEE FORM TYPE (Check one)
- [x] PRELIMINARY
- [ ] FINAL

INSTRUCTIONS. Fill-in items 1 through 14, as applicable, and send the original copy to the License Fee Management Branch.

2. DOCKET NUMBER(S): 50-219

3. ACCESSION NUMBER: 8110150413

4. LICENSEE: Jersey Central

5. PLANT NAME AND UNIT(S): Oyster Creek

6. DATE OF APPLICATION: 10/2/81

7. FEE REMITTED: [x] YES [ ] NO

8. LICENSEE FEE DETERMINATION

CLASS I	CLASS II	CLASS III	CLASS IV	CLASS V	CLASS VI	EXEMPT	NONE
	✓						

9. SUBJECT: transfer authority to possess, use and operate Oyster Creek to PSE&G

10. TAC NUMBER ASSIGNED (if available): 47363

11. APPROVAL: [x] LETTER [ ] ORDER

AMENDMENT NUMBER(S): 59

DATE OF ISSUANCE: 12/29/81

12. NRC FEE DETERMINATION

- [x] The above application has been reviewed in accordance with Section 170.22 of Part 170 and is properly categorized.
- [ ] The above application has been reviewed in accordance with Section 170.22 of Part 170 and is incorrectly classified.

Fee should be class(es):

JUSTIFICATION FOR CLASSIFICATION OR RECLASSIFICATION:

- [ ] This application is a Class _____ type of action and is exempt from fees because it is:
  - [ ] Filed by a nonprofit educational institution.
  - [ ] Filed by a Government agency and is not for a power reactor.
  - [ ] For a Class I, II, or III amendment which results from an NRC request dated _____ for the application and the amendment is to simplify or clarify License or Technical Specifications; has only minor safety significance; and is being issued for the convenience of NRC *(must meet all of the criteria)*.
  - [ ] Other *(State reason therefor)*

HS 10/22/81

S Lombardo — 12-1-81 Date

*13. SIGNATURE *(Branch Chief)*: Thomas V. Wambach for D.M. Crutchfield — DATE: 12/2/81

14. FINAL CERTIFICATION: The preliminary fee determination has been reassessed and is hereby affirmed.
SIGNATURE *(Project Manager or Branch Chief)*: Walter A. Paulson for D.M. CRUTCHFIELD — DATE: 12/30/81

FOR LICENSE FEE MANAGEMENT BRANCH USE ONLY *(All others do not write below this line)*

The above exemption request has been reviewed and is hereby accepted as being exempt.
SIGNATURE *(Chief, LFMB)* — DATE

DISTRIBUTION BY LFMB

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